



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

July 10, 2009

Mr. Timothy J. O'Connor  
Site Vice President  
Monticello Nuclear Generating Plant  
Northern States Power Company - Minnesota  
2807 West County Road 75  
Monticello, MN 55362-9637

SUBJECT: MONTICELLO NUCLEAR GENERATING PLANT (MNGP) - ISSUANCE OF AMENDMENT REGARDING COMPLETION TIME TO RESTORE A LOW-PRESSURE EMERGENCY CORE COOLING SUBSYSTEM TO OPERABLE STATUS (TAC NO. MD9170)

Dear Mr. O'Connor:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 162 to Renewed Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated June 26, 2008, as supplemented by letters dated January 27 and July 2, 2009.

The amendment revises the MNGP TS, changing the Required Actions and Completion Times in TS 3.5.1, "Emergency Core Cooling System [ECCS]," to allow a 72-hour completion time to restore a low-pressure ECCS subsystem to operable status after discovery of two low-pressure ECCS subsystems inoperable.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in cursive script that reads "Karl D. Feintuch".

Karl D. Feintuch, Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-263

Enclosures:

1. Amendment No. 162 to DPR-22
2. Safety Evaluation

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY - MINNESOTA\*

DOCKET NO. 50-263

MONTICELLO NUCLEAR GENERATING PLANT

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 162  
License No. DPR-22

1. The U. S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Nuclear Management Company, LLC\* (the licensee), dated June 26, 2008, as supplemented by letters dated January 27 and July 2, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Renewed Facility Operating License No. DPR-22 is hereby amended to read as follows:

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\*On September 22, 2008, Nuclear Management Company, LLC, transferred its operating authority to its parent, Northern States Power Company, a Minnesota corporation (NSPM) on that date. By letter dated September 3, 2008 (Accession No. ML082470648), NSPM stated that it accepts responsibility for all actions before the NRC staff which were previously initiated or addressed by Nuclear Management Company.

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 162, are hereby incorporated in the license. NSPM shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "Lois M. James". The signature is written in a cursive style with a large initial "L".

Lois M. James, Chief  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment: Changes to Renewed Facility Operating License  
and Technical Specifications

Date of Issuance: July 10, 2009

ATTACHMENT TO LICENSE AMENDMENT NO. 162  
RENEWED FACILITY OPERATING LICENSE NO. DPR-22  
DOCKET NO. 50-263

Replace the following page of Renewed Facility Operating License DPR-22 with the attached revised page. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

REMOVE

INSERT

3

3

Replace the following pages of Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

INSERT

3.5.1-2

3.5.1-2

3.5.1-3

3.5.1-3

3.5.1-4

3.5.1-4

3.5.1-5

3.5.1-5

3.5.1-6

3.5.1-6

3.5.1-7

3.5.1-7

2. Pursuant to the Act and 10 CFR Part 70, NSPM to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operations, as described in the Final Safety Analysis Report, as supplemented and amended, and the licensee's filings dated August 16, 1974 (those portions dealing with handling of reactor fuel) and August 17, 1977 (those portions dealing with fuel assembly storage capacity);
  3. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NSPM to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
  4. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NSPM to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
  5. Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to possess, but not separate, such byproduct and special nuclear material as may be produced by operation of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission, now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
1. Maximum Power Level  
NSPM is authorized to operate the facility at steady state reactor core power levels not in excess of 1775 megawatts (thermal).
  2. Technical Specifications  
The Technical Specifications contained in Appendix A, as revised through Amendment No. 162, are hereby incorporated in the license. NSPM shall operate the facility in accordance with the Technical Specifications.
  3. Physical Protection  
NSPM shall implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. One LPCI pump in both LPCI subsystems inoperable.	C.1 Restore one LPCI pump to OPERABLE status.	7 days
D. Two LPCI subsystems inoperable for reasons other than Condition C or H.	D.1 Restore one LPCI subsystem to OPERABLE status.	72 hours
E. One Core Spray subsystem inoperable.  <u>AND</u>  One LPCI subsystem inoperable.  <u>OR</u>  One or two LPCI pump(s) inoperable.	E.1 Restore Core Spray subsystem to OPERABLE status.  <u>OR</u>  E.2 Restore LPCI subsystem to OPERABLE status.  <u>OR</u>  E.3 Restore LPCI pump(s) to OPERABLE status.	72 hours  72 hours  72 hours
F. Both Core Spray subsystems inoperable.	F.1 Restore one Core Spray subsystem to OPERABLE status.	72 hours
G. Required Action and associated Completion Time of Condition A, B, C, D, E, or F not met.	G.1 Be in MODE 3.  <u>AND</u>  G.2 Be in MODE 4.	12 hours  36 hours
H. Two LPCI subsystems inoperable due to open RHR intertie return line isolation valve(s).	H.1 Isolate the RHR intertie line.	18 hours

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
I. Required Action and associated Completion Time of Condition H not met.	I.1 Be in MODE 2.	6 hours
J. HPCI System inoperable.	J.1 Verify by administrative means RCIC System is OPERABLE.  <u>AND</u> J.2 Restore HPCI System to OPERABLE status.	Immediately  14 days
K. HPCI System inoperable.  <u>AND</u> Condition A, B, or C entered.	K.1 Restore HPCI System to OPERABLE status.  <u>OR</u> K.2 Restore low pressure ECCS injection/spray subsystem(s) to OPERABLE status.	72 hours  72 hours
L. One ADS valve inoperable.	L.1 Restore ADS valve to OPERABLE status.	14 days
M. One ADS valve inoperable.  <u>AND</u> Condition A, B, or C entered.	M.1 Restore ADS valve to OPERABLE status.  <u>OR</u> M.2 Restore low pressure ECCS injection/spray subsystem(s) to OPERABLE status.	72 hours  72 hours

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>N. Required Action and associated Completion Time of Condition J, K, L, or M not met.</p> <p><u>OR</u></p> <p>Two or more ADS valves inoperable.</p> <p><u>OR</u></p> <p>HPCI System or one or more ADS valves inoperable and Condition D, E, F, or H entered.</p>	<p>N.1 Be in MODE 3.</p> <p><u>AND</u></p> <p>N.2 Reduce reactor steam dome pressure to <math>\leq 150</math> psig.</p>	<p>12 hours</p> <p>36 hours</p>
<p>O. Two or more low pressure ECCS injection/spray subsystems inoperable for reasons other than Condition C, D, E, F, or H.</p> <p><u>OR</u></p> <p>HPCI System and one or more ADS valves inoperable.</p>	<p>O.1 Enter LCO 3.0.3.</p>	<p>Immediately</p>



**SURVEILLANCE REQUIREMENTS**

SURVEILLANCE		FREQUENCY
SR 3.5.1.1	Verify, for each low pressure ECCS injection/spray subsystem, the piping is filled with water from the pump discharge valve to the injection valve.	31 days
SR 3.5.1.2	Verify each ECCS injection/spray subsystem manual, power operated, and automatic valve in the flow path, that is not locked, sealed, or otherwise secured in position, is in the correct position.	31 days
SR 3.5.1.3	Verify ADS pneumatic pressure is as follows for each required ADS pneumatic supply:  a. S/RV Accumulator Bank header pressure $\geq$ 88.3 psig; and  b. Alternate Nitrogen System pressure is $\geq$ 410 psig.	31 days
SR 3.5.1.4	-----NOTE----- Only required to be met in MODE 1. -----  Verify the RHR System intertie return line isolation valves are closed.	31 days
SR 3.5.1.5	Verify correct breaker alignment to the LPCI swing bus.	31 days
SR 3.5.1.6	Verify each recirculation pump discharge valve cycles through one complete cycle of full travel or is de-energized in the closed position.	In accordance with the Inservice Testing Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY												
SR 3.5.1.7	<p>Verify the following ECCS pumps develop the specified flow rate against a system head corresponding to the specified reactor to containment pressure.</p> <table border="1"> <thead> <tr> <th><u>System</u></th> <th><u>Flow Rate</u></th> <th><u>No. of Pumps</u></th> <th><u>System Head Corresponding to a Reactor to Containment Pressure of</u></th> </tr> </thead> <tbody> <tr> <td>Core Spray</td> <td>≥ 2800 gpm</td> <td>1</td> <td>≥ 130 psi</td> </tr> <tr> <td>LPCI</td> <td>≥ 3870 gpm</td> <td>1</td> <td>≥ 20 psi</td> </tr> </tbody> </table>	<u>System</u>	<u>Flow Rate</u>	<u>No. of Pumps</u>	<u>System Head Corresponding to a Reactor to Containment Pressure of</u>	Core Spray	≥ 2800 gpm	1	≥ 130 psi	LPCI	≥ 3870 gpm	1	≥ 20 psi	In accordance with the Inservice Testing Program
<u>System</u>	<u>Flow Rate</u>	<u>No. of Pumps</u>	<u>System Head Corresponding to a Reactor to Containment Pressure of</u>											
Core Spray	≥ 2800 gpm	1	≥ 130 psi											
LPCI	≥ 3870 gpm	1	≥ 20 psi											
SR 3.5.1.8	<p>-----NOTE----- Not required to be performed until 12 hours after reactor steam pressure and flow are adequate to perform the test. -----</p> <p>Verify, with reactor steam dome pressure ≤ 1025.3 psig and ≥ 950 psig, the HPCI pump can develop a flow rate ≥ 2700 gpm against a system head corresponding to reactor pressure.</p>	In accordance with the Inservice Testing Program												
SR 3.5.1.9	<p>-----NOTE----- Not required to be performed until 12 hours after reactor steam pressure and flow are adequate to perform the test. -----</p> <p>Verify, with reactor pressure ≤ 165 psig, the HPCI pump can develop a flow rate ≥ 2700 gpm against a system head corresponding to reactor pressure.</p>	24 months												

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.5.1.10	<p>-----NOTE----- Vessel injection/spray may be excluded. -----</p> <p>Verify each ECCS injection/spray subsystem actuates on an actual or simulated automatic initiation signal.</p>	24 months
SR 3.5.1.11	<p>-----NOTE----- Valve actuation may be excluded. -----</p> <p>Verify the ADS actuates on an actual or simulated automatic initiation signal.</p>	24 months
SR 3.5.1.12	<p>-----NOTE----- Not required to be performed until 12 hours after reactor steam flow is adequate to perform the test. -----</p> <p>Verify each ADS valve opens when manually actuated.</p>	24 months
SR 3.5.1.13	Verify automatic transfer capability of the LPC1 swing bus power supply from the normal source to the backup source.	24 months



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 162 TO

RENEWED FACILITY OPERATING LICENSE NO. DPR-22

NORTHERN STATES POWER COMPANY\*

MONTICELLO NUCLEAR GENERATING PLANT

DOCKET NO. 50-263

1.0 INTRODUCTION

By application dated June 26, 2008 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML081830500), as supplemented by letter dated January 27 and July 2, 2009 (ADAMS Accession Nos. ML090280576 and ML091871018), Nuclear Management Company, LLC (NMC\*), proposed changes to the Technical Specifications (TS) of the Monticello Nuclear Generating Plant (MNGP). The licensee proposed to amend the TS by (1) revising existing Condition D of Specification 3.5.1, "ECCS [Emergency Core Cooling System] – Operating," to apply to two entire Low-Pressure Core Injection (LPCI) subsystems being inoperable (currently, the Condition only applies when two LPCI subsystems are inoperable due to inoperable injection paths); (2) adding a new Condition E to provide a 72-hour Completion Time when one Core Spray subsystem and one LPCI subsystem (or one or two LPCI pump(s) are inoperable); (3) adding a new Condition F to provide a 72-hour Completion Time when both Core Spray subsystems are inoperable; and (4) adding editorial changes to the numbering to reflect the new Conditions.

The supplements of January 27 and July 2, 2009, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination (73 FR 46930, dated August 12, 2008).

2.0 REGULATORY EVALUATION

The licensee proposed to change the Required Actions and Completion Times of TS 3.5.1 to more closely match the same in the Standard Technical Specifications for pressurized-water reactors. This proposed amendment is based upon MGNP Loss-of-Coolant Accident (LOCA) analysis and ECCS single failure analysis.

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\*On September 22, 2008, Nuclear Management Company, LLC, transferred its operating authority to its parent, Northern States Power Company, a Minnesota corporation (NSPM). By letter dated September 3, 2008 (Accession No. ML082470648), NSPM stated that it accepts responsibility for all actions before the NRC staff which were previously initiated or addressed by Nuclear Management Company.

The licensee cited amendments for several plants as precedents for this TS change. Dresden 2 and 3 and Quad Cities 1 and 2 have amendments previously approved by the NRC that allowed for 72-hour completion times. Duane Arnold also had a license amendment approved to allow one entire LPCI subsystem, or two Residual Heat Removal pump flow paths to be inoperable for 72 hours. Duane Arnold was also allowed to have both Core Spray (CS) subsystems be inoperable for 72 hours. Limerick is another plant with similar ECCS setup and with Required Actions and Completion Times similar to what is proposed for MNGP.

Monticello Nuclear Generating Plant was designed largely before publication of the 70 General Design Criteria (GDC) for Nuclear Power Plant Construction Permits proposed by the Atomic Energy Commission (AEC) for public comment in July 1967, and constructed prior to the 1971 publication of Appendix A, "General Design Criteria for Nuclear Power Plants", to 10 CFR Part 50. As such, MNGP was not licensed to the Appendix A, GDC. However, MNGP's Updated Safety Analysis Report (USAR), Section 1.2, lists the principal design criteria for the design, construction, and operation of MNGP. USAR Appendix E provides a plant comparative evaluation to the 70 proposed AEC design criteria. The licensee concluded that MNGP conforms to the intent of the 70 proposed GDCs. Specifically, for the ECCS:

GDC Criterion 35 - Emergency core cooling. A system to provide abundant emergency core cooling. The system safety function shall be to transfer heat from the reactor core following any loss of reactor coolant at a rate such that (1) fuel and clad damage that could interfere with continued effective core cooling is prevented, and (2) clad metal-water reaction is limited to negligible amounts.

Suitable redundancy in components and features, and suitable interconnections, leak detection, isolation, and containment capabilities shall be provided to assure that for onsite electric power system operation (assuming offsite power is not available) and for offsite electric power system operation (assuming onsite power is not available) the system safety function can be accomplished, assuming a single failure.

Proposed GDC Criterion 38 - Reliability and Testability of Engineered Safety Features. All engineered safety features shall be designed to provide high functional reliability and ready testability. In determining the suitability of a facility for a proposed site, the degree of reliance upon and acceptance of the inherent and engineered safety afforded by the systems, including engineered safety features, will be influenced by the known and the demonstrated performance capability and reliability of the systems, and by the extent to which the operability of such systems can be tested and inspected where appropriate during the life of the plant.

The NRC staff also used the regulations at 10 CFR 50.46, "Acceptance criteria for emergency core cooling systems for light-water nuclear power reactors," and 10 CFR 50, Appendix K, "ECCS Evaluation Models," in conjunction with Criterion 35 of Appendix A to evaluate the application.

### 3.0 TECHNICAL EVALUATION

The proposed amendment would revise the Required Actions and Completion Times for TS 3.5.1 which apply to the ECCS. Approval of this proposed amendment would allow possible restoration of inoperable subsystems, precluding unwarranted reactor shutdown. Specifically, the licensee proposed to revise TS 3.5.1, Condition D, to apply to two LPCI subsystems being inoperable for reasons other than Conditions C or H. The current Condition D applies

specifically to inoperable subsystems due to inoperable injection paths. The licensee also proposed to add Condition E to allow one LPCI subsystem or one or two LPCI pumps in conjunction with one CS subsystem to be inoperable with an action completion time of 72 hours. Finally, the licensee proposed to add Condition F to allow both CS subsystems to be inoperable with an action completion time of 72 hours.

The ECCS provides makeup water to be injected into the core during accident scenarios. The low-pressure ECCS provides injection of high volumes of water while the reactor is at low pressure. The low pressure ECCS consists of the LPCI and CS systems. The current MNGP TS allows for two LPCI subsystems to be inoperable only for inoperable injection paths. When two LPCI subsystems are inoperable for any other reason, the plant must follow the requirements of Limiting Condition for Operation (LCO) 3.0.3. In addition, the plant must similarly follow LCO 3.0.3 when one LPCI subsystem and one CS subsystem are inoperable or when both CS subsystems are inoperable. The approval of this proposed amendment would allow for a 72-hour completion time to restore a low pressure ECCS subsystem back to operable status when two low-pressure ECCS subsystems are found to be inoperable.

In the application the licensee identified the single failures assumed in the USAR and evaluated the remaining ECCS available for mitigation. The licensee concluded that the proposed TS changes and the resulting equipment out-of-service combinations are all bounded by the existing MNGP ECCS single failure analysis. The MNGP ECCS/LOCA analysis is based on the General Electric (GE) SAFER/GESTR methodology and associated code set, previously approved by the Nuclear Regulatory Commission (NRC) (GE topical report NEDE-23785-1-P-A, "The GESTR-LOCA and SAFER Models for the Evaluation of the Loss-of-Coolant Accident, Vol. 3, SAFER/GESTR Application Methodology," Revision 1, October 1984). The NRC staff finds that the limiting LOCA scenarios were analyzed with the limiting single failures and as a result MNGP complies with 10 CFR 50.46 and 10 CFR 50, Appendix K, requirements. Therefore, the NRC staff concludes that the proposed TS changes are supported by the licensee's safety analysis.

According to GDC 35, a system shall be provided to provide abundant emergency core cooling; the system shall transfer heat from the reactor core following any loss of reactor coolant to prevent fuel and clad damage and clad metal-water reaction. The system is also required to be redundant and be able to accomplish its task assuming a single failure. MNGP has shown through its analysis that the ECCS system would still be able to meet this criterion in the event of a LOCA assuming single failure and inoperable ECCS subsystems.

According to GDC 38, all engineered safety features shall be highly reliable and readily testable. The licensee has demonstrated the reliability of the MNGP ECCS and shown that it is comparable to similar plants' ECCS reliability.

A 72-hour completion time limit has been shown to be acceptable. While there is adequate assurance that protection will be available with two inoperable ECCS subsystems and a single failure, another single failure could present a situation where adequate core cooling may not be available during a design-basis LOCA. Design-basis LOCAs are very low probability events. Multiple failures go beyond a design-basis LOCA analysis and are considered an even lower probability event. Because of the low probability of a design-basis LOCA and multiple failures, a 72-hour completion time limit has been shown to be acceptable for bringing an inoperable ECCS subsystem back to operability.

The Emergency Service Water (ESW) system is used to cool the ECCS room coolers. Each division of ECCS has its own room cooler supplied by a separate subsystem of the ESW. When a subsystem of ESW becomes inoperable, one division of ECCS room coolers is consequently considered inoperable, resulting in a division of ECCS inoperable. According to TS 3.7.2, MNGP is allowed 72 hours to return the ESW to operability. According to TS 3.5.1, Condition M, the plant must be in Mode 4 in 37 hours due to LCO 3.0.3. This does not allow for any time to repair the ESW system and as such the plant will be shut down. This proposed amendment would allow for ESW to be inoperable and would give a 72 hour completion time to restore operability to the ESW and a separate LCO would require the ECCS to be restored to operability within 72 hours by restoring the ESW.

Previously, other licensees had shown that they will still have the ability to safely shut down and cool the plant during a LOCA given two low-pressure ECCS subsystems inoperable and were approved a 72-hour completion time to return one subsystem to operable.

Design-basis LOCAs are very low probability events. MNGP's evaluation showed that the plant could still be safely shut down and cooled during a LOCA with two ECCS subsystems inoperable. Therefore, a 72-hour completion time is acceptable based on the low probability of such an event.

In summary of the above evaluation, the NRC staff finds the proposed changes to the TS acceptable in that the licensee has provided adequate justification by using MNGP's licensing basis ECCS/LOCA analysis. The NRC staff determines that, based on such analysis, the proposed changes will not adversely affect MNGP's ability to comply with applicable regulatory requirements.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendment. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes requirements with respect to the installation and use of facility components located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no-significant-hazards considerations, and there has been no public comment on the finding (73 FR 46930). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 6.0 CONCLUSION

The NRC staff has concluded, on the basis of the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by

operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Joshua Miller

Dated: July 10, 2009



Mr. Timothy J. O'Connor  
Site Vice President  
Monticello Nuclear Generating Plant  
Northern States Power Company - Minnesota  
2807 West County Road 75  
Monticello, MN 55362-9637

July 10, 2009

**SUBJECT: MONTICELLO NUCLEAR GENERATING PLANT (MNGP) - ISSUANCE OF AMENDMENT REGARDING COMPLETION TIME TO RESTORE A LOW-PRESSURE EMERGENCY CORE COOLING SUBSYSTEM TO OPERABLE STATUS (TAC NO. MD9170)**

Dear Mr. O'Connor:

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A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,  
**/RA/**  
Karl D. Feintuch, Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-263

Enclosures:

1. Amendment No. 162 to DPR-22
  2. Safety Evaluation
- cc w/encls: Distribution via ListServ

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\*Safety evaluation input transmitted by memo on date shown.

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