UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 21, 2009

Mr. Charles G. Pardee
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: BRAIDWOOD STATION, UNITS 1 AND 2, AND BYRON STATION, UNIT NOS. 1 AND 2 – CORRECTION PAGES TO AMENDMENT NOS. 158 AND 163 (TAC NOS. MD8515, MD8516, MD8517, AND MD8518)

Dear Mr. Pardee:

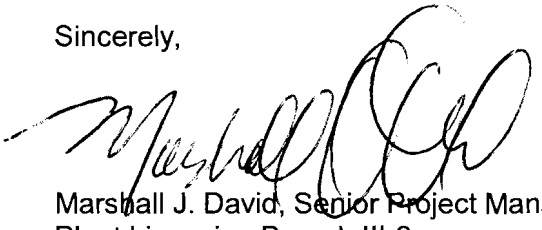
On March 26, 2009, the Nuclear Regulatory Commission (NRC, the Commission) issued Amendment No. 158 to Facility Operating License No. NPF-72 and Amendment No. 158 to Facility Operating License No. NPF-77 for the Braidwood Station, Units 1 and 2 (Braidwood), and Amendment No. 163 to Facility Operating License No. NPF-37 and Amendment No. 163 to Facility Operating License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2 (Byron), respectively.

The amendments revised Technical Specifications (TSs) 5.5.6, "Pre-Stressed Concrete Containment Tendon Surveillance Program," and 5.6.8, "Tendon Surveillance Report," for consistency with the requirements of Title 10 *Code of Federal Regulations* (10 CFR) Section 50.55a, "Codes and standards," paragraph (g)(4) for components classified as American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) Class CC, by replacing the reference to the specific ASME Code year for the tendon surveillance program with a requirement to use the applicable ASME Code and addenda as required by 10 CFR 50.55a.

Your staff recently informed us that TS 5.5.6 for Braidwood, and TS 5.5.6 for Byron, were issued with an error. Specifically, the references to Surveillance Requirement (SR) 3.0.2 had not been removed from these TSs as requested in your application dated April 9, 2008 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML081010540), as supplemented by letter dated October 1, 2008 (ADAMS Accession No. ML082750420). In its Safety Evaluation for Amendment Nos. 158 and 163, the NRC staff had found the removal of the references to SR 3.0.2 from TS 5.5.6 for Braidwood, and TS 5.5.6 for Byron to be acceptable. Therefore, we are issuing the enclosed corrected TS 5.5.6 for Braidwood, and TS 5.5.6 for Byron.

We apologize for any inconvenience this may have caused.

Sincerely,



Marshall J. David, Senior Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-456, STN 50-457,
STN 50-454, and STN 50-455

Enclosures:
1. Corrected TS page 5.5-5 for Braidwood
2. Corrected TS page 5.5-5 for Byron

cc w/encls: Distribution via ListServ

5.5 Programs and Manuals

5.5.6 Pre-Stressed Concrete Containment Tendon Surveillance Program

This program provides controls for monitoring any tendon degradation in pre-stressed concrete containments, including effectiveness of its corrosion protection medium, to ensure containment structural integrity. The program shall include baseline measurements prior to initial operations. The Tendon Surveillance Program, inspection frequencies, and acceptance criteria shall be in accordance with Section XI, Subsection IWL of the ASME Boiler and Pressure Vessel Code and applicable addenda as required by 10 CFR 50.55a, except where an alternative, exemption, or relief has been authorized by the NRC. Determining pre-stressing forces for inspections shall be consistent with the recommendations of Regulatory Guide 1.35.1, July 1990.

The provisions of SR 3.0.3 are applicable to the Tendon Surveillance Program inspection frequencies.

5.5.7 Reactor Coolant Pump Flywheel Inspection Program

This program shall provide for the inspection of each reactor coolant pump flywheel in general conformance with the recommendations of Regulatory Position c.4.b of Regulatory Guide 1.14, Revision 1, August 1975.

In lieu of Regulatory Position c.4.b(1) and c.4.b(2), a qualified in-place UT examination over the volume from the inner bore of the flywheel to the circle of one-half the outer radius or a surface examination (MT and/or PT) of exposed surfaces of the removed flywheel may be conducted at approximately 10 year intervals coinciding with the Inservice Inspection schedule as required by ASME Section XI.

5.5 Programs and Manuals

5.5.6 Pre-Stressed Concrete Containment Tendon Surveillance Program

This program provides controls for monitoring any tendon degradation in pre-stressed concrete containments, including effectiveness of its corrosion protection medium, to ensure containment structural integrity. The program shall include baseline measurements prior to initial operations. The Tendon Surveillance Program, inspection frequencies, and acceptance criteria shall be in accordance with Section XI, Subsection IWL of the ASME Boiler and Pressure Vessel Code and applicable addenda as required by 10 CFR 50.55a, except where an alternative, exemption, or relief has been authorized by the NRC. Determining pre-stressing forces for inspections shall be consistent with the recommendations of Regulatory Guide 1.35.1, July 1990.

The provisions of SR 3.0.3 are applicable to the Tendon Surveillance Program inspection frequencies.

5.5.7 Reactor Coolant Pump Flywheel Inspection Program

This program shall provide for the inspection of each reactor coolant pump flywheel in general conformance with the recommendations of Regulatory Position c.4.b of Regulatory Guide 1.14, Revision 1, August 1975.

In lieu of Regulatory Position c.4.b(1) and c.4.b(2), a qualified in-place UT examination over the volume from the inner bore of the flywheel to the circle of one-half the outer radius or a surface examination (MT and/or PT) of exposed surfaces of the removed flywheel may be conducted at approximately 10 year intervals coinciding with the Inservice Inspection schedule as required by ASME Section XI.

We apologize for any inconvenience this may have caused.

Sincerely,

/RA/

Marshall J. David, Senior Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-456, STN 50-457,
STN 50-454, and STN 50-455

Enclosures:
1. Corrected TS page 5.5-5 for Braidwood
2. Corrected TS page 5.5-5 for Byron

cc w/encls: Distribution via ListServ

DISTRIBUTION:
PUBLIC
RidsNrrDorlLpl3-2 Resource
RidsNrrLATHarris Resource
RidsNrrPMByron Resource
RidsRgn3MailCenter Resource
RidsOgcRp Resource
GHill, OIS
LPL3-2 R/F
RidsNrrDorlDpr Resource
RidsNrrPMBraidwood Resource
RidsAcrsAcnw_MailCTR Resource
RidsNrrDeEmcb Resource
GThomas, NRR

ADAMS Accession No. ML091390368 NRR-106

OFFICE	LPL3-2/PM	LPL3-2/LA	LPL3-2/BC
NAME	MDavid	THarris	CGoodwin for RGibbs
DATE	5/21/09	5/20/09	5/21/09

OFFICIAL RECORD COPY