From: James Kim
Sent: Monday, May 11, 2009 9:22 AM
To: 'Lynch, Joseph R'; Lobo, Walter
Subject: RE: Verbal Authorization Script for Pilgrim PRR-19

The script for verbal authorization is shown below.

Thanks
Jim Kim

By letter dated May 1, 2009, as superseded by letter dated May 7, 2009, Entergy Nuclear Operations, Inc., the licensee for Pilgrim Nuclear Power Station, submitted Request for Alternative PRR-19 from certain American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) requirements under the provisions of Title 10 of the Code of Federal Regulations 50.55a(a)(3)(i).

Specifically, the licensee requested permission to implement a weld overlay repair on the Pilgrim RPV-N9A-1 Jet Pump Instrumentation Nozzle weld. The weld overlay being implemented during the unit's spring 2009 refueling outage is a modification to an existing weld overlay on RPV-N9A-1 which was installed in September 1984. The current weld overlay repair is being implemented in accordance with the provisions of ASME Code Cases N-504-3, "Alternate Rules for Repair of Classes 1, 2, and 3 Austenitic Stainless Steel Piping," and N-638-1, "Similar and Dissimilar Metal Welding Using Ambient Temperature Machine GTAW Temper Bead Technique," as amended by the licensee's requested alternative in PRR-19. ASME Code Cases N-504-3 and N-638-1 have been approved for licensee use by the NRC in NRC Regulatory Guide 1.147, "Inservice Inspection Code Case Acceptability, ASME Section XI, Division 1," with one condition on the use of ASME Code Case N-504-3, which was also addressed in the licensee's submittal.

The licensee's proposed alternative with regard to the implementation of the weld overlay repair and the nondestructive examination of the weld overlay were documented in the licensee's May 1, 2009, letter as superseded by the licensee's letter dated May 7, 2009. Specifically, this included information on:

1. the materials with which the weld overlay will be manufactured
2. the materials to which the overlay will be applied
3. the temper bead welding process to be used for the overlay
4. the design requirements the overlay will meet
5. the preservice examination and testing that will be performed on the overlay, and
6. the inservice inspection plan for the repaired location.

Hence, in accordance with 10 CFR 50.55a(a)(3)(i), the licensee provided information to demonstrate that the proposed alternative would maintain an acceptable level of quality and safety with regard to ensuring the integrity of the subject Pilgrim Nuclear Power Station reactor coolant pressure boundary weld under all licensing basis conditions of operation.

The NRC has completed its review of the information provided in the licensee's submittal. The NRC has concluded that the licensee provided adequate information regarding the design features, preservice inspection, and inservice inspection plan for the weld overlay to substantiate its implementation as an acceptable repair. Hence, as Chief of the Office of Nuclear Reactor Regulation's Vessel and Internals Integrity Branch, I hereby recommend authorization of the licensee's implementation of the proposed alternative in accordance with the information provided in the licensee's letter dated May 7, 2009.

E-mail Properties

Mail Envelope Properties ()

Subject: RE: Verbal Authorization Script for Pilgrim PRR-19
Sent Date: 05/11/2009 9:19:59 AM
Received Date: 05/11/2009 9:22:00 AM
From: James Kim

Created By: James.Kim@nrc.gov

Recipients:
jlynch4@entergy.com ('Lynch, Joseph R')
Tracking Status: None
wlobo@entergy.com (Lobo, Walter)
Tracking Status: None

Post Office:

Files	Size	Date & Time
MESSAGE	13188	05/11/2009

Options
Expiration Date:
Priority: olImportanceNormal
ReplyRequested: False
Return Notification: False

Sensitivity: olNormal
Recipients received: