

REQUEST FOR ADDITIONAL INFORMATION 298-2367 REVISION 2

5/4/2009

US-APWR Design Certification

Mitsubishi Heavy Industries

Docket No. 52-021

SRP Section: 15.06.03 - Radiological Consequences of Steam Generator Tube Failure (PWR) 07/1981
Application Section: 15.6.3

QUESTIONS for Reactor System, Nuclear Performance and Code Review (SRSB)

15.06.03-1

Question 15.6.3-1

Question has been deleted.

15.06.03-2

Question 15.6.3-2

Provide the transient curve for DNBR verses time for the SGTR analysis.