

#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

May 27, 2009

LICENSEE: Florida Power & Light

FACILITY: Turkey Point Nuclear Plant, Units 3 and 4

SUBJECT: SUMMARY OF APRIL 24, 2009, CATEGORY 1 MEETING WITH FLORIDA POWER & LIGHT, TO DISCUSS TURKEY POINT NUCLEAR PLANT'S PROPOSED ALTERNATIVE SOURCE TERM LICENSE AMENDMENT REQUEST

On April 24, 2009, a public meeting was held between the Nuclear Regulatory Commission (NRC), and representatives of Florida Power & Light (FPL), Turkey Point, at NRC Headquarters, One White Flint North, 11555 Rockville Pike, Rockville, MD. The purpose of this presubmittal meeting was to achieve a common understanding between the Agency and Turkey Point of the proposed scope of the Turkey Point Alternative Source Term (AST) application, gain clarification regarding regulatory positions on specific issues relating to the AST, and understand the level of detail of information expected in the application.

The licensee presented information on the reason for adopting AST, which are to update the Turkey Point Units 3 and 4 accident dose analyses using a consistent basis for all events as described by Regulatory Guide (RG) 1.183 and to support a future extended power uprate (EPU). During the presentation the staff communicated to the licensee examples of good submittals that FPL should review for precedence. The licensee continued with the presentation by stating the specific aspects of the analyses and content of their submittal. A couple of the specific aspects that will be included in the submittal are Turkey Point specific source term inventories calculated using ORIGEN 2, which considers conservative burnup ranges, enrichment ranges, and power levels (bounds both current operating conditions and future EPU operating conditions), updated control room and offsite X/Q values based upon RG 1.194 and RG 1.145 using recent meteorological data, and the loss-of-coolant accident dose contribution from the emergency core cooling system back-leakage to the refueling water storage tank considering the effects of sump pH, back-leakage temperature reduction, and total iodine distribution within the tank.

The staff questioned what version of ORIGEN was used to calculate the source term. This was to determine the specific methodology that was used to calculate the source term and to clarify that the licensee consistently used one version. The licensee indicated that they used ORIGEN 2.1 and the staff was satisfied with this version. The licensee continued the presentation by stating the proposed technical specification changes that will be included in the AST license amendment request (LAR).

The licensee concluded the presentation by communicating to the NRC the planned schedule for the AST and EPU LARs. Turkey Point plans to submit the AST LAR in June 2009 and the EPU LAR in the 2<sup>nd</sup> quarter of 2010. The staff questioned if the EPU LAR would be submitted after the review of the AST is completed. Turkey Point responded that the EPU would be submitted approximately in June 2010 after the completion of the AST review, assuming a 1 year AST review.

Members of the public were not in attendance. A representative from Turkey Point filled out a Public meeting feedback form and was satisfied with the outcome of the meeting.

A list of meeting attendees is attached. Also, a copy of the licensee's slides can be found in ADAMS, Accession Number ML091160002.

Please direct any inquiries to Jason Paige at 301-415-5888, or <u>Jason.Paige@nrc.gov</u>.

Sincerely, C

Jason C. Paige, Project Manager Plant Licensing Branch II-2 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket Nos. 50-250 and 50-251

Enclosure: List of Attendees

cc w/encl: Distribution via ListServ

#### LIST OF ATTENDEES

#### APRIL 24, 2009, MEETING WITH

#### **FLORIDA POWER & LIGHT**

#### ALTERNATIVE SOURCE TERM REQUEST FOR TURKEY POINT

#### LICENSE AMENDMENT PRE-SUBMITTAL MEETING

NAME	ORGANIZATION
Eva Brown	NRC/NRR/DORL/LPL2-2
Lois James	NRC/NRR/DORL/LPL3-1
Leta Brown	NRC/NRR/DRA/Accident Dose Branch
Tony Nakanishi	NRC/NRR/DSS/Reactor Systems Branch
Aleem Boatright	NRC/NRR/DRA/Accident Dose Branch
Robert Taylor	NRC/NRR/DRA/Accident Dose Branch
John Parillo	NRC/NRR/DRA/Accident Dose Branch
Greg Myers	Florida Power & Light
Liz Abbott	Florida Power & Light
Mark Pope (by phone)	Nuclear Energy Institute
Steve Hale	Florida Power & Light
Brenda Mozafari	NRC/NRR/DORL/LPL2-2
Allen Howe	NRC/NRR/DORL



# ALTERNATIVE SOURCE TERM (AST) Request for License Amendment Pre-Submittal Meeting

# Turkey Point Units 3 and 4 April 24, 2009

# Turkey Point Units 3 and 4 – Alternative Source Term ATTENDEES FOR FPL

- Liz Abbott FPL EPU Director
- Greg Myers FPL EPU Manager
- Steve Hale FPL EPU Manager
- Jim Harrell Contractor (Numerical Applications, Inc.)



## **PURPOSE OF PRE-SUBMITTAL MEETING**

- Achieve common understanding of the proposed scope of the Turkey Point application
- Gain understanding regarding regulatory positions on specific issues
- Understand the level of detail of information expected in the application



- Reason for Adopting Alternate Source Term (AST)
- Scope of Implementation
- Specific Aspects of Analyses
- Overview of Licensing Basis Changes
- Proposed Technical Specification Changes
- Submittal Content
- Schedules
- Summary



## **REASON FOR ADOPTING AST METHODOLGY**

- To update Turkey Point Units 3 and 4 accident dose analyses using a consistent basis for all events as described by Regulatory Guide 1.183
- To obtain margin for Control Room inleakage
- To support a future extended power uprate



## SCOPE OF IMPLEMENTATION

- Full implementation of AST as described by RG 1.183
- Postulated events analyzed
  - Loss-of-Coolant Accident
  - Fuel Handling Accident
    - -- Separate analysis for Containment and Fuel Handling Building Releases
  - Main Steam Line Break
    - -- No Fuel Damage postulated
    - -- Pre-Accident and Concurrent Iodine Spikes



## SCOPE OF IMPLEMENTATION

- Postulated events analyzed (continued)
  - Steam Generator Tube Rupture
    - -- No Fuel Damage postulated
    - -- Pre-Accident and Concurrent lodine Spikes
    - -- Considers pre-trip releases via condenser and post-trip releases via Steam Generator
  - Locked Rotor
    - -- Postulated Fuel Damage
  - Rod Cluster Control Assembly Ejection
    - -- Containment and Secondary Release Events
    - -- Postulated Fuel Damage



## SCOPE OF IMPLEMENTATION

- Postulated events analyzed (continued)
  - Waste Gas Decay Tank Rupture
    - -- Utilizes Exclusion Area Boundary Limit of 0.1 rem TEDE consistent with RIS 2006-04
  - Spent Fuel Cask Drop
    - Postulated fuel damage



## SPECIFIC ASPECTS OF THE ANALYSES

- AST analysis approach in accordance with RG 1.183
- Turkey Point specific source term inventories calculated using ORIGEN 2
  - -- Considers conservative burnup ranges, enrichment ranges and power levels
  - -- Bounds both current operating conditions and future extended power uprate operating conditions (2652 MWt, including calorimetric uncertainties)
- Updated control room and offsite X/Q values based upon Reg. Guide 1.194 and Reg. Guide 1.145 using recent meteorological data



## SPECIFIC ASPECTS OF THE ANALYSES (cont)

- 10% of the total iodine activity in the ECCS leakage outside containment assumed airborne
- LOCA dose contribution from the ECCS back-leakage to RWST considers the effects of sump pH, backleakage temperature reduction, and total iodine distribution within the tank
- Analysis inputs and atmospheric steam releases are set conservatively to bound current plant operating conditions and future extended power uprate operating conditions (2652 MWt, including calorimetric uncertainties)



## SPECIFIC ASPECTS OF THE ANALYSES (cont)

- Analyses support implementation of AST at current power level, independent of EPU
- Preliminary dose analyses support increased Control Room unfiltered inleakage rates
  - -- Control Room inleakage testing performed in August 2003 indicated less than 10 cfm of unfiltered inleakage
  - -- Control Room is shared between Units 3 and 4
  - -- 150 cfm unfiltered inleakage assumed
- Containment sump pH will be confirmed via calculations considering NUREG/CR-5950



## Turkey Point Units 3 and 4 – Alternative Source Term SPECIFIC ASPECTS OF THE ANALYSES (cont)

**Turkey Point AST Evaluation Preliminary Results** 

Event	EAB	LPZ	CR
LOCA	5.66	1.54	4.81
FHA – in Containment	0.91	0.20	1.01
FHA – in Fuel Handling Building	0.91	0.20	3.95
MSLB – Pre-accident iodine spike	0.029	0.024	1.49
MSLB – Concurrent iodine spike	0.046	0.041	1.54
SGTR – Pre-accident iodine spike	0.82	0.18	2.85
SGTR – Concurrent iodine spike	0.28	0.07	1.10

All units are rem TEDE



# Turkey Point Units 3 and 4 – Alternative Source Term SPECIFIC ASPECTS OF THE ANALYSES (cont)

**Turkey Point AST Evaluation Preliminary Results** 

Event	EAB	LPZ	CR
Locked Rotor	0.57	0.60	1.48
RCCA Ejection – Containment	0.88	0.40	2.34
RCCA Ejection – Secondary	0.59	0.56	1.32
Waste Gas Decay Tank Rupture	0.08	0.02	0.04
Spent Fuel Cask Drop	0.26	0.06	2.51

All units are rem TEDE.



## **SPECIFIC ASPECTS OF THE ANALYSES (cont)**

Consideration of High Burnup Rods for Fuel Handling Accident

- To assure that failure of any offloaded fuel assembly is bounded by the Fuel Handling Accident, all of the rods in a single fuel assembly are conservatively assumed to exceed the burnup limits upon which the Reg. Guide 1.183 gap release fractions are based
- Guidance of NUREG/CR-5009 is used to determine the conservative gap release fractions
  - NUREG/CR-5009 endorses the gap release fractions for fuel handling events outlined in Reg. Guide 1.25 with some modification for higher burnups



## OVERVIEW OF ANTICIPATED LICENSING BASIS CHANGES

- The total effective dose equivalent (TEDE) acceptance criterion of 10CFR50.67(b)(2) replaces the previous whole body and thyroid dose guidelines of 10CFR100.11
- New onsite (Control Room) and offsite atmospheric dispersion factors are developed
- Dose conversion factors for inhalation and submersion are from Federal Guidance Reports (FGR) Nos. 11 and 12, respectively



### OVERVIEW OF ANTICIPATED LICENSING BASIS CHANGES (cont)

- A bounding value for control room unfiltered air inleakage was established by increasing the inleakage until the dose acceptance criteria for the limiting event (LOCA) was approached
- A primary coolant specific activity for Dose Equivalent I-131 that is more restrictive than the current Technical Specification limit is utilized
- A steam generator tube leakage rate is used that is more restrictive than the current Technical Specification program limit for primary-to-secondary accident induced leakage



### OVERVIEW OF ANTICIPATED LICENSING BASIS CHANGES (cont)

- A containment leakage value that is more restrictive than the current Technical Specification limit is utilized
- Sump pH control is provided by sodium tetraborate decahydrate (NaTB) baskets
- Containment ESF filter units are not credited
- In accordance with TSTF-490, the Reactor Coolant System specific activity limit for gross radioactivity is changed from 100/E-bar to dose equivalent Xenon-133
  - The 100/E-bar value is directly converted to dose equivalent Xenon-133, so this change is not required to implement AST



## **PROPOSED PHYSICAL PLANT CHANGES**

- The Control Room Emergency Intakes are proposed to be relocated to reduce control room doses and support development of updated control room X/Q values
- Sodium tetraborate decahydrate (NaTB) baskets are proposed to be added to the containment for pH control



## PROPOSED TECHNICAL SPECIFICATION CHANGES

- Definition of Dose Equivalent I-131 in Section 1.10 is revised to reference Federal Guidance Report No. 11 (FGR 11), "Limiting Values of Radionuclide Intake and Air Concentration and Dose Conversion Factors for Inhalation, Submersion, and Ingestion" 1989, as the source of effective dose conversion factors.
- Reactor Coolant System (RCS) specific activity limit for dose equivalent lodine-131 (DE I-131), stated in Limiting Condition for Operation (LCO) 3.4.8.a, is reduced from 1 microcurie per gram to 0.25 microcurie per gram.



## PROPOSED TECHNICAL SPECIFICATION CHANGES (cont)

- Reactor Coolant System specific activity limit for gross radioactivity stated in Limiting Condition for Operation (LCO) 3.4.8.b is changed from 100/E-bar to dose equivalent Xenon-133.
- Terminology used in Limiting Condition for Operation (LCO) 3.7.9 to set the maximum contents of the Waste Gas Decay Tank is clarified to read 'DOSE EQUIVALENT Xe-133.'



## PROPOSED TECHNICAL SPECIFICATION CHANGES (cont)

- Maximum allowable containment leakage rate acceptance criterion stated in TS 6.8.4.h, "Containment Leakage Rate Testing Program," is reduced from 0.25% to 0.20% of containment air weight per day.
- Maximum allowable primary-to-secondary accident induced leakage rate acceptance criterion stated in TS 6.8.4.j.b.2, "Steam Generator Program," is reduced from 500 gpd to 300 gpd through any one steam generator.



## PROPOSED TECHNICAL SPECIFICATION CHANGES (cont)

- A method for controlling the pH of the post-LOCA containment sump solution using sodium tetraborate decahydrate (NaTB) is being proposed. This passive system will consist of baskets of NaTB in the lower regions of the containment. Appropriate Technical Specifications and Surveillance Requirements are proposed for Section 3/4.6 "Containment Systems."
- Operability requirements for emergency containment filter units in Section 3/4.6.3 will be deleted.



# Turkey Point Units 3 and 4 – Alternative Source Term SUBMITTAL CONTENT

- Licensing Technical Report which presents the analysis assumptions and documents conformance with Reg. Guide 1.183
- Table addressing NRC Regulatory Issue Summary 2006-04 issues
- Analyses key input parameters and results tables
- Technical Specification changes
- No Significant Hazards Considerations Evaluation
- Plant modification descriptions



## SCHEDULE

- Planned AST Request for License Amendment (RLA) submittal in June 2009
  - AST RLA provides dose analysis to support EPU
- Planned EPU RLA submittal in 2<sup>nd</sup> quarter of 2010
  - EPU RLA will reference AST analyses provided in AST RLA
  - Second review of dose analyses will not be required for EPU



## SUMMARY

- Addresses control room inleakage margin
- Conformance with RG 1.183
- Addresses NRC RIS 2006-04 concerns
- Utilizes conservative inputs to bound current operating conditions and planned future extended power uprate operating conditions
- Can be implemented at current power level
- Planned submittal in June 2009



#### April 13, 2009

MEMORANDUM TO:	Thomas H. Boyce, Chief Plant Licensing Branch II-2 Division of Operating Reactor Licens	ing
FROM:	Jason Paige, Project Manager Plant Licensing Branch II-2 Division of Operating Reactor Licens	/ <b>RA</b> / ing
SUBJECT:	TURKEY POINT NUCLEAR PLANT, MEETING WITH FLORIDA POWER	UNITS 3 AND 4 - FORTHCOMING & LIGHT COMPANY
DATE & TIME:	Friday, April 24, 2009 10:00 a.m. – 12:00 p.m.	
LOCATION:	U.S. Nuclear Regulatory Commission One White Flint North 11555 Rockville Pike, Room O-6B6 Rockville, Maryland	n (NRC)
PURPOSE:	The purpose of this pre-submittal me understanding between the NRC and proposed scope of the Turkey Point application, gain clarification regardin issues relating to the AST, and unde information expected in the application	eeting is to achieve a common d Florida Power & Light (FPL) of the Alternative Source Term (AST) ng regulatory positions on specific rstand the level of detail of on.
CATEGORY 1: *	This is a Category 1 meeting. The p meeting and will have one or more o NRC after the business portion but b Members of the public who wish to a e-mail the contact listed below.	ublic is invited to observe this pportunities to communicate with the efore the meeting is adjourned. ttend are encouraged to telephone or

MEETING CONTACTS:	Jason Paige, NRR	Brenda Mozafari, NRR
	301-415-5888	301-415-2020
	jason.paige@nrc.gov	brenda.mozafari@nrc.gov

\*Commission's Policy Statement on "Enhancing Public Participation in NRC Meetings" (67 FR 36920), May 28, 2002. The NRC provides reasonable accommodation to individuals with disabilities where appropriate. If you need a reasonable accommodation to participate in a meeting, or need a meeting notice or a transcript or other information from a meeting in another format (e.g., Braille, large print), please notify the NRC's meeting contact. Determinations on requests for reasonable accommodation will be made on a case-by-case basis. PARTICIPANTS: Participants from the NRC include members of the Office of Nuclear Reactor Regulation (NRR).

NRC		FPL		
J. Paige, NRR R. Taylor, NRR L. Brown, NRR	T. Boyce, NRR B. Mozafari, NRR	E. Abbott, FPL G. Myers, FPL J. Harrell, FPL	B. Tomonto, FPL	

Docket Nos. 50-250 and 50-251

Enclosure: Agenda

cc w/enclosure: Distribution via Listserv

#### <u>AGENDA</u>

#### FORTHCOMING MEETING WITH FLORIDA POWER & LIGHT

#### TURKEY POINT NUCLEAR PLANT, UNITS 3 AND 4

#### Meeting on Friday, April 24, 2009

#### <u>10:00 a.m. – 12:00 p.m.</u>

- Introductions
- Reason for Adopting the Alternative Source Term
- Scope of Implementation
- Specific Aspects of the Analyses
- Overview of Changes to the Licensing Basis
- Proposed Technical Specification Changes
- Submittal Content
- Schedules
- Summary
- Public Participation and Adjournment

Members of the public were not in attendance. A representative from Turkey Point filled out a Public meeting feedback form and was satisfied with the outcome of the meeting.

A list of meeting attendees is attached. Also, a copy of the licensee's slides can be found in ADAMS, Accession Number ML091160002.

Please direct any inquiries to Jason Paige at 301-415-5888, or Jason.Paige@nrc.gov.

Sincerely,

/**RA**/

Jason C. Paige, Project Manager Plant Licensing Branch II-2 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket Nos. 50-250 and 50-251

Enclosure: List of Attendees

cc w/encl: Distribution via ListServ

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