UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
611 RYAN PLAZA DRIVE, SUITE 400
ARLINGTON, TEXAS 76011-4005

April 20, 2009

Michael Perito
Vice President, Operations
Entergy Operations, Inc.
River Bend Station
5485 US Highway 61N
St. Francisville, LA 70775

SUBJECT: RIVER BEND STATION - NRC RADIATION SAFETY TEAM INSPECTION REPORT 05000458/2009006

Dear Mr. Perito:

On March 30, 2009, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at your River Bend Station facility. The enclosed Radiation Safety Team inspection report documents the inspection findings which were discussed with you and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license.

The team reviewed selected procedures and records, observed activities, and interviewed personnel. Specifically, the team evaluated the inspection areas within the Radiation Protection Strategic Performance Area that are scheduled for review every two years. These areas are:

- Radiation Monitoring Instrumentation
- Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems
- Radiological Environmental Monitoring Program and Radioactive Material Control Program

This inspection report documents one NRC-Identified violation of very low safety significance (Green). However, because the finding was of very low safety significance and was entered into your corrective action program, the NRC is treating this finding as a non-cited violation consistent with Section VI.A of the NRC Enforcement Policy. If you contest this non-cited violation or its significance, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington D.C. 20555-0001; with copies to the Regional Administrator, U.S. Nuclear Regulatory Commission Region IV, 612 E. Lamar Blvd, Suite 400, Arlington, Texas 76011-4005; the Director, Office of Enforcement, U.S. Nuclear Regulatory Commission, Washington, D. C. 20555-001; and the NRC Resident Inspector at the River Bend Station.

In addition, if you disagree with the characterization of any finding in this report, you should provide a response within 30 days of the date of this inspection report, with the basis for your disagreement, to the Regional Administrator, Region IV, and the NRC Resident Inspector at River Bend Station. The information you provide will be considered in accordance with Inspection Manual Chapter 0305.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

Sincerely,

/RA/ JAMES F. DRAKE *for*

Gregory E. Werner, Chief
Plant Support Branch 2
Division of Reactor Safety

Dockets: 50-458
Licenses: NPF-47

Enclosure:
NRC Inspection Report
w/Attachment: Supplemental Information

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Sunsi Review Completed: Yes ADAMS: ■ Yes □ No Initials:
■ Publicly Available □ Non-Publicly Available □ Sensitive ■ Non-Sensitive

RIV:DRS/PSB2	PSB2	PSB2	C:PSB2	C:DRP/C
LTRicketson/dch	LCCarsonII	DLStearns	GEWerner	GBMiller
/RA/	/RA/	/RA/	/RA/	/RA/
4/20/09	4/10/09	4/9/09	4/20/09	4/20/09

C:PSB2				
GEWerner				
/RA/ JFDrake for				
4/20/09				

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U.S. NUCLEAR REGULATORY COMMISSION
REGION IV

Dockets: 05000458

Licenses: NPF-47

Report No: 05000458/2009006

Licensee: Entergy Operations, Inc.

Facility: River Bend Station

Location: 5485 U.S. Highway 61
St. Francisville, LA

Dates: February 9 through March 30, 2009

Inspectors: L. T. Ricketson, P.E., Senior Health Physicist, - Team Leader
L. C. Carson II, Senior Health Physicist
D. L. Stearns, Health Physicist

Approved By: Gregory E. Werner, Chief
Plant Support Branch 2
Division of Reactor Safety

SUMMARY OF FINDINGS

IR 05000458/2009006; 02/09/09 – 03/30/09; River Bend Station; Radiation Monitoring Instrumentation and Protective Equipment;

The report covered a 49-day period of inspection (five days on site) by a team of three region-based health physics inspectors. Based upon the results of the inspection, the team identified one NRC-Identified violation of very low safety significance (Green). The significance of most findings is indicated by their color (Green, White, Yellow, Red) using IMC 0609, "Significance Determination Process," (SDP). Findings for which the SDP does not apply may be "Green" or be assigned a severity level after NRC management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 4, dated December 2006.

A. NRC-Identified and Self-Revealing Findings

- Green. The team identified a non-cited violation of 10 CFR Part 50, Appendix B, Criterion XVI, "Corrective Action," for the failure to implement required actions to ensure that conditions were promptly corrected. Specifically, on February 10, 2009, during a review of corrective action documents, the inspectors noted that corrective actions for condition report CR-RBS-2007-03034 were inadequate to correct a condition in which an instrument was not treated as measuring and test equipment. The team noted that corrective action was proposed, but not implemented, and the condition report was closed. The condition which prompted the condition report still existed at the time of the inspection. The licensee entered this issue into corrective action program as condition report CR-RBS-2009-00747.

  The failure to implement timely corrective action is a performance deficiency. The finding is greater than minor because if left uncorrected, the performance deficiency would have the potential to lead to a more significant safety concern, such as an improperly calibrated main steam line monitor. The performance deficiency affected the barrier integrity cornerstone in that the proper calibration of the main steam line monitors is necessary to ensure proper isolation of containment in the event of fuel damage. Using Phase 1 worksheet from Manual Chapter 0609, "Significance Determination Process," this finding was determined to have very low safety significance because it did not represent a degradation of the radiological barrier function provided for the control room, auxiliary building, spent fuel pool, or standby gas treatment system; did not represent an actual open pathway in the physical integrity of the reactor containment and heat removal components, and did not involve an actual reduction in function of hydrogen ignitors in the reactor containment. This finding has a crosscutting aspect in the area of human performance associated with the decision making component because the licensee did not use conservative assumptions to demonstrate that the decision to close the condition report with no further action was appropriate (H1.b). (Section 4OA2)

B. Licensee-Identified Violations

None

Report Details

2. RADIATION SAFETY

Cornerstones: Occupational Radiation Safety [OS] and Public Radiation Safety [PS]

2OS3 Radiation Monitoring Instrumentation and Protective Equipment (71121.03)

a. Inspection Scope

This area was inspected to determine the accuracy and operability of radiation monitoring instruments that are used for the protection of occupational workers and the adequacy of the program to provide self-contained breathing apparatus (SCBA) to workers. The team used the requirements in 10 CFR Part 20 and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed:

- Calibration of area radiation monitors associated with transient high and very high radiation areas and post-accident monitors used for remote emergency assessment;
- Calibration of portable radiation detection instrumentation, electronic alarming dosimetry, and continuous air monitors used for job coverage;
- Calibration of whole body counting equipment and radiation detection instruments utilized for personnel and material release from the radiologically controlled area;
- Audits and self-assessments; licensee event reports or special reports, if any were required since the previous inspection;
- Corrective action program reports since the last inspection;
- Licensee action in cases of repetitive deficiencies or significant individual deficiencies;
- Calibration expiration and source response check currency on radiation detection instruments staged for use;
- The licensee's capability for refilling and transporting self-contained breathing apparatus air bottles to and from the control room and operations support center during emergency conditions, status of SCBA staged and ready for use in the plant and associated surveillance records, and personnel qualification and training;

- Qualification documentation for onsite personnel designated to perform maintenance on the vendor-designated vital components, and the vital component maintenance records for self-contained breathing apparatus units;

Specific documents reviewed during this inspection are listed in the attachment.

Either because the conditions did not exist or an event had not occurred, no opportunities were available to review the following items:

- Licensee event reports or special reports

These activities constitute completion of nine of the required nine samples as defined in Inspection Procedure 71121.03-05.

b. Findings

No findings of significance were identified.

2PS1 Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems (71122.01)

a. Inspection Scope

This area was inspected to: (1) ensure that the gaseous and liquid effluent processing systems are maintained so that radiological discharges are properly mitigated, monitored, and evaluated with respect to public exposure; (2) ensure that abnormal radioactive gaseous or liquid discharges and conditions, when effluent radiation monitors are out-of-service, are controlled in accordance with the applicable regulatory requirements and licensee procedures; (3) verify that the licensee's quality control program ensures that the radioactive effluent sampling and analysis requirements are satisfied so that discharges of radioactive materials are adequately quantified and evaluated; and (4) verify the adequacy of public dose projections resulting from radioactive effluent discharges. The team used the requirements in 10 CFR Part 20; 10 CFR Part 50, Appendices A and I; 40 CFR Part 190; the Offsite Dose Calculation Manual (ODCM), and licensee procedures required by the Technical Specifications as criteria for determining compliance.

The team conducted in-office inspection and reviewed:

- Appropriate program documents, procedures and evaluations related to the radiological effluent controls program listed in the attachment to this report;

- The implementation of the Radiological Effluent Controls Program requirements as described in Radiological Effluent Technical Specifications;

- Changes, if any, to the liquid or gaseous radioactive waste system design, procedures, or operation as described in the Updated Final Safety Analysis Report;

- Changes, if any, to the Offsite Dose Calculation Manual made by the licensee since the last inspection;

- Effluent monitoring instrumentation documentation to ensure adequate methods and monitoring of effluents;

- The program for identifying, assessing, and controlling contaminated spills and leaks;

- The annual effluent release reports and the correlation to the environmental monitoring results;

- The results from quality assurance audits.

The team conducted an onsite inspection which included interviewing cognizant licensee personnel, performing walkdowns of facilities and equipment, and observing licensee activities to review:

- The gaseous and liquid discharge system configuration;

- Selected point of discharge effluent radiation monitoring systems and flow measurement devices;

- The observation of selected portions of the routine processing and discharge of radioactive gaseous and liquid effluent (sample collection and analysis) including a selection of radioactive gaseous and liquid waste effluent discharge permits;

- Effluent discharges made with inoperable (declared out-of-service) effluent radiation monitors including the projected doses to members of the public;

- Surveillance test results on non-safety related ventilation and gaseous discharge systems (high efficiency particulate air and charcoal filtration) including the methodology to determine the stack and vent flow rates;

- The identification of non-radioactive systems that have become contaminated, if applicable;

- Effluent monitoring instrument (installed and counting room) maintenance, quality control, and calibration;

- The methods used to determine the isotopes in the plant source term, meteorological dispersion and deposition factors, and hydrogeologic characteristics used in the ODCM and effluent dose calculations including a selection of monthly, quarterly, and annual dose calculations;

- The land-use census;

- Records of abnormal gaseous or liquid discharges, if any, including the evaluation and analysis of events involving spills or discharges, dose assessments to members of the public, required (or voluntary) offsite notifications, and assessments and reporting of abnormal discharges in the Annual Radiological Effluent Release Report;

- Evaluations of discharges from onsite surface water bodies, if any;

- Routine groundwater monitoring results;

- Audits and self-assessments; licensee event reports or special reports, if any were required since the previous inspection;

- The results of the inter-laboratory comparison program;

- Effluent sampling records;

- The calibration of post-accident effluent monitoring instrumentation and expected accident source.

The team reviewed the licensee's program of problem identification and resolution, including:

- Placement of problems identified through audits, self assessments, and monitoring results into the corrective action program and adequacy of immediate and long term corrective actions;

- Problem identification and resolution follow-up activities;

- Identification of repetitive deficiencies or significant individual deficiencies in problem identification and resolution identified by the licensee's self-assessment activities.

Specific documents reviewed during this inspection are listed in the attachment.

Either because the conditions did not exist or an event had not occurred, no opportunities were available to review the following items:

- Licensee event reports or special reports

These activities constitute completion of three of the required three samples, as defined in Inspection Procedure 71122.01-05.

b. Findings

No findings of significance were identified.

2PS3 Radiological Environmental Monitoring Program and Radioactive Material Control Program (71122.03)

a. Inspection Scope

This area was inspected to ensure that the radiological environmental monitoring program verifies the impact of radioactive effluent releases to the environment and sufficiently validates the integrity of the radioactive gaseous and liquid effluent release program; and that the licensee's surveys and controls are adequate to prevent the inadvertent release of licensed materials into the public domain. The team used the requirements in 10 CFR Part 20, Appendix I of 10 CFR Part 50, the Offsite Dose Calculation Manual, and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed

- Annual environmental monitoring reports and licensee event reports;
- Selected air sampling and thermoluminescence dosimeter monitoring stations;
- Collection and preparation of environmental samples;
- Operability, calibration, and maintenance of meteorological instruments;
- Each event documented in the Annual Environmental Monitoring Report which involved a missed sample, inoperable sampler, lost thermoluminescence dosimeter, or anomalous measurement;
- Significant changes made by the licensee to the Offsite Dose Calculation Manual as the result of changes to the land census or sampler station modifications since the last inspection;
- Calibration and maintenance records for air samplers, composite water samplers, and environmental sample radiation measurement instrumentation, quality control program, interlaboratory comparison program results, and vendor audits;
- Locations where the licensee monitors potentially contaminated material leaving the radiological controlled area [or controlled access area] and the methods used for control, survey, and release from these areas;

- Type of radiation monitoring instrumentation used to monitor items released, survey and release criteria of potentially contaminated material, radiation detection sensitivities, procedural guidance, and material release records;

- Audits, self-assessments, corrective action documents and licensee event reports or special reports, if any were required, since the previous inspection;

Specific documents reviewed during this inspection are listed in the attachment.

Either because the conditions did not exist or an event had not occurred, no opportunities were available to review the following items:

- Licensee event reports or special reports

These activities constitute completion of ten of the required ten samples, as defined in Inspection Procedure 71122.03-05.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES

4OA2 Problem Identification and Resolution

Annual Sample Review

a. Inspection Scope

The team evaluated the effectiveness of the licensee's problem identification and resolution process with respect to the following inspection areas:

- Radiation Monitoring Instrumentation (Section 2OS3)

- Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems (Section 2PS1)

- Radioactive Material Processing and Transportation (Section 2PS2)

- Radiological Environmental Monitoring Program and Radioactive Material Control Program (Section 2PS3)

b. Findings and Observations

Introduction. The inspectors identified a non-cited violation of 10 CFR Part 50, Appendix B, Criterion XVI, "Corrective Action," for the failure to promptly correct a condition adverse to quality. The violation had very low safety significance.

Description. On February 10, 2009, during a review of corrective action documents, the inspectors noted that condition report CR-RBS-2007-03034 was closed without all corrective actions being taken. During the previous NRC radiation safety team inspection, performed in July of 2007, the team noted a portable Eberline RO-7 radiation monitor was being used as a secondary calibration instrument standard for the calibration of main steam line radiation monitors under surveillance test procedure STP-511-4201. In that procedure, the portable radiation monitor was listed as measuring and test equipment. EN-MA-105, "Control of Measuring and Test Equipment" specifies requirements for storage, handling/use, and calibration of measuring and test equipment. However, the Eberline RO-7 was not being controlled as measuring and test equipment in accordance with procedure EN-MA-105. This issue was determined by the team to be of minor significance, in that the inaccuracy of the main steam line monitors calibrated using the methodology in the surveillance test procedure was small.

The licensee entered the failure to treat the Eberline RO-7 radiation monitor as measuring and test equipment into the corrective action program. Two corrective action items were assigned. Corrective Action-1 required an evaluation of the issue and appropriate action. The evaluation concurred the Eberline RO-7 was considered measurement and test equipment, since it was used to quantify the performance of the main steam line radiation monitor. Corrective Action-1 also listed an action plan which identified Corrective Action-2. Corrective Action-2 stated, "Contact appropriate radiation protection department personnel to determine a process for specific Eberline RO-7 instrument(s) to be identified as measuring and test equipment and be formally controlled under the guidelines of EN-MA-105. The response to Corrective Action-2 stated, in part, "The RO-7 is checked against the calibrated radiation source at time of use to verify calibration and is essentially functioning as a transfer medium for the main steam line monitor. Traceability is controlled through the radiation protection process, therefore, the Eberline RO-7 is not measuring and test equipment controlled under EN-MA-105. No further actions are needed, this condition report is closed." During a review of the closure actions, the team determined the individual closing the condition report had not verified his/her assumption about the traceability of the instrument calibration. In fact, the Eberline RO-7 was not checked against a calibrated radiation source at the time of use, since the Shepard Calibrator used to "source check" the Eberline RO-7 had not been calibrated since 2006. Therefore, the licensee's basis for closing condition report CR-RBS-2007-03034 without implementing further corrective action was not valid and not based on a conservative assumption. The team also determined that the Eberline RO-7 was still listed in the surveillance procedure as measuring and test equipment at the time of the current inspection. The licensee documented the failure to promptly correct a condition adverse to quality in the corrective action program as condition report CR-RBS-2009-00747.

Analysis. The failure to promptly correct a condition adverse to quality is a performance deficiency. The finding is greater than minor because if left uncorrected, the performance deficiency would have the potential to lead to a more significant safety concern. Additionally, the guidance provided in the NRC Enforcement Manual, Chapter

2.10.f, states, “Where a licensee does not take corrective action for a minor violation…the matter should be considered more than minor.” The performance deficiency affected the barrier integrity cornerstone in that the proper calibration of the main steam line monitors is necessary to ensure proper isolation of containment in the event of fuel damage. Using Phase 1 worksheet from Manual Chapter 0609, “Significance Determination Process,” this finding was determined to have very low safety significance because it did not represent a degradation of the radiological barrier function provided for the control room, auxiliary building, spent fuel pool, or standby gas treatment system; did not represent an actual open pathway in the physical integrity of the reactor containment and heat removal components, and did not involve an actual reduction in function of hydrogen ignitors in the reactor containment. This finding has a crosscutting aspect in the area of human performance associated with the decision making component because the licensee did not use conservative assumptions to demonstrate that the decision to close the condition report with no further action was appropriate (H1.b).

Enforcement. Title 10 of the Code of Federal Regulations, Part 50, Appendix B, Criterion XVI, “Corrective Action,” requires that measures be taken to ensure that conditions adverse to quality are promptly identified and corrected. Contrary to the above, the licensee failed to promptly correct a condition adverse to quality. Specifically, from July 2007 to February 2009, the licensee did not take corrective actions to control the Eberline RO-7 as measuring and test equipment as required by procedure EN-MA-105, “Control of Measuring and test equipment.” Because this finding is of very low safety significance and was entered into the licensee’s corrective action program as CR-RBS-2009-00747, this violation is being treated as a non-cited violation in accordance with Section VI.A of the Enforcement Policy: NCV 05000458/2009006-01, “Failure to promptly correct a condition adverse to quality.”

4OA5 Other Activities

.1 (Closed) Temporary Instruction 2515/173, “Review of the Implementation of the Industry Groundwater Protection Voluntary Initiative”

a. Inspection Scope

An NRC assessment was performed of the licensee’s groundwater protection program to determine whether the licensee implemented the voluntary Industry Groundwater Protection Initiative, dated August 2007 (Nuclear Energy Institute 07-07, ADAMS Accession Number ML072610036). Inspectors interviewed personnel, performed walk-downs of selected areas, and reviewed the following items:

- Records of the site characterization of geology and hydrology;
- Evaluations of systems, structures, and or components that contain or could contain licensed material and evaluations of work practices that involve licensed

material for which there is a credible mechanism for the licensed material to reach the groundwater;

- Implementation of an onsite groundwater monitoring program to monitor for potential licensed radioactive leakage into groundwater;

- Procedures for the decision making process for potential remediation of leaks and spills,
including consideration of the long term decommissioning impacts;

- Records of leaks and spills recorded, if any, in the licensee's decommissioning files in accordance with 10 CFR 50.75(g);

- Licensee briefings of local and state officials on the licensee's groundwater protection initiative;

- Protocols for notification to the local and state officials, and to the NRC regarding detection of leaks and spills;

- Protocols and/or procedures for thirty day reports if an onsite groundwater sample exceeds the criteria in the radiological environmental monitoring program;

- Groundwater monitoring results as reported in the annual effluent and/or environmental monitoring report;

- Licensee and industry assessments of implementation of the groundwater protection initiative.

b. Findings

No findings of significance were identified. Implementation of the Industry Groundwater Protection Initiative is voluntary. Under the final initiative, each site was to have developed an effective, technically sound groundwater protection program by August 2008. The licensee's corporate office completed the Entergy fleet self-assessment of the initiative on August 25, 2008. According to the Entergy self-assessment, the licensee had not met the objectives established in Nuclear Energy Institute 07-07 Ground Water Protection Initiative for the River Bend site. The team also found that the licensee had not fully implemented the Ground Water Protection Initiative. According to the Entergy fleet self-assessment, River Bend had not implemented the following aspects of the Ground Water Protection Initiative:

- An overall groundwater monitoring program and plan had not been implemented

- The site-specific hydrogeological evaluation that was conducted in January 2007 needed to be upgraded to reflect groundwater flow characteristics.

- The risk assessment and leak detection program for systems, structures, and components needed to be updated regarding potential onsite contamination and leaks to groundwater (i.e. monitoring and sentinel wells).

- A remediation protocol to prevent migration of licensed material off-site and to minimize decommissioning aspects had not been established.

  As of a result of this team inspection, River Bend issued Condition Report RBS-2009-00780 to perform a line-by-line review of the NEI Ground Water Protection Initiative and Entergy fleet self-assessment.

4OA6 Management Meetings

Exit Meeting Summary

On February 13, 2009, the team presented the inspection results to Mr. M. Perito, Vice President, Operations, and other members of his staff who acknowledged the findings. The team confirmed that proprietary information was not provided or examined during the inspection. On March 30, 2009, the team discussed the finding documented in Section 4OA2 of this report with Mr. E. Olson, General Manager, Plant Operations, and other members of his staff.

ATTACHMENT: SUPPLEMENTAL INFORMATION

SUPPLEMENTAL INFORMATION

KEY POINTS OF CONTACT

Licensee personnel

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J. Brown; Technician, Radiation Protection
A. Carter; Technician, Radiation Protection
P. Ellis, Technician, Radiation Protection
R. Heath, Supervisor, Chemistry
B. Houston, Manager, Radiation Protection
K. Huffstatler, Senior Licensing Specialist, Licensing
V. Huffstatler, Technical Specialist, Chemistry
B. Michura, Master Nuclear Environmental Technician, Chemistry
B. Noel; System Engineer, Radiation Monitoring System
E. Olson, General Manager, Plant Operations
M. Perito, Vice President, Operations
K. Rockwood, ALARA Supervisor, Radiation Protection
W. Spell, Senior Environmental Specialist, Chemistry

NRC

G. Larkin, Senior Resident Inspector
C. Norton, Resident Inspector

LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

Opened and Closed During this Inspection

05000458/2009006-01	NCV	Failure to promptly correct a condition adverse to quality

Previous Items Closed

NONE

Previous Items Discussed

NONE

LIST OF DOCUMENTS REVIEWED

Section 2OS3: Radiation Monitoring Instrumentation and Protective Equipment

Procedures

NUMBER	TITLE	REVISION
EN-RP-301	Radiation Protection Instrument Control	Revision 2
EN-RP-302	Operation of Radiation Protection Instrumentation	Revision 1
EN-RP-303	Source Checking of Radiation Protection Instrumentation	Revision 2
EN-RP-306	Calibration and Operation of the Eberline PM-7	Revision 2
EN-RP-307	Operation and Calibration of the Eberline Personnel Contamination Monitors	Revision 1
EN-RP-308	Operation and Calibration of Gamma Scintillation Tool Monitors	Revision 2
EN-MA-105	Control of Measuring and Test Equipment	Revision 3
EN-RP-501	Respiratory Protection Program	Revision 3
EN-RP-502	Inspection and Maintenance of Respiratory Protection Equipment	Revision 4
EN-RP-504	Breathing Air	Revision 2
RPP-0074	Refilling SCBA Cylinders	Revision 11

CORRECTIVE ACTION DOCUMENT NAME

RBS-2007-3025	RBS-2007-3034	RBS-2007-3563	RBS-2007-3206	RBS-2007-3398
RBS-2007-3411	RBS-2007-3974	RBS-2007-4553	RBS-2007-4746	RBS-2007-4866
RBS-2007-5232	RBS-2007-5567	RBS-2007-5695	RBS-2008-0282	RBS-2008-1932
RBS-2008-2077	RBS-2008-2467	RBS-2008-3180	RBS-2008-4347	RBS-2008-5376
RBS-2008-5766	RBS-2008-6510	HQN-2007-0445		

Area Monitor Calibrations

Channel No.	Monitor Description	Surveillance Procedure	Calibration Dates
RE16A	Primary Containment Area Radiation Monitor	STP-511-4249	5/09/2008
RE16B	Primary Containment Area Radiation Monitor	STP-511-4250	3/25/2008

Radiation Protection Instrumentation Calibrations

Identification No.	Instrument Type	Calibration Date
Accuscan II (96-5818)	Whole Body Counter	3/27/2008
Accuscan II (96-9762)	Whole Body Counter	3/26/2008
A006 #000201	Tool Monitor	8/19/2008
89-0560	Tool Monitor	8/26/2008
PCM-1B #356	Personnel Contamination Monitor	11/20/2007
PCM-2 #603	Personnel Contamination Monitor	1/20/2009
419	PM-7	8/19/2008
395	PM-7	8/28/2008

Miscellaneous Documents

D17-K610	I&C Setpoint Data Sheet (MSL Monitor High-High Alarm)	11/05/2004
RMS-ES22	I&C Setpoint Data Sheet (MSL Monitor High Alarm)	11/05/2004

Air Quality Test Reports

Location:	Date of Test
SCBA Compressor	11/25/2008
Plant Air System	11/25/2008
SCBA Compressor	9/02/2008
Plant Air System	9/02/2008

Section 2PS1: Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems

Procedures

NUMBER	TITLE	REVISION
COP-0046	Samplling Gaseous Effluents via the Wide Range Gas Monitors	Revision 11
RHP-0032	Dose Rate Calculation from Gaseous Effluents	Revision 9
RSP-0008	Offsite Dose Calculation Manual	Revision 13

CORRECTIVE ACTION DOCUMENTS

RBS-2007-04646 RBS-2007-05105 RBS-2008-04180 RBS-2008-03565

Audits and Self Assessments

QA-6-2007-RBS, Quality Assurance Audit of the Effluent and Environmental Monitoring Programs

Effluent Monitor Calibrations

Channel No.	Monitor Description	Surveillance Procedure	Calibration Dates
RE125	Plant vent exhaust noble gas	STP-511-4214	12/11/2008
RE3A	Main Steam Line Monitor	STP-511-4214	7/10/2007
RE125	Plant vent stack flow rate	STP-511-4231	1/04/2008
RE107	Liquid Radwaste Effluent Monitor	STP-511-4280	7/11/2007

Miscellaneous Documents

2006 and 2007 Annual Radiological Effluent Release Reports

Section 2PS3: Radiological Environmental Monitoring Program and Radioactive Material Control Program

Audits and Self-assessments
Quality Assurance Audit: Effluent and Environmental Monitoring Programs, October 2007

CORRECTIVE ACTION DOCUMENTS (CR-RBS)

2007-02395	2007-03724	2007-03776	2007-04128	2007-04645
2007-04679	2007-04920	2008-00797	2008-1623	2008-04761
2008-04679	2007-05207	2007-05217	2008-05351	2008-05207
2008-05565	2008-06385	2009-0110	2009-0202	2009-0441

Procedures

NUMBER	TITLE	REVISION
CSP-0006	Chemistry Surveillance and Scheduling System	Revision 24
ESP-8-022	Monitoring of Sanitary and Storm Sewers for Radioactivity	Revision 9
RSP-0008	Offsite Dose Calculation Manual	Revision 13
EN-CY-102	Laboratory Analytical Quality Control	Revision 1
EN-EV-116	Radiological Environmental Analytical Services	Revision 2
EN-RP-108	Radiation Protection Posting	Revision 7
EN-RP-121	Radioactive Material Control	Revision 4
EN-RP-143	Source Control	Revision 1

Miscellaneous Documents

2007 Radioactive Effluent Release Report

2007 and 2008 River Bend Station Interlaboratory Comparison Results

Calibration Records for Environmental Air Samplers 2008

2008 Calibration Records for Environmental Gamma Spectroscopy Detectors 1, 2, 4, and 5

Environmental Counting Instrument Quality Control Records 2007 - 2008

AREVA NP Environmental Laboratory
Analytical Service Semi-Annual Quality Assurance Status Report
(January – June 2007)

AREVA NP Environmental Laboratory
Analytical Service Semi-Annual Quality Assurance Status Report
(July – December 2007)

Meteorological System Calibrations and System Data 2007 - 2008

Section 4OA5 Temporary Instruction (TI) 2515/173

Procedures

NUMBER	TITLE	REVISION
EN-CY-108	Monitoring of Nonradioactive Systems	Revision 0
EN-CY-109	Sampling and Analysis of Groundwater Monitoring Wells	Revision 9
EN-CY-113	Response to Contaminated Spill/Leaks	Revision 3
EN-DC-343	Buried Piping and Tanks inspection and Monitoring Program	Revision 1

Miscellaneous Documents

NEI 07-07 Industry Groundwater Protection Initiative – Final Guidance Document

Site Hydrologic Assessment in Support of Entergy GPI: River Bend Station
January 2007