



# TURKEY POINT NRC EXAM – 03/12/09

Q #76

Unit 3 experienced a LOCA.

While performing 3-EOP-E-1, "Loss of Reactor or Secondary Coolant", the SRO directs the RO to check the status of the Safety Injection Accumulators.

RCS hot leg temperatures: 335°F

Accumulator pressures: 150 psig

Which ONE of the following identifies actions (if any) directed by 3-EOP-E-1 and the parameter used to determine the status of accumulators while in 3-EOP-E-1?

- A. Conditions are NOT met to isolate accumulators.  
RCS hot leg temperatures indicate the accumulators have NOT completed injecting water into the core.
- B. Conditions are NOT met to isolate accumulators.  
Accumulator pressures indicate the accumulators have NOT completed injecting water into the core.
- C. Conditions are met to isolate accumulators.  
Hot leg temperatures indicate the accumulators have completed injecting water into the core.
- D. Conditions are met to isolate accumulators.  
Accumulator pressures indicate the accumulators have completed injecting water into the core.

# TURKEY POINT NRC EXAM – 03/12/09

Q #77

Operators are performing 3-EOP-ES-0.2, "Natural Circulation Cooldown."

- When directed by 3-EOP-ES-0.2, the SRO directs the RO to place RHR in service using 3-OP-050, "Residual Heat Removal System."
- The RO is unable to start either RHR pump.

Which ONE of the following describes the correct implementation of 3-ONOP-050, "Loss of RHR"?

3-ONOP-050:

- A. shall NOT be performed in parallel with 3-EOP-ES-0.2.  
The SRO shall stop performance of 3-EOP-ES-0.2 until 3-ONOP-050 has been completed.
- B. shall NOT be performed in parallel with 3-EOP-ES-0.2.  
The SRO shall NOT perform 3-ONOP-050 until 3-EOP-ES-0.2 has been completed.
- C. may be performed in parallel with 3-EOP-ES-0.2.  
The SRO shall stop the plant cooldown and continue to perform other steps of 3-EOP-ES-0.2 while attempting to establish RHR cooling.
- D. may be performed in parallel with 3-EOP-ES-0.2.  
The SRO shall continue the plant cooldown while attempting to establish RHR cooling.

# TURKEY POINT NRC EXAM – 03/12/09

Q #78

Unit 3 is at 100% power when the following events occur:

- Annunciator H8/6, CCW HEAD TANK HI/LO LEVEL alarms.
- Annunciator A4/6, VCT HI/LO LEVEL alarms.
- CCW Head Tank level decreases to less than zero.
- When the Component Cooling Water Surge Tank Makeup valve, MOV-3-832, is fully opened, CCW Surge Tank level stabilizes at 30%.

In accordance with 3-ONOP-030, "Component Cooling Water Malfunction", which ONE of the following identifies the leak location and the basis for the maximum allowed time to restore the CCW Head Tank level?

A leak is occurring in the:

- A. Seal Water Heat Exchanger.  
Restore CCW system level to be within the CCW Head Tank within 24 hours to ensure boiling will not occur in the CCW system during a LOCA.
- B. Seal Water Heat Exchanger.  
Restore CCW system level to be within the CCW Head Tank within 24 hours to satisfy CCW Tech Specs operability requirements.
- C. Non-Regen Heat Exchanger.  
Restore CCW system level to be within the CCW Head Tank within 24 hours to satisfy CCW Tech Specs operability requirements.
- D. Non-Regen Heat Exchanger.  
Restore CCW system level to be within the CCW Head Tank within 24 hours to ensure boiling will not occur in the CCW system during a LOCA .

# TURKEY POINT NRC EXAM – 03/12/09

Q #79

The Turkey Point switchyard de-energized.

- 4A EDG is running and its output breaker shows red light indication.
- 4B EDG is running and its output breaker shows green light indication.
- 4B RCP has no light indication.
- The station blackout tie permissive blue light is off.

Which ONE of the following identifies the required initial operator action in accordance with 4-ONOP-004.3, "Loss of 4B 4KV Bus", and the correct Emergency Classification?

Direct an operator to:

- A. close the 4B EDG output breaker.  
Declare an Alert.
- B. close the 4B EDG output breaker.  
Declare an Unusual Event.
- C. open the 4B RCP breaker.  
Declare an Alert.
- D. open the 4B RCP breaker.  
Declare an Unusual Event.

# TURKEY POINT NRC EXAM – 03/12/09

Q #80

A hurricane has caused extensive damage to Turkey Point systems including the loss of the Switchyard and all Instrument Air.

- Neither system will be available for several days.
- All Unit 3 and Unit 4 Emergency Diesel generators are running.

Which ONE of the following identifies a problem that will result from the extended loss of offsite power and instrument air if no action is taken and identifies the procedure that will be used to preclude that problem?

- A. Unit 3 will experience a Loss of All AC power  
Have I&C install a Nitrogen supply to Unit 3 EDG Day Tank fill valves in accordance with 3-ONOP-004, "Loss of Off-site power."
- B. Unit 3 will experience a Loss of All AC power  
Have I&C install a Nitrogen supply to Unit 3 EDG Day Tank fill valves in accordance with 0-ONOP-013, "Loss of Instrument Air."
- C. Unit 4 will experience a Loss of All AC power.  
Have I&C install a Nitrogen supply to Unit 4 EDG Day Tank fill valves in accordance with 4-ONOP-004, "Loss of Off-site power."
- D. Unit 4 will experience a Loss of All AC power  
Have I&C install a Nitrogen supply to Unit 4 EDG Day Tank fill valves in accordance with 0-ONOP-013, "Loss of Instrument Air."

# TURKEY POINT NRC EXAM – 03/12/09

Q #81

Operators are performing 3-EOP-ECA-1.2, "LOCA Outside Containment".

- After closing all valves required by 3-EOP-ECA-1.2, RCS pressure is still decreasing.

Which ONE of the following identifies the procedure and mitigating strategy the SRO will use next?

Transition from 3-EOP-ECA-1.2 to:

- A. 3-EOP-ECA-1.1, "Loss of Emergency Coolant Recirculation".  
Minimize HHSI flow and maintain RCS cooldown rate at less than 100°F/hr.
- B. 3-EOP-E-1, "Loss of Reactor or Secondary Coolant".  
Minimize HHSI flow and maintain RCS cooldown rate at less than 100°F/hr.
- C. 3-EOP-ECA-1.1, "Loss of Emergency Coolant Recirculation".  
Maintain two HHSI pumps running and cool down the RCS at the maximum achievable rate.
- D. 3-EOP-E-1, "Loss of Reactor or Secondary Coolant".  
Maintain two HHSI pumps running and cool down the RCS at the maximum achievable rate.

# TURKEY POINT NRC EXAM – 03/12/09

Q #82

Unit 3 power was 100% when Control Bank “D” rod M-8 dropped into the core.

- The following conditions exist one hour after retrieval of Rod M-8 was completed in accordance with 3-ONOP-028.3, “Dropped RCC”:
  - Reactor power is 49%.
  - “D” Bank step counters read 180 steps.
  - RPI indications for Bank “D” control rods are:

<u>Control Rod</u>	<u>RPI Indication</u>
M-8	167
H-8	166
D-8	195
H-4	160
H-12	199

Which ONE of the following is correct in accordance with Tech specs 3.1.3.1, “Movable Control Rod Assemblies – Group Height”?

- A. Only Control Rods H-4 and H-12 exceed the Allowed Rod Misalignment. Restore rod alignment within 1 hour or enter TS 3.0.3.
- B. Only Control Rods H-4 and H-12 exceed the Allowed Rod Misalignment. Restore rod alignment within 1 hour or be in Mode 3 within the following 6 hours.
- C. All Bank “D” control rods exceed the Allowed Rod Misalignment. Restore rod alignment within 1 hour or enter TS 3.0.3.
- D. All Bank “D” control rods exceed the Allowed Rod Misalignment. Restore rod alignment within 1 hour or be in Mode 3 within the following 6 hours.

# TURKEY POINT NRC EXAM – 03/12/09

Q #83

Unit 3 is at 50% power when the controlling Pressurizer level transmitter fails high.

Which ONE of the following identifies the initial effect of this failure on Charging flow rate and the Tech Spec limitations on continued operation?

Charging flow rate will:

- A. decrease until the failed channel is selected out.  
The failed channel must be placed in the tripped condition within 6 hours.  
Operation may continue until another channel's operational test is required.
- B. decrease until the failed channel is selected out.  
The failed channel must be placed in the tripped condition within 6 hours.  
Continued operation is not affected.
- C. increase until the failed channel is selected out.  
The failed channel must be placed in the tripped condition within 6 hours.  
Operation may continue until another channel's operational test is required.
- D. increase until the failed channel is selected out.  
The failed channel must be placed in the tripped condition within 6 hours.  
Continued operation is not affected.

# TURKEY POINT NRC EXAM – 03/12/09

Q #84

- At 0300, Operators evacuated the Control Room and are performing the actions of 0-ONOP-105, "Control Room Evacuation."
- At 0318, the Outside SNPO has NOT yet reported to the Unit Supervisor that control of shutdown systems has been established.

In accordance with 0-EPIP-20101, "Duties of Emergency Coordinator", which ONE of the following describes the implications (if any) of the failure of the Outside SNPO to report?

The Emergency Coordinator (1) to implement an Owner Controlled Area Evacuation and (2) the authority to downgrade the emergency classification.

- |    | (1)             | (2)           |
|----|-----------------|---------------|
| A. | is required     | retains       |
| B. | is required     | no longer has |
| C. | is NOT required | no longer has |
| D. | is NOT required | retains       |

# TURKEY POINT NRC EXAM – 03/12/09

Q #85

Operators are performing 4-EOP-ES-0.3, "Natural Circulation Cooldown with Steam Void in Vessel (With RVLMS)."

- All conditions required for starting the 4B RCP are satisfied.

The RO is preparing to start the 4B RCP.

- RVLMS head indication: 33%
- Pressurizer level: 59%
- CET subcooling: 80°F
- Pressurizer Pressure: 1935 psig
- Pressurizer Steam Space Temp: 632°F
- Pressurizer Water Space Temp: 620°F

In accordance with 4-EOP-ES-0.3, which ONE of the following describes the actions that are required to be taken?

- A. Increase charging flow to increase Pressurizer level and turn on Pressurizer Backup heaters to saturate the pressurizer water.
- B. Increase charging flow to increase Pressurizer level and dump steam to increase RCS subcooling.
- C. Decrease charging flow to decrease Pressurizer level and turn on Pressurizer Backup heaters to saturate the pressurizer water.
- D. Decrease charging flow to decrease Pressurizer level and dump steam to increase RCS subcooling.

# TURKEY POINT NRC EXAM – 03/12/09

Q #86

Unit 3 is at 100% power in a normal system alignment.

The following annunciators alarm:

- A-1/5, RCP SEAL LEAKOFF HIGH FLOW
- A-6/5, RCP LABYRINTH SEAL LO  $\Delta P$
- G 2/2 - RCP "B" STANDPIPE HI LEVEL
  
- 3B RCP #1 Seal leak-off flow is stable at 6.5 gpm as indicated on ERDADS/DCS.

In accordance with 3-ONOP-041.1, "Reactor Coolant Pump Off-Normal", which ONE of the following identifies the procedure operators will use and the required operator actions?

- A. Trip Unit 3 Reactor using 3-EOP-E-0, "Reactor Trip or Safety Injection."  
When the reactor trip breakers are open, stop the 3B RCP and maintain Seal Leakoff Isolation Valve CV-3-303B open.
- B. Trip Unit 3 Reactor using 3-EOP-E-0, "Reactor Trip or Safety Injection."  
When the reactor trip breakers are open, stop the 3B RCP and close Seal Leakoff Isolation Valve CV-3-303B.
- C. Shutdown using 3-ONOP-100, "Fast Load Reduction."  
When the reactor trip breakers are open, stop the 3B RCP and maintain Seal Leakoff Isolation Valve CV-3-303B open.
- D. Shutdown using 3-ONOP-100, "Fast Load Reduction."  
When the reactor trip breakers are open, stop the 3B RCP and close Seal Leakoff Isolation Valve CV-3-303B.

# TURKEY POINT NRC EXAM – 03/12/09

Q #87

Unit 3 is in Mode 5.

- RCS Fill and Vent has been completed.
- 3A RHR loop is in operation.
- 3B RHR loop is INOPERABLE.
- Steam Generator levels are:
  - 3A: 40% Narrow Range (NR)
  - 3B: 15% Narrow Range (NR)
  - 3C: 0% Wide Range (WR)
- RCS pressure decreases to 50 psig.

Which ONE of the following describes the current status of Tech spec 3.4.1.4.1, "RCS Cold Shutdown – Loops Filled" including the reason as stated in the Tech Spec Basis?

The "RCS Cold Shutdown – Loops Filled" Limiting Condition for Operation is:

- A. satisfied. RCS pressure is greater than the minimum required
- B. satisfied. Two SG levels must be greater than 10% NR.
- C. NOT satisfied. RCS pressure is less than the minimum required.
- D. NOT satisfied. All SG levels must be greater than 10% NR.

# TURKEY POINT NRC EXAM – 03/12/09

Q #88

Unit 4 is at 100% power when the following annunciators alarm:

A 7/1 PRT HI/LO LEVEL HI PRESS/TEMP

A 7/2 PZR PORV HI TEMP

- PRT pressure is above the alarm setpoint and slowly increasing.
- Pressurizer PORV temperature is above the alarm setpoint and slowly increasing.
- RCS leakage through Pressurizer PORV, PCV-4-456, is 5.0 gpm.

Which ONE of the following identifies the required operator action in accordance with 4-ONOP-041.5, "Pressurizer Pressure Control Malfunction" and the subsequent operability status of PCV-4-456?

- A. Close and remove power from PRZ PORV Block Valve, MOV-4-535. PCV-4-456 is INOPERABLE.
- B. Close and remove power from PRZ PORV Block Valve, MOV-4-535. PCV-4-456 is OPERABLE.
- C. Close and maintain power to PRZ PORV Block Valve, MOV-4-535. PCV-4-456 is INOPERABLE.
- D. Close and maintain power to PRZ PORV Block Valve, MOV-4-535. PCV-4-456 is OPERABLE.

# TURKEY POINT NRC EXAM – 03/12/09

Q #89

Unit 4 is at 100% power.

- Pressurizer Spray valve, PCV-4-455A, fails open.
- The RO takes manual control of the PCV-4-455A controller and drives the demand to zero.
- The RO requests confirmation from the SRO that PCV-4-455A is closed.

Which ONE of the following describes the PCV-4-455A icon on the ERDADS/DCS display screen and the directions given in accordance with 4-ONOP-041.5, "Pressurizer Pressure Control Malfunction"?

**IF** the interior of the valve icon on the ERDADS/DCS display screen is RED, **THEN**

- A. the spray valve is closed. The reactor is not required to be tripped. Maintain pressurizer pressure greater than 2000 psig.
- B. the spray valve is closed. The reactor is not required to be tripped. Check for leaking pressurizer PORVs and safety valves.
- C. the spray valve is open. The reactor is required to be tripped. 4C RCP must be stopped.
- D. the spray valve is open. The reactor is required to be tripped. 4A RCP must be stopped.

# TURKEY POINT NRC EXAM – 03/12/09

Q #90

Following a large break design basis LOCA, operators have established Piggy-Back cold leg recirculation in accordance with 3-EOP-ES-1.3, "Transfer to Cold Leg Recirculation."

- 3A Containment Spray Pump (CSP) is running.
- The running 3A Residual heat Removal (RHR) pump's motor amps and flow begin to oscillate.

Which ONE of the following identifies the effect on CSP operation and the required operator actions?

3A CSP will:

- A. lose its suction source.  
Transition to 3-EOP-ECA-1.1 "Loss of Emergency Coolant Recirculation."  
Stop 3A RHR Pump and 3A CSP.  
Establish core cooling from one Unit 4 HHSI Pump.
- B. lose its suction source.  
Transition to 3-EOP-ECA-1.1 "Loss of Emergency Coolant Recirculation."  
Stop 3A RHR Pump and 3A CSP.  
Establish core cooling from two Unit 4 HHSI Pumps.
- C. NOT lose its suction source.  
Remain in 3-EOP-ES-1.3.  
Stop 3A RHR Pump, maintain 3B RHR Pump off and maintain 3A CSP running.
- D. NOT lose its suction source.  
Remain in 3-EOP-ES-1.3.  
Start 3B RHR pump, stop 3A RHR pump and maintain 3A CSP running.

# TURKEY POINT NRC EXAM – 03/12/09

Q #91

Operators are reloading the core on Unit 4.

- The manipulator crane operator is raising a fuel element using the manipulator hoist.
- As the load indicator increases to 2500 pounds, the overload light on the manipulator console comes on and the hoist motion stops.
- The element is half way out of the containment upender basket.
- The manipulator operator is unable to raise or lower the hoist.
- Subsequent investigation reveals no binding is occurring in the manipulator hoist.

In accordance with 0-OP-038.10, "Manipulator Crane - Operating Instructions", which ONE of the following correctly describes the response of the overload interlock and if the Shift Manager may authorize bypassing the interlock for this situation?

The manipulator hoist overload interlock actuated:

- A. at the correct load.  
The Shift Manager can NOT grant permission to bypass the overload interlock.
- B. at the correct load.  
The Shift Manager can grant permission to bypass the overload interlock.
- C. too early.  
The Shift Manager can NOT grant permission to bypass the overload interlock.
- D. too early.  
The Shift Manager can grant permission to bypass the overload interlock.

# TURKEY POINT NRC EXAM – 03/12/09

Q #92

Unit 3 is in Mode 6.

Annunciator X4/1, ARMS HI RADIATION, unexpectedly alarms.

- Unit 3 Containment Channel 3 (In-Core Drive) is in alarm.
- A flux map detector has been withdrawn past the 10 path transfer device.
- As a result of this event, an I&C technician has received a wholebody dose (TEDE) of 26 rems.

Which ONE of the following identifies the reportability requirements of 0-ADM-115, "Notification of Plant Events"?

- A. The NRCOC shall be notified immediately.
- B. The NRCOC shall be notified anytime within 8 hours.
- C. Only the NRC resident shall be notified immediately.
- D. Only the NRC resident shall be notified anytime within 8 hours.

# TURKEY POINT NRC EXAM – 03/12/09

Q #93

Unit 4 is at 100% power when the RO reports:

- Service Air pressure is abnormally low.
- Containment pressure has increased to 3.1 psig.

A system walk down reveals Service Air to Containment Isolation valve, 4-40-204, (OC) is open.

Which ONE of the following correctly describes the required Tech Spec action and the basis for this action?

- A. Restore Containment integrity within 6 hours to prevent an unmonitored release.
- B. Restore Containment integrity within 6 hours to prevent off-site doses from exceeding 10CFR100 limits during a LOCA.
- C. Restore Containment pressure to within limits within 1 hour to prevent Containment pressure from exceeding design pressure during a LOCA.
- D. Restore Containment pressure to within limits within 1 hour to prevent early SI actuation in the event of RCS leakage.

# TURKEY POINT NRC EXAM – 03/12/09

Q #94

Operators have stabilized Unit 3 after a reactor trip from 100% power.

In accordance with 0-ADM-115, "Notification of Plant Events", which ONE of the following describes the Shift Manager's responsibilities related to this event?

An Event Notification Worksheet has to be completed and:

- A. The NRC has to be notified within 4 hours.  
A Condition Report has to be generated.
- B. The NRC has to be notified within 4 hours.  
A Condition Report does NOT have to be generated.
- C. The NRC has to be notified within 8 hours.  
A Condition Report has to be generated.
- D. The NRC has to be notified within 8 hours.  
A Condition Report does NOT have to be generated.

# TURKEY POINT NRC EXAM – 03/12/09

Q #95

A reactor startup is in progress on Unit 4.

- After the **second doubling**, the 1/M plot predicts reactor criticality at rod height C-170 which is 420 pcm different from the ECC rod height.

Which ONE of the following describes the required operator actions in accordance with O-OSP-040.4, "Estimated Critical Conditions"?

- A. Allow the startup to continue. If the difference is the same after the third doubling, it will take permission from the Operations Manager to continue.
- B. Allow the startup to continue. If the difference is the same after the third doubling, it will take permission from the Reactor Engineering Supervisor to continue.
- C. Stop the startup. It will take permission from the Operations Manager to continue.
- D. Stop the startup. It will take permission from the Reactor Engineering Supervisor to continue.

# TURKEY POINT NRC EXAM – 03/12/09

Q #96

Unit 3 is in Mode 3 and Unit 4 is in Mode 4.

- Motor Control Center (MCC) 4K de-energizes.

Which ONE of the following describes Tech Spec requirements associated with the loss of 4K MCC?

- A. Test the operability of the remaining EDGs within 12 hours. If any of the remaining EDGs is discovered to be inoperable, restore at least one of the inoperable EDGs to operability within 2 hours.
- B. Test the operability of the remaining EDGs within 24 hours. If any of the remaining EDGs is discovered to be inoperable, restore at least one of the inoperable EDGs to operability within 2 hours.
- C. SU XFR Operability Verification must be performed within 1 hour. If either SU XFR alignment is discovered to be inoperable, the NRC shall be notified within 4 hours
- D. SU XFR Operability Verification must be performed within 1 hour. If either SU XFR alignment is discovered to be inoperable, the NRC shall be notified within 8 hours

## TURKEY POINT NRC EXAM – 03/12/09

Q #97

An Emergency Response Team is being briefed on a mission to rescue a worker who has suffered a broken leg.

- The worker is in a location where the general area dose rates are 100 mrem/hr.
- The team will pass through high dose rate areas to reach the worker.
- Health Physics estimates each team member will receive a total dose (TEDE) of 9 rem and a thyroid dose of 6 rem while performing this rescue.

Which ONE of the following correctly describes the estimated dose as compared to the TEDE limit of 0-EPIP-20111, "Re-entry" and the guidance for issuing Potassium Iodide (KI) to the team members?

The estimated total dose

- A. is within the TEDE limit.  
KI should be issued to prevent iodine absorption
- B. is within the TEDE limit.  
KI should NOT be issued.
- C. exceeds the TEDE limit.  
KI should NOT be issued
- D. exceeds the TEDE limit.  
KI should be issued to prevent iodine absorption.

# TURKEY POINT NRC EXAM – 03/12/09

Q #98

A radiation source caused by a leaking valve stem has forced a local evacuation of the 4' elevation of the Auxiliary building.

- Health Physics personnel have surveyed the area and determined the general area dose rates are 900 mrem/hr.
- Mechanical Maintenance requests permission from the Shift Manager to immediately send a mechanic into the area to repair the leak.

In accordance with 0-ADM-600, "Radiation Protection Manual", which ONE of the following describes limitations related to the Shift Manager authorizing the mechanic to enter the area?

The Shift Manager can

- A. NOT authorize the mechanic to enter the area under these conditions.
- B. only authorize the mechanic to enter the area after a Specific RWP is issued.
- C. authorize the mechanic to enter the area without an escort under a General Entry RWP.
- D. authorize the mechanic to enter the area under a General Entry RWP only if escorted by a person qualified in radiation protection procedures.

# TURKEY POINT NRC EXAM – 03/12/09

Q #99

Unit 4 has experienced a tube leak in the 4C steam generator.

- Leakage is greater than charging pump capacity and an Alert has been declared.

Which ONE of the following (if any), identifies some of the **minimum** action(s) directed by the Emergency Coordinator in accordance with 0-EPIP-20101, "Duties of Emergency Coordinator and 0-EPIP-20104, Emergency Response Organization Notifications / Staff Augmentation?"

The Emergency Coordinator (1) activate on Site Emergency Response Facilities and (2) implement an Owner Controlled Area Evacuation.

- |    | <u>(1)</u> | <u>(2)</u> |
|----|------------|------------|
| A. | will not   | will not   |
| B. | will       | will not   |
| C. | will not   | will       |
| D. | will       | will       |

# TURKEY POINT NRC EXAM – 03/12/09

Q #100

Operators are performing 4-EOP-ES-0.2, "Natural Circulation Cooldown."

- With RCS cold leg temperatures equal to 270°F, the SRO dispatches a field operator to locally obtain Steam Generator secondary temperature measurements.
- The field operator reports the following secondary temperature measurements:
  - 4A SG: 325°F
  - 4B SG: 315°F
  - 4C SG: 315°F

NOTE:

Tech Spec 3.4.1.3: "Reactor Coolant System – Hot Shutdown"

Tech Spec 3.1.1.1: "Boration Control Shutdown Margin –  
Tavg Greater Than 200°F"

Which ONE of the following identifies the requirements of 4-EOP-ES-0.2 regarding the starting of an RCP under these conditions and the effect of starting an RCP on Tech Specs and its basis?

- A. Operators may start the 4B or 4C RCP but shall NOT start the 4A RCP. Starting 4A RCP would be a violation of Tech Spec 3.4.1.3 and could result in an unacceptable RCS pressure transient
- B. Operators may start the 4B or 4C RCP but shall NOT start the 4A RCP. Starting 4A RCP would be a violation of Tech Spec 3.1.1.1 and could result in an unacceptable positive reactivity insertion.
- C. Operators shall NOT start any RCPs. Starting an RCP would be a violation of Tech Spec 3.4.1.3 and could result in an unacceptable RCS pressure transient
- D. Operators shall NOT start any RCPs. Starting an RCP would be a violation of Tech Spec 3.1.1.1 and could result in an unacceptable positive reactivity insertion.

**Turkey Point 2009-301 Initial Written Examination Reference List  
For RO and SRO Written Examinations**

**Steam Tables**

**SRO PORTION OF EXAM**

76	C
77	D
78	A
79	D
80	B
81	A
82	B
83	B
84	B
85	A
86	B
87	C
88	D
89	C
90	A
91	D
92	A
93	C
94	A
95	B
96	C
97	A
98	D
99	B
100	C