



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 20, 2009

Mr. Larry Meyer
Site Vice President
FPL Energy Point Beach, LLC
6610 Nuclear Road
Two Rivers, WI 54241

SUBJECT: POINT BEACH NUCLEAR PLANT, UNIT 2, REVIEW OF THE 2008 (U2R29)
STEAM GENERATOR TUBE INSPECTION REPORT (TAC NO. MD9689)

Dear Mr. Meyer:

By letter dated September 16, 2008 (Agencywide Documents and Access Management System (ADAMS) Accession Number ML082590073), as supplemented by letter dated March 3, 2009 (ADAMS Accession Number ML090641097), FPL Energy Point Beach, LLC (the licensee), submitted information pertaining to the 2008 steam generator tube inspections performed at Point Beach Nuclear Plant, Unit 2, during the 29th refueling outage.

The Nuclear Regulatory Commission staff has completed its review of these reports and concludes that the licensee provided the information required by Point Beach Nuclear Plant Unit 2 Technical Specifications and that no additional follow-up is required at this time. The NRC staff's review of the report is enclosed.

Sincerely,

A handwritten signature in black ink, appearing to read "J. Poole", written over a white background.

Justin C. Poole, Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-301

Enclosure:
As stated

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POINT BEACH NUCLEAR PLANT UNIT 2
REVIEW OF 2008 STEAM GENERATOR TUBE INSPECTION REPORTS
FOR THE SPRING 2008 REFUELING OUTAGE
DOCKET NUMBER 50-301

By letter dated September 16, 2008 (Agencywide Documents and Access Management System (ADAMS) Accession Number ML082590073), as supplemented by letter dated March 3, 2009 (ADAMS Accession Number ML090641097), FPL Energy Point Beach, LLC (the licensee), submitted information pertaining to the 2008 steam generator (SG) tube inspections performed at Point Beach Nuclear Plant (PBNP), Unit 2, during the 29th refueling outage (U2R29).

The two SGs at PBNP Unit 2 were replaced in 1996 with SGs designed and fabricated by Westinghouse. Each SG contains 3,499 thermally treated Alloy 690 tubes. Each tube has a nominal outside diameter of 0.875 inches and a nominal wall thickness of 0.050 inches. The tubes were hydraulically expanded at both ends for the full depth of the tubesheet and are supported by seven stainless steel tube support plates and one stainless steel flow distribution baffle. The tubes installed in rows 1 through 14 were thermally stress relieved after bending.

The licensee provided the scope, extent, methods, and results of the PBNP Unit 2 SG tube inspections in the document referenced above. In addition, the licensee described corrective actions (e.g., tube plugging) taken in response to the inspection findings.

Based on its review of the reports submitted, the Nuclear Regulatory Commission (NRC) staff has the following observations and comments:

- As of the U2R29 outage, the SGs had operated for approximately 98 effective full power months (EFPM) in the 144 EFPM sequential period.
- Wear indications at the anti-vibration bars and the tube supports were reported for the first time during the 2008 inspection. Although the indications were first reported in 2008, a review of prior inspection data for these locations indicated that some of the indications were present in prior inspections but not detected/reported because of their shallow depth. The depths of the indications ranged from 4- to 11-percent through-wall (i.e., very shallow).
- The upper internals and the top tube support plate of SG A were visually inspected with no signs of degradation. In addition, the top tube support plate in SG B was visually inspected. There was no observed blockage of the trifoil shaped holes in the top tube support plate in either SGs A or B.

Enclosure

- One tube had a possible loose part indication that could not be visually inspected. The tube was left in service since there was no degradation at this location.

Based on a review of the information provided, the NRC staff concludes that the licensee provided the information required by their technical specifications. In addition, the staff concludes that there are no technical issues that warrant follow-up action at this time since the inspections appear to be consistent with the objective of detecting potential tube degradation and the inspection results appear to be consistent with industry operating experience at similarly designed and operated units.

Principal Contributor: K. Karwoski, NRR

Date: April 20, 2009

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Sincerely,

/RA/

Justin C. Poole, Project Manager
Plant Licensing Branch III-1
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Amendment Accession Number: ML090970225

*per memo dated April 2, 2009

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