



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

April 7, 2009

Mr. Dale E. Young, Vice President  
Crystal River Nuclear Plant (NA1B)  
ATTN: Supervisor, Licensing & Regulatory Programs  
15760 W. Power Line Street  
Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER NUCLEAR PLANT, UNIT NO. 3 – WITHDRAWAL OF  
LICENSE AMENDMENT REQUEST #310, REVISION 0, APPLICATION TO  
ADOPT TECHNICAL SPECIFICATIONS TASK FORCE TRAVELER 490,  
REVISION 0, “DELETION OF E BAR DEFINITION AND REVISION TO RCS  
SPECIFIC ACTIVITY TECH SPEC” (TAC NO. ME0731)

Dear Mr. Young:

By letter dated February 26, 2009 (Agencywide Documents Access and Management System Accession No. ML090620582), Florida Power Corporation (the licensee) submitted a license amendment request (LAR) for Crystal River, Unit 3 (CR-3). The proposed amendment would replace the current CR-3 Improved Technical Specification (ITS) 3.4.15 limit on reactor coolant system (RCS) gross specific activity with a new limit on RCS noble gas specific activity. The noble gas specific activity limit would be based on a new Dose Equivalent Xe-133 definition that would replace the current E—Average Disintegration Energy (E Bar) definition. In addition, the amendment would revise the current Dose Equivalent I-131 definition to allow the use of additional Committed Effective Dose Equivalent dose conversion factors. The licensee stated that the proposed changes would be consistent with the Nuclear regulatory Commission (NRC)-approved Technical Specification Task Force (TSTF) change traveler, TSTF-490, Revision 0, “Deletion of E Bar Definition and Revision to RCS Specific Activity Tech Spec [Technical Specification],” that was announced for availability in the *Federal Register* on March 19, 2007 (72 FR 12838) as part of the consolidated line item improvement process (CLIP).

Acceptance review, per Office of Nuclear Reactor Regulation, Office Instruction LIC-109, “Acceptance Review Procedures,” was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical review. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Consistent with Section 50.90 of Title 10 of the *Code of Federal Regulations* (10 CFR), an amendment to the license (including the Technical Specifications) must fully describe the changes requested, and following as far as applicable, the form prescribed for original applications. Section 50.34 of 10 CFR addresses the content of technical information required. This section stipulates that the submittal address the design and operating characteristics, unusual or novel design features, and principal safety considerations.

On April 1, 2009, Mr. Kenneth Wilson of your staff informed the NRC staff during a public meeting between the licensee and the NRC that the licensee intends to withdraw this license

amendment request and resubmit it, either in parallel with the extended power uprate (EPU) application submittal, or after the EPU amendment approval. Therefore, your proposed change is considered to be withdrawn and the NRC staff activities on the review have ceased. If this information is not correct, please notify the NRC staff of your intended actions within 10 days of receipt of this letter.

The NRC staff review to date has identified that your application does not provide the following technical and regulatory information in sufficient detail to enable the staff to complete its detailed review. Therefore, if you decide to resubmit the request, it must address the following concerns:

- The subject LAR is missing significant analyses or evaluations. The licensee requests this LAR for post EPU conditions at CR-3, with implementation during refueling outage 17, which is scheduled for fall 2011. The EPU conditions, not previously reviewed by the NRC, must be already accepted for the CLIIP to be used. Therefore, the amendment cannot be reviewed under the CLIIP and the model application including the model "Proposed No Significant Hazards Consideration Determination," as published in *Federal Register* dated March 15, 2007 (72 FR 12217), is not applicable to this LAR.
- Secondly, the safety analyses for the EPU conditions proposed in the LAR were not submitted. In accordance with 10 CFR 50.36, "The technical specifications will be derived from the analyses and evaluation included in the safety analysis report, and amendments thereto, submitted pursuant to § 50.34." This amendment attempts to establish the technical specification prior to establishing the analyses. These deficiencies impede the completion of the acceptance review.
- To complete the acceptance review, the licensee should submit the safety analyses that support the proposed changes and provide justifications for the changes. Supporting information for the site-specific information and justification should be provided in the "brackets" for TSTF-490, Babcock & Wilcox Operating Group Standard Technical Specification. It is preferred that the calculations and Final Safety Analysis Report changes be submitted with the supplements to the application, since this will increase the efficiency of the review.

If you have any questions, please contact me at (301) 415-1447.

Sincerely,



Farideh E. Saba, Senior Project Manager  
Plant Licensing Branch II-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-302

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