



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 27, 2009

Mr. Rick A. Muench
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, KS 66839

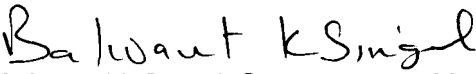
SUBJECT: WOLF CREEK GENERATING STATION – REVIEW OF INSPECTION
REPORTS FOR 2008 STEAM GENERATOR TUBE INSPECTIONS
PERFORMED DURING 16TH REFUELING OUTAGE (TAC NO. ME0124)

Dear Mr. Muench:

By letter dated October 27, 2008 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML083090879), as supplemented by letter dated January 29, 2009 (ADAMS Accession No. ML090430507), Wolf Creek Nuclear Operating Corporation (the licensee) submitted information summarizing the results of the 2008 steam generator tube inspections at the Wolf Creek Generating Station (WCGS), performed during the 16th refueling outage.

The U.S. Nuclear Regulatory Commission (NRC) staff has completed its review of these reports and concludes that the licensee provided the information required by WCGS Technical Specifications and that no additional follow-up is required at this time. The results of the NRC staff's review of the report submitted by the licensee are summarized in the enclosure to this letter. Please contact me at 301-415-3016 with any questions.

Sincerely,


Balwant K. Singal, Senior Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosure
As stated

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OFFICE OF NUCLEAR REACTOR REGULATION
REVIEW OF INSPECTION REPORTS FOR 2008 STEAM GENERATOR TUBE
INSPECTIONS PERFORMED DURING 16TH REFUELING OUTAGE
WOLF CREEK NUCLEAR OPERATING CORPORATION
WOLF CREEK GENERATING STATION
DOCKET NO. STN 50-482

By letters dated October 27, 2008 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML083090879), as supplemented by letter dated January 29, 2009 (ADAMS Accession No. ML090430507), Wolf Creek Nuclear Operating Corporation (the licensee) submitted information pertaining to the 2008 steam generator (SG) tube inspections performed at Wolf Creek Generating Station (WCGS) during its 16th refueling outage in accordance with WCGS Technical Specifications (TSs). In addition, the U.S. Nuclear Regulatory Commission (NRC) staff participated in a conference call with the licensee regarding the SG tube inspection activities at WCGS during the refueling outage and the results of the discussions are summarized in NRC staff letter dated May 8, 2008 (ADAMS Accession No. ML081280305).

WCGS has four Westinghouse Model F SGs. There are 5626 thermally treated Alloy 600 tubes in each SG. The tubes have an outside diameter of 0.688-inch, a wall thickness of 0.040-inch, and are supported by stainless steel tube supports with quatrefoil-shaped holes and V-shaped chrome-plated Alloy 600 anti-vibration bars.

The licensee provided the scope, extent, methods, and results of its SG tube inspections in the documents referenced above. In addition, the licensee described corrective actions (i.e., tube plugging or repair) taken in response to the inspection findings.

Based on its review of the reports submitted, the NRC staff has the following observations and comments:

- At the end of the 16th refueling outage, WCGS was in the third sequential (60 effective full power months (EFPMs)) inservice inspection period. The SGs had operated approximately 12.6 EFPMs in this period.
- Most of the tube inspections were performed in SGs B and C. Only the tube ends were inspected in SGs A and D.
- The interim alternate repair criteria for tube-end indications was applied during the outage. Tube-end indications were found in SGs B, C, and D. One hundred percent of the tube ends were inspected in SGs B, C, and D. Thirty percent of the tube ends were inspected in SG A. Since no tube-end indications were found in this SG during the 30 percent sample, the licensee did not inspect 100 percent of the tube ends in SG A.

Enclosure

- During secondary side inspections of the feedwater ring in SG B, evidence of minor erosion on two J-nozzles was identified. The licensee evaluated the erosion and concluded no action was required. All other visual inspection results in the upper bundle of SG B were acceptable and no additional action was required.
- Scale profiling was performed using eddy current data from SGs B and C. The current deposit inventory was reported to be acceptable and below the threshold that would require further action.

Based on a review of the information provided, the NRC staff concludes that the licensee provided the information required by its TSs. In addition, the NRC staff concludes that there are no technical issues that warrant follow-up action at this time.

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/RA/

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*Memo dated 2/25/09

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