

## INSERVICE INSPECTION DOCUMENT REQUEST

**Inspection Dates:** March 30 through April 17, 2009

**Inspection Procedures:** IP 71111.08, "Inservice Inspection"

**Inspectors:** Don Jones (630) 829-9622  
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### ***I. Information Requested For The In-Office Preparation Week***

The following information (electronic copy CD ROM if possible) is requested by March 24, 2009, to facilitate the selection of specific items that will be reviewed during the on-site inspection week. The inspector will select specific items from the information requested below and request a list of additional documents needed on-site from your staff. We request that the specific items selected from the lists be available and ready for review on the first day of inspection. If you have any questions regarding this information, please call the inspector as soon as possible.

1. A detailed schedule and description of:
  - a. Non-Destructive Examinations (NDE) planned for Class 1 and 2 Systems and containment, performed as part of your ASME Code ISI Program, and NDE examinations planned for other systems performed as part of a Risk Informed (RI)-ISI Program, or other programs committed to as part of an industry initiative such as Alloy 600 inspections (MRP-139) during the scheduled inspection weeks, to include:
    - System Identification;
    - Code Class;
    - Weld Number; and
    - Exam Type (VT, UT, MT, etc.).
  - b. Steam generator (SG) tube inspection and repair activities for the upcoming outage;
  - c. Reactor vessel upper head examinations pursuant to NRC order 03-009; and
  - d. A list identifying NDE reports (ultrasonic, radiography, magnetic particle, dye penetrant, visual (VT-1, VT-2, VT-3), which have identified relevant indications on Code Class 1 and 2 Systems since the beginning of the last Unit 2 refueling outage. Also, identify the NDE examinations with recorded indications in the vessel head penetration nozzles, which have been accepted for continued service.
2. List of the welds in Code Class 1 and 2 Systems, which have been fabricated due to component repair/replacement activities since the beginning of the last Unit 2 refueling

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outage and identify the system, weld number, and reference applicable documentation.

3. If reactor vessel weld examinations required by the ASME Code are scheduled to occur during the inspection period, provide a description of the welds to be examined.

### ***II. Information to be available on-site to the inspector following the entrance meeting (March 23, 2009).***

1. For welds selected by the inspector from above, provide copies of the following documents of the weld number and location (e.g., system, train, branch):
  - a. Document with a detail of the weld construction;
  - b. Applicable Code Edition and Addenda for weldment;
  - c. Applicable Code Edition and Addenda for welding procedures;
  - d. Applicable weld procedures (WPs) used to fabricate the welds;
  - e. Copies of procedure qualification records (PQRs) supporting the WPs on selected welds;
  - f. Copies of welders' performance qualification records (WPQ);
  - g. Copies of mechanical test reports identified in the PQRs above;
  - h. Copies of the nonconformance reports for the selected welds;
  - i. Radiographs of the selected welds and access to equipment to allow viewing radiographs; and
  - j. Copies of the preservice examination records for the selected welds.
2. For the ISI related corrective action issues selected by the inspector from above, provide a copy of the corrective actions and supporting documentation.
3. For the nondestructive examination reports with relevant indications on Code Class 1 and 2 Systems selected by the inspector from above, provide a copy of the examination records and associated corrective action documents.
4. Updated schedules for item (including schedule showing contingency repair plans if available).
5. Copy of documentation including nondestructive examination reports for the last upper vessel head examination.
6. Copy of evaluation or calculation demonstrating the scope of the visual examination of the upper head will meet the 95 percent minimum coverage required by NRC Order EA-03-009 (if a visual examination is planned).

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7. Provide the susceptibility ranking calculation for the vessel head operating time and temperature. Also, provide the plant specific records (or vendor information) used to determine the inputs for this calculation.
8. Copy of the procedures, which govern the scope, equipment used, and implementation of the inspections required to identify boric acid leakage from systems and components above the vessel head.
9. Copy of any documentation of:
  - a. engineering evaluations/assessments of boric acid related deposits, and associated wastage, or corrosion for safety significant components; and
  - b. corrective actions for coolant leakage, including boric acid deposits on safety related components identified since the beginning of the last refueling outage.
10. Copy of the plant procedures used to perform inspections to identify reactor coolant system leaks or boric acid deposits and the procedures for resolution of leaks or boric acid deposits.
11. Provide a copy of the EPRI Technique Specification Sheets, which support qualification of the ET probes to be used during the upcoming SG tube inspections.
12. Provide a copy of the guidance to be followed if a loose part or foreign material is identified in the SGs.
13. Copy of document describing actions to be taken if a new SG tube degradation mechanism is identified.
14. Identify the types of SG tube repair processes, which will be implemented for defective SG tubes, (including any NRC reviews/evaluation/approval of this repair process). Provide the flaw depth sizing criteria to be applied for ET indications identified in the SG tubes.
15. Ready access to:
  - a. Sections V, IX, and XI of the ASME Code with Editions applicable to the inservice inspection program, and the repair/replacement program;
  - b. MRP-139, "Materials Reliability Program: Primary System Piping Butt Weld Inspection, and Evaluation Guideline," and associated site implementing procedures;
  - c. EPRI and industry standards referenced in the site procedures used to perform the SG tube ET examination, that includes EPRI documents: TR-107621-R1, "Steam Generator Integrity Assessment Guidelines, TR-107620-R1, "Steam Generator In-

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Situ-Pressure Test Guidelines, and 1003138, "Pressurized Water Reactor Steam Generator Examination Guidelines;

- d. Copy of SG history documentation given to vendors performing eddy current (ET) testing of the SGs during the upcoming outage;
- e. Copy of procedure containing screening criteria used for selecting tubes for in-situ-pressure testing and the procedure to be used for in-situ-pressure testing;
- f. Copy of previous outage SG tube operational assessment completed following ET of the SGs;
- g. Copy of the document defining the planned ET scope for the SGs and the scope expansion criteria, which will be used. Also, identify and describe any deviations in this scope, or expansion criteria from the EPRI Guidelines;
- h. Copy of the document describing the ET probe types, ETTS documents, and ET acquisition equipment to be used, including which areas of the SG (e.g., top of tube sheet) each probe will be used in. Also, provide your response letter(s) to Generic Letters 95-03, 95-05, 97-05, 97-06, and 04-01; and
- i. Identify and quantify any SG tube leakage experienced during the previous operating cycle. Also, provide documentation identifying which SG was leaking, and corrective actions completed, or planned for this condition.

These copies are provided to the inspector to use for the duration of the inspection at the on-site inspection location and will be returned at the end of the inspection.