



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001  
March 19, 2009

Mr. Mano Nazar  
Senior Vice President, Nuclear and  
Chief Nuclear Officer  
Florida Power and Light Company  
P.O. Box 14000  
Juno Beach, Florida 33408-0420

SUBJECT: ST. LUCIE UNIT 1 - CORRECTED PAGES FOR AMENDMENT NO. 206  
REGARDING ALTERNATE SOURCE TERM (TAC NO. MD6173)

Dear Mr. Nazar:

On November 26, 2008, the U.S. Nuclear Regulatory Commission issued Amendment No. 206 to Facility Operating License No. DPR-67 for St. Lucie Plant, Unit 1. This Amendment was in response to your application dated July 16, 2007, as supplemented by letters dated February 14, March 18, April 14, June 2, July 11, and August 13, 2008.

It recently became apparent that the amendment needed a few corrections. The list of Technical Specification pages included page 3/4 3-25, which was not part of the amendment, therefore, it has been removed from the list. Page numbers inadvertently omitted have been added. In addition, page 6-15f did not include a new section added by the previous Amendment No. 205, which also created a modified page 6-15g due to rollover.

Enclosed are the three corrected pages. We regret any inconvenience this may have caused.

Sincerely,

A handwritten signature in cursive script, reading "Siva P. Lingam", is positioned above the typed name.

Siva P. Lingam, Project Manager  
Plant Licensing Branch II-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-335

Enclosure: Corrected pages

cc w/enclosure: Distribution via Listserv

ENCLOSURE

AMENDMENT NO. 206

CORRECTED PAGES

ATTACHMENT TO LICENSE AMENDMENT NO. 206  
TO RENEWED FACILITY OPERATING LICENSE NO. DPR-67  
DOCKET NO. 50-335

Replace Page 3 of Renewed Operating License DPR-67 with the attached page 3.

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Pages

Page 1-3  
Page 3/4 3-21  
Page 3/4 3-22  
Page 3/4 3-22a  
Page 3/4 3-23  
Page 3/4 3-24  
Page 3/4 6-27  
Page 3/4 6-28  
Page 3/4 6-29  
Page 3/4 7-24  
Page 3/4 7-25  
Page 3/4 7-26  
Page 6-15b  
Page 6-15d  
Page 6-15e  
Page 6-15f  
Page 6-15g

Insert Pages

Page 1-3  
Page 3/4 3-21  
Page 3/4 3-22  
Page 3/4 3-22a  
Page 3/4 3-23  
Page 3/4 3-24  
Page 3/4 6-27  
Page 3/4 6-28  
Page 3/4 6-29  
Page 3/4 7-24  
Page 3/4 7-25  
Page 3/4 7-26  
Page 6-15b  
Page 6-15d  
Page 6-15e  
Page 6-15f  
Page 6-15g

## ADMINISTRATIVE CONTROLS (continued)

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### I. Steam Generator (SG) Program (continued)

d. Provisions for SG tube inspections. Periodic SG tube inspections shall be performed. The number and portions of the tubes inspected and methods of inspection shall be performed with the objective of detecting flaws of any type (e.g., volumetric flaws, axial and circumferential cracks) that may be present along the length of the tube, from the tube-to-tubesheet weld at the tube inlet to the tube-to-tubesheet weld at the tube outlet, and that may satisfy the applicable tube repair criteria. The tube-to-tubesheet weld is not part of the tube. In addition to meeting the requirements of d.1, d.2, and d.3 below, the inspection scope, inspection methods, and inspection intervals shall be such as to ensure that SG tube integrity is maintained until the next SG inspection. An assessment of degradation shall be performed to determine the type and location of flaws to which the tube may be susceptible and, based on this assessment, to determine which inspection methods need to be employed and at what locations.

1. Inspect 100% of the tubes in each SG during the first refueling outage following SG replacement.
2. Inspect 100% of the tubes at sequential periods of 144, 108, 72, and, thereafter, 60 effective full power months. The first sequential period shall be considered to begin after the first inservice inspection of the SGs. In addition, inspect 50% of the tubes by the refueling outage nearest the midpoint of the period and the remaining 50% by the refueling outages nearest the end of the period. No SG shall operate for more than 72 effective full power months or three refueling outages (whichever is less) without being inspected.
3. If crack indications are found in any SG tube, then the next inspection for each SG for the degradation mechanism that caused the crack indication shall not exceed 24 effective full power months or one refueling outage (whichever is less). If definitive information, such as from examination of a pulled tube, diagnostic non-destructive testing, or engineering evaluation indicates that a crack-like indication is not associated with a crack(s), then the indication need not be treated as a crack.

e. Provisions for monitoring operational primary-to-secondary leakage.

### m. Control Room Envelope Habitability Program

A Control Room Envelope (CRE) Habitability Program shall be established and implemented to ensure that CRE habitability is maintained such that, with an OPERABLE Control Room Emergency Ventilation System (CREVS), CRE occupants can control the reactor safely under normal conditions and maintain it in a safe condition following a radiological event, hazardous chemical release, or a smoke challenge. The program shall ensure that adequate radiation protection is provided to permit access and occupancy of the CRE under design basis accident (DBA) conditions without personnel receiving radiation exposures in excess of 5 rem total effective dose equivalent (TEDE) for the duration of the accident.

The program shall include the following elements:

- a. The definition of the CRE and the CRE boundary.
- b. Requirements for maintaining the CRE boundary in its design condition including configuration control and preventive maintenance.

## **ADMINISTRATIVE CONTROLS (continued)**

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- m. Control Room Envelope Habitability Program (continued)
  - c. Requirements for (i) determining the unfiltered air leakage past the CRE boundary into the CRE in accordance with the testing methods and at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, "Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors," Revision 0, May 2003, and (ii) assessing CRE habitability at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, Revision 0.
  - d. Measurement, at designated locations, of the CRE pressure relative to all external areas adjacent to the CRE boundary during the pressurization mode of operation by one train of the CREVS, operating at the flow rate required by the VFTP, at a Frequency of 36 months on a STAGGERED TEST BASIS. The results shall be trended and used as part of the 36 month assessment of the CRE boundary.
  - e. The quantitative limits on unfiltered air leakage into the CRE. These limits shall be stated in a manner to allow direct comparison to the unfiltered air leakage measured by the testing described in paragraph c. The unfiltered air leakage limit for radiological challenges is the leakage flow rate assumed in the licensing basis analyses of DBA consequences. Unfiltered air leakage limits for hazardous chemicals must ensure that exposure of CRE occupants to these hazards will be within the assumptions in the licensing basis.
  - f. The provisions of SR 4.0.2 are applicable to the Frequencies for assessing CRE habitability, determining CRE unfiltered leakage, and measuring CRE pressure and assessing the CRE boundary as required by paragraphs c and d, respectively.

## **6.9 REPORTING REQUIREMENTS**

### **ROUTINE REPORTS**

- 6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the NRC.

### **STARTUP REPORT**

- 6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment of the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal or hydraulic performance of the plant.

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Siva P. Lingam, Project Manager  
Plant Licensing Branch II-2  
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