



U.S. Nuclear Regulatory Commission


[Home](#)
[Who We Are](#)
[What We Do](#)
[Nuclear
Reactors](#)
[Nuclear
Materials](#)
[Radioactive
Waste](#)
[Public
Involvement](#)
[Electronic
Reading Room](#)

[Home](#) > [Electronic Reading Room](#) > [Document Collections](#) > [NRC Formal \(NUREG-Series\) Publications](#) > [Staff Reports](#) > NUREG-0800

Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants, (NUREG-0800)

On this page:

- [Chapter 3 - Design of Structures, Components, Equipment, and Systems](#)
- [Chapter 4 - Reactor](#)
- [Chapter 5 - Reactor Coolant System and Connected Systems](#)
- [Chapter 6 - Engineered Safety Features](#)
- [Chapter 7 - Instrumentation and Controls](#)
- [Chapter 8 - Electric Power](#)
- [Chapter 9 - Auxiliary Systems](#)
- [Chapter 10 - Steam and Power Conversion System](#)
- [Chapter 11 - Radioactive Waste Management](#)
- [Chapter 12 - Radiation Protection](#)
- [Chapter 13 - Conduct of Operations](#)
- [Chapter 14 - Initial Test Program and ITAAC-Design Certification](#)
- [Chapter 15 - Accident Analysis](#)
- [Chapter 18 - Human Factors Engineering](#)
- [Chapter 19 - Severe Accidents](#)

All links on this page are to documents in portable document format (PDF). See our [Plugins, Viewers, and Other Tools](#) page for more information.

Chapter 3 - Design of Structures, Components, Equipment, and Systems

Section	Title
3.4.1	Flood Protection, Revision 2, July 1981
3.5.1.1	Internally Generated Missiles (Outside Containment), Revision 2, July 1981
3.5.1.2	Internally Generated Missiles (Inside Containment), Revision 2, July 1981
3.6.1	Plant Design for Protection Against Postulated Piping Failures in Fluid Systems Outside Containment, Revision 1, July 1981
3.6.2	Determination of Rupture Locations and Dynamic Effects Associated with the Postulated Rupture of Piping, Draft Revision 2, April 1996
3.9.1	Special Topics for Mechanical Components, Draft Revision 3, April 1996
3.9.2	Dynamic Testing and Analysis of Systems, Components, and Equipment, Draft Revision 3, April 1996
3.9.3	ASME Code Class 1, 2, and 3 Components, Component Supports, and Core Support Structures, Draft Revision 2, April 1996

3.9.5	Reactor Pressure Vessel Internals, Draft Revision 3, April 1996
3.9.6	Inservice Testing of Pumps and Valves, Draft Revision 3, April 1996
3.10	Seismic and Dynamic Qualification of Mechanical and Electrical Equipment, Draft Revision 3, April 1996
3.11	Environmental Qualification of Mechanical and Electrical Equipment, Draft Revision 3, April 1996

Chapter 4 - Reactor

Section	Title
4.2	Fuel System Design, Draft Revision 3, April 1996
4.3	Nuclear Design, Draft Revision 3, April 1996
4.4	Thermal and Hydraulic Design, Draft Revision 2, April 1996
4.5.1	Control Rod Drive Structural Materials, Draft Revision 3, April 1996
4.5.2	Reactor Internal and Core Support Materials, Draft Revision 3, April 1996
4.6	Functional Design of Control Rod Drive System, Draft Revision 2, April 1996

Chapter 5 - Reactor Coolant System and Connected Systems

Section	Title
5.2.1.1	Compliance with the Codes and Standards Rule, 10 CFR 50.55a, Draft Revision 3, April 1996
5.2.2	Overpressure Protection, Draft Revision 3, April 1996
5.2.3	Reactor Coolant Pressure Boundary Materials, Draft Revision 3, April 1996
5.2.4	Reactor Coolant Pressure Boundary Inservice Inspection and Testing, Draft Revision 2, April 1996
5.3.1	Reactor Vessel Materials, Draft Revision 2, April 1996
5.3.2	Pressure-Temperature Limits and Pressurized Thermal Shock, Draft Revision, Draft Revision 2, April 1996
5.3.3	Reactor Vessel Integrity, Draft Revision 2, April 1996
5.4.2.2	Steam Generator Tube Inservice Inspection, Draft Revision 2, April 1996
5.4.6	Reactor Core Isolation Cooling System (BWR), Draft Revision 4, April 1996
5.4.7	Residual Heat Removal (RHR) System, Draft Revision 4, April 1996
5.4.8	Reactor Water Cleanup System (BWR), Draft Revision 3, April 1996
5.4.11	Pressurizer Relief Tank, Revision 2, July 1981

Chapter 6 - Engineered Safety Features

Section	Title
6.1.1	Engineered Safety Features Materials, Draft Revision 2, April 1996
6.1.2	Protective Coating Systems (Paints) - Organic Materials, Draft Revision 3, April 1996
6.2.1	Containment Functional Design, Revision 2, July 1981
6.2.1.1.A	PWR Dry Containments, Including Subatmospheric Containments, Revision 2, July 1981
6.2.1.1.B	Ice Condenser Containments, Revision 2, July 1981
6.2.1.1.C	Pressure-Suppression Type BWR Containments, Revision 6, August 1984
6.2.1.2	Subcompartment Analysis, Revision 2, July 1981
6.2.1.3	Mass and Energy Release Analysis for Postulated Loss-of-Coolant, Revision 1, July 1981
6.2.1.4	Mass and Energy Release Analysis for Postulated Secondary System Pipe Ruptures, Revision 1, July 1981
6.2.1.5	Minimum Containment Pressure Analysis for Emergency Core Cooling System Performance Capability Studies, Revision 2, July 1981
6.2.2	Containment Heat Removal Systems, Revision 4, October 1985
6.2.3	Secondary Containment Functional Design, Revision 2, July 1981
6.2.5	Combustible Gas Control in Containment, Revision 2, July 1981
6.3	Emergency Core Cooling System, Draft Revision 3, April 1996
6.4	Control Room Habitability System, Draft Revision 3, April 1996
6.5.1	ESF Atmosphere Cleanup Systems, Revision 2, July 1981
6.5.3	Fission Product Control Systems and Structures, Revision 2, July 1981
6.7	Main Steam Isolation Valve Leakage Control System (BWR), Revision 2, July 1981

Chapter 7 - Instrumentation and Controls

Section	Title
7.0 - 7.8	Instrumentation and Controls, Revision 4, June 1997

Chapter 8 - Electric Power

Section	Title
8.1	Electric Power - Introduction, Draft Revision 3, April 1996
8.2	Offsite Power System, Draft Revision 4, April 1996
Appendix 8-A	Offsite Power System, Draft Revision 4, April 1996
Appendix 8-B	Offsite Power System, Draft Revision 4, April 1996
8.3.1	A-C Power Systems (Onsite), Draft Revision 3, April 1996

8.3.2	D-C Power Systems (Onsite), Draft Revision 3, April 1996
-----------------------	--

Chapter 9 - Auxiliary Systems

Section	Title
9.1.1	New Fuel Storage, Draft Revision 3, April 1996
9.1.2	Spent Fuel Storage, Draft Revision 4, April 1996
9.1.3	Spent Fuel Pool Cooling and Cleanup System, Revision 1, July 1981
9.1.4	Light Load Handling System (Related to Refueling), Revision 2, July 1981
9.2.1	Station Service Water System, Revision 4, June 1985
9.2.2	Reactor Auxiliary Cooling Water Systems, Revision 3, June 1986
9.2.5	Ultimate Head Sink, Revision 2, July 1981
9.3.3	Equipment and Floor Drainage System, Revision 2, July 1981
9.3.4	Chemical and Volume Control System (PWR) (Including Boron Recovery System), Draft Revision 3, April 1996
9.3.5	Standby Liquid Control System (BWR), Draft Revision 3, April 1996
9.4.1	Control Room Area Ventilation System, Revision 2, July 1981
9.4.2	Spent Fuel Pool Area Ventilation System, Revision 2, July 1981
9.4.3	Auxiliary and Radwaste Area Ventilation System, Revision 2, July 1981
9.4.4	Turbine Area Ventilation System, Revision 2, July 1981
9.4.5	Engineered Safety Feature Ventilation System, Revision 2, July 1981
9.5.1	Fire Protection Program, Revision 3, July 1981
9.5.4	Emergency Diesel Engine Fuel Oil Storage and Transfer System, Revision 2, July 1981

Chapter 10 - Steam and Power Conversion System

Section	Title
10.2	Turbine Generator, Revision 2, July 1981
10.3	Main Steam Supply System, Revision 3, April 1984
10.4.1	Main Condensers, Revision 2, July 1981
10.4.2	Main Condenser Evacuation System, Revision 2, July 1981
10.4.3	Turbine Gland Sealing System, Revision 2, July 1981
10.4.4	Turbine Bypass System, Revision 2, July 1981
10.4.5	Circulating Water System, Revision 2, July 1981
10.4.7	Condensate and Feedwater System, Revision 3, April 1984

10.4.8	Steam Generator Blowdown System (PWR), Draft Revision 3, April 1996
10.4.9	Auxiliary Feedwater System (PWR), Revision 2, July 1981

Chapter 11 - Radioactive Waste Management

Section	Title
11.1	Source Terms, Draft Revision 3, April 1996
11.2	Liquid Waste Management Systems, Draft Revision 3, April 1996
11.3	Gaseous Waste Management Systems, Draft Revision 3, April 1996
11.4	Solid Waste Management Systems, Draft Revision 3, April 1996

Chapter 12 - Radiation Protection

Section	Title
12.2	Radiation Sources, Draft Revision 3, April 1996
12.3 - 12.4	Radiation Protection Design Features, Draft Revision 3, April 1996
12.5	Operational Radiation Protection Program, Draft Revision 3, April 1996

Chapter 13 - Conduct of Operations

Section	Title
13.2.1	Draft Report for Comment - Reactor Operator Training, Draft Revision 2, December 2002
13.2.2	Draft Report for Comment - Training for Nonlicensed Plant Staff, Draft Revision 2, December 2002
13.5.2.1	Draft Report for Comment - Operating and Emergency Operating Procedures, Draft Revision 1, December 2002

Chapter 14 - Initial Test Program and ITAAC-Design Certification

Section	Title
14.2.1	Draft Report for Comment - Generic Guidelines for Extended Power Uprate Testing Programs, Draft Revision 0, December 2002

Chapter 15 - Accident Analysis

Section	Title
15.0.1	Radiological Consequence Analyses Using Alternative Source Terms, Revision 0, July 2000
15.1.1 - 15.1.4	Decrease in Feedwater Temperature, Increase in Feedwater Flow, Increase in Steam Flow, and Inadvertent Opening of a Steam Generator Relief or Safety Valve, Draft Revision 2, April 1996

15.1.5	Steam System Piping Failures Inside and Outside of Containment (PWR), Draft Revision 3, April 1996
15.1.5.A	Radiological Consequences of Main Steam Line Failures Outside Containment of a PWR, Draft Revision 3, April 1996
15.0.2	Review of Transient and Accident Analysis Methods
15.2.1 - 15.2.5	Loss of External Load; Turbine Trip; Loss of Condenser Vacuum; Closure of Main Steam Isolation Valve (BWR); and Steam Pressure Regulator Failure (Closed), Draft Revision 2, April 1996
15.2.6	Loss of Nonemergency AC Power to the Station Auxiliaries, Draft Revision 2, April 1996
15.2.7	Loss of Normal Feedwater Flow, Draft Revision 2, April 1996
15.2.8	Feedwater System Pipe Breaks Inside and Outside Containment, Draft Revision 2, April 1996
15.3.1 - 15.3.2	Loss of Forced Reactor Coolant Flow Including Trip of Pump Motor and Flow Controller Malfunctions, Draft Revision 2, April 1996
15.3.3 - 15.3.4	Reactor Coolant Pump Motor Seizure and Reactor Coolant Pump Shaft Break, Draft Revision 3, April 1996
15.4.1	Uncontrolled Control Rod Assembly Withdrawal from a Subcritical or Low Power Startup Condition, Draft Revision 3, April 1996
15.4.2	Uncontrolled Rod Assembly Withdrawal at Power, Draft Revision 3, April 1996
15.4.3	Control Rod Misoperation (System Malfunction or Operator), Draft Revision 3, April 1996
15.4.4 - 15.4.5	Startup of an Inactive Loop or Recirculation Loop at an Incorrect Temperature, and Flow Controller Malfunction Causing an Increase in BWR Core Flow Rate, Draft Revision 2, April 1996
15.4.6	Chemical and Volume Control System Malfunction that Results in a Decrease in Boron Concentration in the Reactor Coolant (PWR), Draft Revision 2, April 1996
15.4.7	Inadvertent Loading and Operation of a Fuel Assembly in an Improper Position, Draft Revision 2, April 1996
15.4.8	Spectrum of Rod Ejection Accidents (PWR), Draft Revision 3, April 1996
15.4.8.A	Radiological Consequences of a Control Rod Ejection Accident (PWR), Draft Revision 2, April 1996
15.4.9	Spectrum of Rod Drop Accidents (BWR), Draft Revision 3, April 1996
15.4.9.A	Radiological Consequences of Control Rod Drop Accident (BWR), Draft Revision 3, April 1996
15.5.1 - 15.5.2	Inadvertent Operation of ECCS and Chemical and Volume Control System Malfunction that Increases Reactor Coolant Inventory, Draft Revision 2, April 1996
15.6.1	Inadvertent Opening of a PWR Pressurizer Pressure Relief Valve or a BWR Pressure Relief Valve, Draft Revision 2, April 1996
15.6.2	Radiological Consequences of the Failure of Small Lines Carrying Primary Coolant Outside Containment, Draft Revision 3, April 1996
15.6.3	Radiological Consequences of Steam Generator Tube Failure, Draft Revision 3, April 1996
15.6.4	Radiological Consequences of Main Steam Line Failure Outside Containment (BWR), Draft Revision 3, April 1996

15.6.5	Loss-of-Coolant Accidents Resulting From Spectrum of Postulated Piping Breaks Within the Reactor Coolant Pressure Boundary, Draft Revision 3, April 1996
15.6.5.A	Radiological Consequences of a Design Basis Loss-of-Coolant Accident including Containment Leakage Contribution, Draft Revision 2, April 1996
15.6.5.B	Radiological Consequences of a Design Basis Loss-of-Coolant Accident Including Containment Leakage Contribution, Draft Revision 2, April 1996
15.6.5.D	Radiological Consequences of a Design Basis Loss-of-Coolant Accident: Leakage From Main Steam Isolation Valve Leakage Control System (BWR), , Draft Revision 2, April 1996
15.7.4	Radiological Consequences of Fuel Handling Accidents, Draft Revision 2, April 1996
15.7.5	Spent Fuel Cask Drop Accidents, Draft Revision 3, April 1996

Chapter 18 - Human Factors Engineering

Section	Title
18	Draft Report for Comment - Human Factors Engineering, Draft Revision 1, December 2002

Chapter 19 - Severe Accidents

Section	Title
19	Use of Probabilistic Risk Assessment in Plant-specific, Risk-informed Decisionmaking: General Guidance
19.1	Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities

[Privacy Statement](#) | [Site Disclaimer](#)
Last revised Tuesday, January 28, 2003