

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

February 26, 2009

Vice President, Operations Entergy Nuclear Operations, Inc. Palisades Nuclear Plant 27780 Blue Star Memorial Highway Covert, MI 49043-9530

SUBJECT: PALISADES NUCLEAR PLANT - UPCOMING STEAM GENERATOR TUBE INSERVICE INSPECTION (TAC NO. ME0692)

Dear Sir or Madam:

Inservice inspections of steam generator (SG) tubes play a vital role in assuring SG tube integrity. The reporting requirements of plant Technical Specifications typically require submission of a report within 180 days following completion of the inspection. The content of the report includes the following:

- 1. The scope of inspections performed on each SG,
- 2. Active degradation mechanisms found,
- 3. Nondestructive examination techniques utilized for each degradation mechanism,
- 4. Location, orientation (if linear), and measured sizes (if available) of service induced indications,
- 5. Number of tubes plugged during the inspection outage for each active degradation mechanism,
- 6. Total number and percentage of tubes plugged to date, and
- 7. The results of condition monitoring, including the results of tube pulls and in-situ testing.

A phone conference has been arranged with members of your staff to discuss the ongoing results of the SG tube inspections to be conducted during the upcoming Palisades Nuclear Plant refueling outage. This phone call will occur after the majority of the tubes have been inspected, but before the SG inspection activities have been completed. Attached is a list of discussion points to facilitate this phone conference.

The U.S. Nuclear Regulatory Commission (NRC) staff plans to document a summary of the conference call, including any material that you provide to the NRC staff in support of the call.

Sincerely,

Thang Mangat for

Mahesh Chawla, Project Manager Plant Licensing Branch III-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket No. 50-255

Enclosure: SG Tube Inspection Discussion Points

cc w/encl: Distribution via ListServ

STEAM GENERATOR TUBE INSPECTION DISCUSSION POINTS

PREPARED BY THE OFFICE OF NUCLEAR REACTOR REGULATION

PALISADES NUCLEAR PLANT

DOCKET NO. 50-255

The following discussion points have been prepared to facilitate the conference call arranged with the Palisades licensee to discuss the results of the steam generator (SG) tube inspections to be conducted during the upcoming spring 2009 refueling outage. This conference call is scheduled to occur towards the end of the planned SG tube inspections, but before the unit completes the inspections and repairs.

The U.S. Nuclear Regulatory Commission staff plans to document a summary of the conference call, as well as any material that is provided in support of the call.

- 1. Discuss any trends in the amount of primary-to-secondary leakage observed during the recently completed cycle.
- 2. Discuss whether any secondary side pressure tests were performed during the outage and the associated results.
- 3. Discuss any exceptions taken to the industry guidelines.
- 4. For each steam generator, provide a description of the inspections performed including the areas examined and the probes used (e.g., dents/dings, sleeves, expansion-transition, U-bends with a rotating probe), the scope of the inspection (e.g., 100 percent of dents/dings greater than 5 volts and a 20 percent sample between 2 and 5 volts), and the expansion criteria.
- 5. For each area examined (e.g., tube supports, dent/dings, sleeves, etc.), provide a summary of the number of indications identified to-date for each degradation mode (e.g., number of circumferential primary water stress corrosion cracking indications at the expansion transition). For the most significant indications in each area, provide an estimate of the severity of the indication (e.g., provide the voltage, depth, and length of the indication). In particular, address whether tube integrity (structural and accident induced leakage integrity) was maintained during the previous operating cycle. In addition, discuss whether any location exhibited a degradation mode that had not previously been observed at this location at this unit (e.g., observed circumferential primary water stress corrosion cracking at the expansion transition for the first time at this unit).
- 6. Describe repair/plugging plans.
- 7. Describe in-situ pressure test and tube pull plans and results (as applicable and if available).

- 8. Discuss the following regarding loose parts:
 - what inspections are performed to detect loose parts
 - a description of any loose parts detected and their location within the SG (including the source or nature of the loose part, if known)
 - if the loose parts were removed from the SG
 - indications of tube damage associated with the loose parts
- 9. Discuss the scope and results of any secondary side inspection and maintenance activities (e.g., in-bundle visual inspections, feedring inspections, sludge lancing, assessing deposit loading, etc).
- 10. Discuss any unexpected or unusual results.
- 11. Provide the schedule for SG-related activities during the remainder of the current outage.

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The U.S. Nuclear Regulatory Commission (NRC) staff plans to document a summary of the conference call, including any material that you provide to the NRC staff in support of the call.

Sincerely,

/RA/ by TWengert for

Mahesh Chawla, Project Manager Plant Licensing Branch III-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket No. 50-255

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* Per email dated 2/20/2009 NRR-106

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