



JAMES R. MORRIS, VICE PRESIDENT

Duke Energy Carolinas, LLC  
Catawba Nuclear Station  
4800 Concord Road / CNO1VP  
York, SC 29745

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February 19, 2009

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, D.C. 20555

Subject: Duke Energy Carolinas, LLC (Duke)  
Catawba Nuclear Station, Unit 2  
Docket Number 50-414  
Proposed Technical Specifications (TS) Amendment  
TS 5.5.9, "Steam Generator (SG) Program"  
TS 5.6.8, "Steam Generator (SG) Tube Inspection  
Report"

Reference: Letters from Duke to NRC, same subject, dated  
November 13, 2008 and February 5, 2009

The reference letters collectively comprise Duke's request for an amendment to Catawba Facility Operating License NPF-52 and the subject TS. The amendment request proposed a one-cycle revision to the subject TS to incorporate an Interim Alternate Repair Criterion (IARC) in the provisions for SG tube repair criteria during the End of Cycle 16 Refueling Outage and subsequent Cycle 17 operation.

On February 12, 2009, a telephone conference call was held among members of Duke and the NRC staff. Based on the results of this call, Duke is revising the Facility Operating License condition proposed in the November 13, 2008 reference letter. The revised condition is included in the attachment to this letter.

The original regulatory and environmental analyses supporting the amendment request are not impacted by this amendment request supplement.

There are no new NRC commitments contained in this letter or its attachment.

A001  
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Pursuant to 10 CFR 50.91, a copy of this letter and its attachment is being sent to the designated official of the State of South Carolina.

Inquiries on this matter should be directed to L.J. Rudy at (803) 701-3084.

Very truly yours,

A handwritten signature in cursive script, appearing to read "James R. Morris".

James R. Morris

LJR/s

Attachment

February 19, 2009

James R. Morris affirms that he is the person who subscribed his name to the foregoing statement, and that all the matters and facts set forth herein are true and correct to the best of his knowledge.

*James R. Morris*

James R. Morris, Vice President

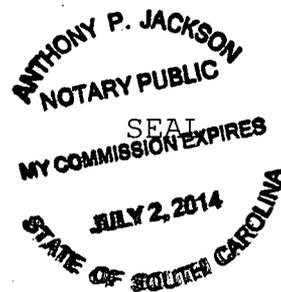
Subscribed and sworn to me:

2/19/09  
Date

*Anthony P. Jackson*  
Notary Public

My commission expires:

7/2/2014  
Date



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xc (with attachment):

L.A. Reyes  
U.S. Nuclear Regulatory Commission  
Regional Administrator, Region II  
Atlanta Federal Center  
61 Forsyth St., SW, Suite 23T85  
Atlanta, GA 30303

A.T. Sabisch  
Senior Resident Inspector (CNS)  
U.S. Nuclear Regulatory Commission  
Catawba Nuclear Station

J.H. Thompson (addressee only)  
NRC Project Manager (CNS)  
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S.E. Jenkins  
Section Manager  
Division of Waste Management  
South Carolina Department of Health and Environmental  
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2600 Bull St.  
Columbia, SC 29201

ATTACHMENT

REVISED FACILITY OPERATING LICENSE CONDITION

<u>Amendment Number</u>	<u>Additional Condition</u>	<u>Implementation Date</u>
165	<p>The schedule for the performance of new and revised surveillance requirements shall be as follows:</p> <p>For surveillance requirements (SRs) that are new in Amendment No. 165 the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment No. 165. For SRs that existing prior to Amendment No. 165, including SRs with modified acceptance criteria and SRs who intervals of performance are being extended, the first performance is due at the end of the first surveillance interval that begins on the date the surveillance was last performed prior to implementation of amendment No. 165. For SRs that existed prior to Amendment No. 165, whose intervals of performance are being reduced, the first reduced surveillance interval begins upon completion of the first surveillance performed after implementation of Amendment No. 165</p>	By January 31, 1999
172	The maximum rod average burnup for any rod shall be limited to 60 GWd/mtU until the completion of an NRC environmental assessment supporting an increased limit.	Within 30 days of date of amendment.
	For steam generator (SG) integrity assessments, the ratio of 2.5 will be used in completion of both the Condition Monitoring (CM) and the Operational Assessment (OA) upon implementation of the Interim Alternate Repair Criterion (IARC). For example, for the CM assessment, the component of leakage from the lower 4 inches of the most limiting SG during the prior cycle of operation will be multiplied by a factor of 2.5 and added to the total leakage from any other source and compared to the allowable accident analysis leakage assumption. For the OA, the difference in leakage from the allowable limit during the limiting design basis accident minus the leakage from the other sources will be divided by 2.5 and compared to the observed leakage. An administrative limit will be established to not exceed the calculated value.	Prior to any entry into Mode 4 during Cycle 17 operation

Renewed License No. NPF-52  
Amendment No.