



HITACHI

GE Hitachi Nuclear Energy

MFN 09-086
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U.S. Nuclear Regulatory Commission
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Subject: Request for Information to Support the Assessment of the Impact of Two-Sided Oxidation

The following information is provided in response to the Reference 1 request.

Plant Type	BWR/2	BWR/3
NRC Approved GE Methodology	SAFER-CORCL	SAFER/GESTR
Clad O.D. [in]	0.440	0.404
Clad Thickness [in]	0.028	0.026
PCT (PCT node) [°F]	1976	2123
ECR (PCT node) [%]	14.76	8.92
PCT (rupture node) [°F]	1976	2123
ECR (rupture node) [%]	14.76	8.92
Strain at Ruptured Node [%]	46	48
Flow Area Reduction in Rupture Region [%]	N/A	N/A

In both SAFER-CORCL and SAFER/GESTR methodologies, used in external pump and jet pump designs, respectively, cladding rupture occurs in the PCT node. Therefore, the PCT in PCT node and the PCT in the burst node are the same. The methods do not credit cooling effects of the rupture and disregard the flow area reduction in the rupture region.

Please contact me or Dr. Kurshad Muftuoglu at 910-819-4556 if you have additional questions about this information.

Sincerely,

James F. Harrison
Vice President, Fuels Licensing
Regulatory Affairs
GE Hitachi Nuclear Energy

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Project No. 710

Reference: Letter from WH Ruland (US NRC) to RE Brown (GEH), Subject: Request for Information to Support the Assessment of the Impact of Two-Sided Oxidation, November 24, 2008, MFN 08-941.

cc: MC Honcharik, USNRC
SS Philpott, USNRC
WH Ruland, USNRC
PM Clifford, USNRC
PL Campbell, GEH/Washington
AA Lingenfelter, GNF/Wilmington
RE Brown, GEH/Wilmington
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