MITSUBISHI HEAVY INDUSTRIES, LTD.

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TOKYO, JAPAN

January 29, 2009

Document Control Desk U.S. Nuclear Regulatory Commission Washington, DC 20555-0001

Attention: Mr. Jeffrey A. Ciocco

Docket No. 52-021 MHI Ref: UAP-HF-09022

Subject: MHI's Response to US-APWR DCD RAI No. 126-1558 Revision 0

Reference: 1) "Request for Additional Information No. 126-1558 Revision 0, SRP Section: 06.02.01, Application Section: 6.2.1" dated December16, 2008.

With this letter, Mitsubishi Heavy Industries, Ltd. ("MHI") transmits to the U.S. Nuclear Regulatory Commission ("NRC") a document entitled "Response to Request for Additional Information No. 126-1558 Revision 0."

Enclosed is the response to one RAI contained within Reference 1. Of these RAIs, the following four RAIs will not be answered within this package.

06.02.01-3 06.02.01-4 06.02.01-5 06.02.01-6

MHI will need additional sensitivities or confirmatory analyses for the responses to these RAIs. The responses to RAIs (06.02.01-3, 06.02.01-4, 06.02.01-5) will be submitted by 22nd March and to RAI (06.02.01-6) will be submitted by 22nd April.

Please contact Dr. C. Keith Paulson, Senior Technical Manager, Mitsubishi Nuclear Energy Systems, Inc. if the NRC has questions concerning any aspect of the submittals. His contact information is below.

Sincerely,

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Yoshiki Ogata General Manager- APWR Promoting Department Mitsubishi Heavy Industries, LTD.



Enclosure:

1. Response to Request for Additional Information No. 126-1558 Revision 0

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CC: J. A. Ciocco C. K. Paulson

Contact Information

C. Keith Paulson, Senior Technical Manager Mitsubishi Nuclear Energy Systems, Inc. 300 Oxford Drive, Suite 301 Monroeville, PA 15146 E-mail: ck_paulson@mnes-us.com Telephone: (412) 373-6466

Docket No. 52-021 MHI Ref: UAP-HF-09022

Enclosure 1

UAP-HF-09022 Docket Number 52-021

Response to Request for Additional Information No. 126-1558 Revision 0

January 2009

RESPONSE TO REQUEST FOR ADDITIONAL INFORMATION

1/29/2009

US-APWR Design Certification Mitsubishi Heavy Industries Docket No. 52-021

RAI NO.:NO.126-1558 REVISION 0SRP SECTION:06.02.01 - CONTAINMENT FUNCTIONAL DESIGNAPPLICATION SECTION:6.2.1DATE OF RAI ISSUE:12/16/2008

QUESTION NO. : 06.02.01-2

DCD Section 6.2.1.1.1, page 6.2-2, third paragraph states that Table 6.2.1-1 "summarizes containment temperature and pressure, for a <u>broad range</u> of postulated breaks". However, this table includes only one break. Please, either include in the table all the cases that were analyzed, or change the table description in the text to something like "for the worst break or for the worst case."

ANSWER:

Description of the indicated phrase and the title of Table 6.2.1-1 will be changed. See Attachment.

Impact on DCD

The DCD will be changed as the description in Attachment.

Impact on COLA

There is no impact on the COLA

Impact on PRA

There is no impact on the PRA

Attachment

Editorial:

The second paragraph of the subsection 6.2.1.1.1 of DCD Revision 1 will be changed as follows:

Table 6.2.1-1 summarizes containment temperature and pressure (and comparisons to design pressure), for a broad-range the worst case of postulated breaks, and assumed system and component failures. Figure 6.2.1-1 through Figure 6.2.1-4 are plots of containment internal pressure and temperature versus time for the most severe primary and secondary system piping failures. These figures show that internal containment pressure is reduced to less than 50% of the peak value 24 hours after event initiation.

Editorial:

The title of Table 6.2.1-1, page 6.2-65 of DCD Revision 1 will be changed as follows:

 Table 6.2.1-1 Summary of Calculated Containment Temperature and Pressure

 Results for a Range the Worst Case of Postulated Piping Failure Scenarios