



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
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ARLINGTON, TEXAS 76011-4125

January 23, 2009

John T. Conway
Senior Vice President &
Chief Nuclear Officer
Pacific Gas and Electric Company
P. O. Box 3
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Avila Beach, CA 93424

SUBJECT: DIABLO CANYON POWER PLANT - NRC RADIATION SAFETY TEAM
INSPECTION REPORT 05000275/2008009 AND 05000323/2008009

Dear Mr. Conway:

On December 11, 2008, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at your Diablo Canyon Power Plant, Units 1 and 2, facility. The enclosed Radiation Safety Team inspection report documents the inspection findings which were discussed with Mr. K. Peters, Station Director, and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license.

The team reviewed selected procedures and records, observed activities, and interviewed personnel. Specifically, the team evaluated the inspection areas within the Radiation Protection Strategic Performance Area that are scheduled for review every two years. These areas are:

- Radiation Monitoring Instrumentation
- Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems
- Radioactive Material Processing and Transportation
- Radiological Environmental Monitoring Program and Radioactive Material Control Program

This inspection report documents one NRC-identified violation. However, because the finding was a Severity Level IV violation and was entered into your corrective action program, the NRC is treating this finding as a non-cited violation consistent with Section VI.A.1 of the NRC Enforcement Policy. If you contest this non-cited violation or its significance, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to

the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C., 20555-0001; with copies to the Regional Administrator, U.S. Nuclear Regulatory Commission Region IV, 612 E. Lamar Blvd, Suite 400, Arlington, Texas 76011-4005; the Director, Office of Enforcement, U.S. Nuclear Regulatory Commission, Washington DC 20555-001; and the NRC Resident Inspector at the Diablo Canyon Power Plant.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be made available electronically for public inspection in the NRC Public

Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Gregory E. Werner, Chief
Plant Support Branch 2
Division of Reactor Safety

Dockets: 50-275, 50-323
Licenses: DPR-80, DPR-82

Enclosure:
NRC Inspection Report 05000275/2008009
and 05000323/2008009
w/Attachment: Supplemental Information

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Sunsi Review Completed: ADAMS: Initials:

Publicly Available Non-Publicly Available Sensitive Non-Sensitive

RIV:DRS/PSB	PSB	PSB	PSB	C:PSB2
LTRicketson/dch	LCCarsonl	DCGraves	DLStearns	GEWerner
/RA/	/RA/	/RA/	/RA/	/RA/
1/21/09	1/22/09	1/21/09	1/22/09	1/22/09

B:DRP/	C:PSB2			
VGGaddy	GEWerner			
/RA/	/RA/			
1/22/09	1/23/09			

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**U.S. NUCLEAR REGULATORY COMMISSION
REGION IV**

Dockets: 50-275, 50-323

Licenses: DPR-80, DPR-82

Report: 05000275/2008009
05000323/2008009

Licensee: Pacific Gas and Electric Company

Facility: Diablo Canyon Power Plant, Units 1 and 2

Location: 7 ½ miles NW of Avila Beach
Avila Beach, California

Dates: December 8 - 11, 2008

Inspectors: L. T. Ricketson, P.E., Senior Health Physicist and Team Leader
L. C. Carson II, Senior Health Physicist
D. C. Graves, Health Physicist
D. L. Stearns, Health Physicist

Approved By: Gregory E. Werner, Chief
Plant Support Branch 2
Division of Reactor Safety

SUMMARY OF FINDINGS

IR 05000275/2008009 and 05000323/2008009; 12/08/2008 – 12/11/2008; Diablo Canyon Power Plant Units 1 and 2; Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems

The report covered a one-week period of onsite inspections by a team of four region-based health physics inspectors. Based upon the results of the inspection, the team identified one finding, a Severity Level IV violation. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using Inspection Manual Chapter 0609, "Significance Determination Process." Findings for which the significance determination process does not apply may be Green or be assigned a severity level after NRC management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 4, dated December 2006.

A. NRC-Identified Findings and Self-Revealing Findings

Cornerstone: Public Radiation Safety (PS)

- SLIV. The team identified a non-cited violation of 10 CFR 50.71(e) for the failure of the licensee to periodically (every 24 months) update its Final Safety Analysis Report Update with all changes made in the facility or procedures. Specifically, in July 2005, the licensee stopped using the boric acid evaporator system as described in the Final Safety Analysis Report Update, Section 11.2.6, and did not submit an update to the NRC regarding this operational change. This issue was entered into the licensee's corrective action program as Notification 50116337.

The team determined that the failure to update the Final Safety Analysis Report Update to reflect changes made to the facility was a performance deficiency. This issue is subject to traditional enforcement because it had the potential for impacting the NRC's ability to perform its regulatory function. The finding is characterized as a Severity Level IV, non-cited violation in accordance with NRC Enforcement Policy, Supplement I, Example D.6, in that, the erroneous information in the Final Safety Analysis Report Update was not used to make an unacceptable change to the facility or procedures. (Section 2PS2)

B. Licensee-Identified Violations

None

Report Details

2. RADIATION SAFETY

Cornerstones: Occupational Radiation Safety [OS] and Public Radiation Safety [PS]

2OS3 Radiation Monitoring Instrumentation and Protective Equipment (71121.03)

a. Inspection Scope

This area was inspected to determine the accuracy and operability of radiation monitoring instruments that are used for the protection of occupational workers and the adequacy of the program to provide self-contained breathing apparatus (SCBA) to workers. The team used the requirements in 10 CFR Part 20 and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed:

- Calibration of area radiation monitors associated with transient high and very high radiation areas and post-accident monitors used for remote emergency assessment;
- Calibration of portable radiation detection instrumentation, electronic alarming dosimetry, and continuous air monitors used for job coverage;
- Calibration of whole body counting equipment and radiation detection instruments utilized for personnel and material release from the radiologically controlled area;
- Audits and self-assessments; licensee event reports or special reports, if any were required since the previous inspection;
- Corrective action program reports since the last inspection;
- Licensee action in cases of repetitive deficiencies or significant individual deficiencies;
- Calibration expiration and source response check currency on radiation detection instruments staged for use;
- The licensee's capability for refilling and transporting SCBA air bottles to and from the control room and operations support center during emergency conditions, status of SCBA staged and ready for use in the plant and associated surveillance records, and personnel qualification and training;

- Qualification documentation for onsite personnel designated to perform maintenance on the vendor-designated vital components, and the vital component maintenance records for SCBA units;

Specific documents reviewed during this inspection are listed in the attachment.

These activities constitute completion of nine of the required nine samples, as defined in Inspection Procedure 71121.03-05.

b. Findings

No findings of significance were identified.

2PS1 Radioactive Gaseous And Liquid Effluent Treatment And Monitoring Systems (71122.01)

a. Inspection Scope

This area was inspected to: (1) ensure that the gaseous and liquid effluent processing systems are maintained so that radiological discharges are properly mitigated, monitored, and evaluated with respect to public exposure; (2) ensure that abnormal radioactive gaseous or liquid discharges and conditions, when effluent radiation monitors are out-of-service, are controlled in accordance with the applicable regulatory requirements and licensee procedures; (3) verify that the licensee's quality control program ensures that the radioactive effluent sampling and analysis requirements are satisfied so that discharges of radioactive materials are adequately quantified and evaluated; and (4) verify the adequacy of public dose projections resulting from radioactive effluent discharges. The team used the requirements in 10 CFR Part 20; 10 CFR Part 50, Appendices A and I; 40 CFR Part 190; the Offsite Dose Calculation Manual, and licensee procedures required by the Technical Specifications as criteria for determining compliance.

The team conducted in-office inspection and reviewed:

- Appropriate program documents, procedures and evaluations related to the radiological effluent controls program listed in the attachment to this report;
- The implementation of the Radiological Effluent Controls Program requirements as described in Radiological Effluent Technical Specifications;
- Changes, if any, to the liquid or gaseous radioactive waste system design, procedures, or operation as described in the Final Safety Analysis Report Update;
- Changes, if any, to the Offsite Dose Calculation Manual made by the licensee since the last inspection;

- Effluent monitoring instrumentation documentation to ensure adequate methods and monitoring of effluents;
- The program for identifying, assessing, and controlling contaminated spills and leaks;
- The annual effluent release reports and the correlation to the environmental monitoring results;
- The results from quality assurance audits.

The team conducted an onsite inspection which included interviewing cognizant licensee personnel, performing walkdowns of facilities and equipment, and observing licensee activities to review:

- The gaseous and liquid discharge system configuration;
- Selected point of discharge effluent radiation monitoring systems and flow measurement devices;
- The observation of selected portions of the routine processing and discharge of radioactive gaseous and liquid effluent (sample collection and analysis) including a selection of radioactive gaseous and liquid waste effluent discharge permits;
- Effluent discharges made with inoperable (declared out-of-service) effluent radiation monitors including the projected doses to members of the public;
- Surveillance test results on non-safety related ventilation and gaseous discharge systems (high efficiency particulate air and charcoal filtration) including the methodology to determine the stack and vent flow rates;
- The identification of non-radioactive systems that have become contaminated, if applicable;
- Effluent monitoring instrument (installed and counting room) maintenance, quality control, and calibration;
- The methods used to determine the isotopes in the plant source term, meteorological dispersion and deposition factors, and hydrogeologic characteristics used in the Offsite Dose Calculation Manual and effluent dose calculations including a selection of monthly, quarterly, and annual dose calculations;
- The land-use census;

- Records of abnormal gaseous or liquid discharges, if any, including the evaluation and analysis of events involving spills or discharges, dose assessments to members of the public, required (or voluntary) offsite notifications, and assessments and reporting of abnormal discharges in the Annual Radiological Effluent Release Report;
- Evaluations of discharges from onsite surface water bodies, if any;
- Routine ground water monitoring results;
- Audits and self-assessments; licensee event reports or special reports, if any were required since the previous inspection;
- The results of the inter-laboratory comparison program;
- Effluent sampling records;
- The calibration of post-accident effluent monitoring instrumentation and expected accident source.

The team reviewed the licensee's program of problem identification and resolution, including:

- Placement of problems identified through audits, self assessments, and monitoring results into the corrective action program and adequacy of immediate and long term corrective actions;
- Problem identification and resolution follow-up activities;
- Identification of repetitive deficiencies or significant individual deficiencies in problem identification and resolution identified by the licensee's self-assessment activities.

Specific documents reviewed during this inspection are listed in the attachment.

These activities constitute completion of three of the required three samples, as defined in Inspection Procedure 71122.01-05.

b. Findings

Introduction. The team identified a Severity Level IV non-cited violation of 10 CFR 50.71(e) for the failure of the licensee to periodically (every 24 months) provide the NRC a Final Safety Analysis Report Update with all changes made in the facility or procedures. The finding had very low safety significance.

Description. While conducting a review of the Unit 2 liquid radiological waste system, the team found that the system was not being operated in accordance with the

description provided in the Final Safety Analysis Report Update. Specifically, the boric acid evaporator and concentrate system was not being used as specified in Section 11.2.6 of the Final Safety Analysis Report Update. Section 11.2.6 of the Final Safety Analysis Report Update stated that two-thirds of boric acid evaporator system distillate was recycled to the primary water storage tank. However, during interviews, licensee representatives stated that the boric acid evaporator and concentrate system was taken out of service in July 2005. Approximately 350,000 gallon/year of boric acid distillate from each unit was supposed to be recycled and evaporated, but the team found, based on reviewing the annual radiological effluent release reports, that none of the boric acid system distillate was being recycled and evaporated. Instead, it had been routed directly to the liquid radioactive waste system for processing and discharge as liquid radioactive effluent. Therefore, the team concluded that the licensee had been operating outside of the Final Safety Analysis Report design basis for approximately 3.5 years.

Analysis. The team determined that the failure to update the Final Safety Analysis Report Update to reflect changes made to the facility was a performance deficiency. This issue is subject to traditional enforcement because it had the potential for impacting the NRC's ability to perform its regulatory function. The finding is characterized as a Severity Level IV, non-cited violation because the erroneous information in the Final Safety Analysis Report Update was not used to make an unacceptable change to the facility or procedures, as described in the NRC Enforcement Policy, Supplement I, Example D.6.

Enforcement. Paragraph 50.71(e) of Title 10 requires that the licensee periodically update the final safety analysis report, as provided in subsequent paragraphs. Subparagraph 50.71(e)(4) requires subsequent revisions must be filed annually or 6 months after each refueling outage provided the interval between successive updates does not exceed 24 months. Contrary to the above, in July 2005 the licensee made a change to facility operations and procedures, in that the licensee stopped operating the boric acid evaporator as part of radwaste system operations as specified by Final Safety Analysis Report Update, Section 11.2.6. However, as of the date of the inspection, the licensee had not updated the Final Safety Analysis Report Update to reflect the change for over 40 months. The failure to update the Final Safety Analysis Report Update was characterized as a Severity Level IV non-cited violation. Because this violation was of very low safety significance, was not repetitive or willful, and was entered into the licensee's corrective action program as Notification 50116337, this violation is being treated as a non-cited violation, consistent with Section VI.A.1 of the NRC Enforcement Policy: NCV 05000275/2008009-01 and 05000323/2008009-01: Failure to Update the Final Safety Analysis Report.

2PS2 Radioactive Material Processing and Transportation (71122.02)

a. Inspection Scope

This area was inspected to verify that the licensee's radioactive material processing and transportation program complies with the requirements of 10 CFR Parts 20, 61, and 71

and Department of Transportation regulations contained in 49 CFR Parts 171-180. The team interviewed licensee personnel and reviewed:

- The radioactive waste system description, recent radiological effluent release reports, and the scope of the licensee's audit program;
- Liquid and solid radioactive waste processing systems configurations, the status and control of any radioactive waste process equipment that is not operational or is abandoned in place, changes made to the radioactive waste processing systems since the last inspection, and current processes for transferring radioactive waste resin and sludge discharges;
- Radio-chemical sample analysis results for radioactive waste streams and use of scaling factors and calculations to account for difficult-to-measure radionuclides;
- Shipment packaging, surveying, labeling, marking, placarding, vehicle checking, driver instructing, and disposal manifesting;
- Shipping records for non-excepted package shipments;
- Audits and self-assessments; licensee event reports, special reports, audits, and state agency reports, if any were required since the previous inspection.

Specific documents reviewed during this inspection are listed in the attachment.

These activities constitute completion of six of the required six samples, as defined in Inspection Procedure 71122.02-05.

b. Findings

No findings of significance were identified.

2PS3 Radiological Environmental Monitoring Program and Radioactive Material Control Program (71122.03)

a. Inspection Scope

This area was inspected to ensure that the radiological environmental monitoring program verifies the impact of radioactive effluent releases to the environment and sufficiently validates the integrity of the radioactive gaseous and liquid effluent release program; and that the licensee's surveys and controls are adequate to prevent the inadvertent release of licensed materials into the public domain. The team used the requirements in 10 CFR Part 20, Appendix I of 10 CFR Part 50, the Offsite Dose Calculation Manual, and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed:

- Annual environmental monitoring reports and licensee event reports, if any were required since the previous inspection;
- Selected air sampling and thermoluminescence dosimeter monitoring stations;
- Collection and preparation of environmental samples;
- Operability, calibration, and maintenance of meteorological instruments;
- Each event, if any, documented in the Annual Environmental Monitoring Report which involved a missed sample, inoperable sampler, lost thermoluminescence dosimeter, or anomalous measurement;
- Significant changes, if any, made by the licensee to the Offsite Dose Calculation Manual as the result of changes to the land census or sampler station modifications since the last inspection;
- Calibration and maintenance records for air samplers, composite water samplers, and environmental sample radiation measurement instrumentation, quality control program, interlaboratory comparison program results, and vendor audits;
- Locations where the licensee monitors potentially contaminated material leaving the radiological controlled area [or controlled access area] and the methods used for control, survey, and release from these areas;
- Type of radiation monitoring instrumentation used to monitor items released, survey and release criteria of potentially contaminated material, radiation detection sensitivities, procedural guidance, and material release records;
- Audits, self-assessments, corrective action documents and licensee event reports or special reports, if any were required, since the previous inspection.

Specific documents reviewed during this inspection are listed in the attachment.

These activities constitute completion of ten of the required ten samples, as defined in Inspection Procedure 71122.03-05.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES

4OA2 Problem Identification and Resolution

Annual Sample Review

a. Inspection Scope

The team evaluated the effectiveness of the licensee's problem identification and resolution process with respect to the following inspection areas:

- Radiation Monitoring Instrumentation; (Section 2OS3)
- Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems; (Section 2PS1)
- Radioactive Material Processing and Transportation; (Section 2PS2)
- Radiological Environmental Monitoring Program and Radioactive Material Control Program. (Section 2PS3)

b. Findings and Observations

No findings of significance were identified.

4OA5 Other Activities

.1 (Closed) Temporary Instruction (TI) 2515/173, "Review of the Implementation of the Industry Ground Water Protection Voluntary Initiative"

a. Inspection Scope

An NRC assessment was performed of the licensee's ground water protection program to determine whether the licensee implemented the voluntary Industry Ground Water Protection Initiative, dated August 2007 (Nuclear Energy Institute 07-07, ADAMS accession number ML072610036). Inspectors interviewed personnel, performed walk-downs of selected areas, and reviewed the following items:

- Records of the site characterization of geology and hydrology;
- Evaluations of systems, structures, and or components that contain or could contain licensed material and evaluations of work practices that involve licensed material for which there is a credible mechanism for the licensed material to reach the ground water;
- Implementation of an onsite ground water monitoring program to monitor for potential licensed radioactive leakage into ground water;
- Procedures for the decision making process for potential remediation of leaks and spills, including consideration of the long term decommissioning impacts;
- Records of leaks and spills recorded, if any, in the licensee's decommissioning files in accordance with 10 CFR 50.75(g);

- Licensee briefings of local and state officials on the licensee's ground water protection initiative;
- Protocols for notification to the local and state officials, and to the NRC regarding detection of leaks and spills;
- Protocols and/or procedures for thirty day reports if an onsite ground water sample exceeds the criteria in the radiological environmental monitoring program;
- Ground water monitoring results as reported in the annual effluent and/or environmental monitoring report;
- Licensee and industry assessments of implementation of the ground water protection initiative.

b. Findings

No findings of significance were identified. Implementation of the Industry Ground Water Protection Initiative is voluntary. Under the final Ground Water Protection Initiative, each site was to have developed an effective, technically sound ground water protection program by August 2008. The team found the licensee had not completed its evaluation of all systems, structures, and components that contain licensed radioactive material to determine potential leak or spill mechanisms. The licensee had not prepared procedures for the decision making process for potential remediation of leaks and spills. The licensee had not established protocols for notification to local and state officials and to the NRC regarding detection of leaks and spills. The licensee identified these actions had not been completed in a self-assessment conducted in October 2008. At the time of the inspection, the licensee still had not completed the actions but the findings of the self-assessment were entered into the corrective action program.

40A6 Management Meetings

Exit Meeting Summary

On December 11, 2008, the team presented the inspection results to Mr. K. Peters, Station Director, and other members of his staff who acknowledged the findings. The team confirmed that proprietary information was not provided or examined during the inspection.

ATTACHMENT: SUPPLEMENTAL INFORMATION

SUPPLEMENTAL INFORMATION

KEY POINTS OF CONTACT

Licensee Personnel

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S. Sabharwal, System Engineer, Gaseous Radwaste
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S. Zawalick, Senior NRC Interface, Regulatory Services

NRC

M. Brown, Resident Inspector
G. Werner, Chief, Plant Support Branch 2

LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

Opened and Closed During this Inspection

05000275/2008009-01; 05000323/2008009-01	NCV	Failure to update the Final Safety Analysis Report (Section 2PS1)
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Previous Items Closed

NONE

Previous Items Discussed

NONE

LIST OF DOCUMENTS REVIEWED

Section 20S3: Radiation Monitoring Instrumentation and Protective Equipment (IP71121.03)

Procedures

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
RCP D-353	Operation of the Canberra Fastscan Whole Body Counters	12
RCP D-900	Performance Tests for Radiation Protection Instruments	39
RCP D-970	Radiation Control Instrumentation Calibration Schedule	16
RCP D-981	Operation and Calibration of the 400 Ci J.L. Shepherd Model 89 Shielded Calibrator	2
MP I-RC24	NE Technology SAM-9 Tool Contamination Monitor Calibration	2
MP I-RC26	Calibration of Eberline PM7 Portal Radiation Monitor	3
MP I-RC28	Calibration of Ludlum 177 Count Rate Meter	0
MP I-RD01	Calibration of Eberline Model RO-2 and RO-2A Ion Chamber Survey Instruments	5
MP I-RD03	Calibration of Eberline 6112B Teletector G-M Survey Instrument	10
MP I-RD33	Calibration of MGP AMP-100 Dose Rate Meter	5

Work Orders

<u>NUMBER</u>	<u>TITLE</u>
R0320294 01	Functional Test of Containment High Range Area Monitor
R0304155 01	Calibration of Plant Vent Radiation Monitor RM-24
R0304159 01	Calibration of Plant Vent Radiation Monitor RM-14

Miscellaneous Documents

<u>TITLE</u>	<u>DATE</u>
Calibration of Canberra GEM-5 #97-9321	7/31/2008
Calibration of Canberra ARGOS #97-9313	9/09/2008
Calibration Verification of the ABACOS-2000 Fastscan Counting System #97-5119	7/14/2008
Calibration Verification of the ABACOS-2000 Fastscan Counting System #97-2391	7/15/2008
Environmental Radiation Monitor Calibration Certificate, 04D85001	10/13/2008
Radiation Monitoring System Health Report for Third Quarter 2008	

Corrective Action Document

A0689149	A0693329	A0694783	A0694895	A0701416	A0703316
A0703360	A0705264	A0707196	A0708103	A0710421	A0721755
A0722696	A0731781	A0733881	A0735518	A0735955	A0735956
A0736426	A0737236	A0737606	A0737956	A0740566	A0740655
A0740703	50037753	50039662	50040908	50041248	50044333
50082320	50083118	50083119	50083274	50085573	

Section 2PS1: Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems (IP71122.01)

Procedures

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
CAP A-11	Liquid Radwaste Processing System Process Selection	8
CAP A-5	Liquid Radwaste Discharge Management	41A
CAP A-6	Gaseous Radwaste Discharge Management	29
CAP A-8	Off-Site Dose Calculations,	31
CAP D-11	Tritium in Liquid Samples	9
CY2	Radiological Monitoring and Controls Program	2
CY2.ID1	Radioactive Effluent Controls Program	8
RCP EM-5D	DCPP Ground Water Sampling	1
STP G-9	General HEPA Filter Bank Penetration Test	8
STP G-10	General Charcoal Filter Bank Penetration Test	8
STP G-11	Procedure for Obtaining Charcoal Filter Media for Laboratory Testing (Methyl Iodine)	16A
STP M-3A	Auxiliary Building Ventilation System - DOP and Halide Penetration Test	13B
STP M-53	Control Room Ventilation System - DOP and Halide Penetration Test	14B
STP I-19	LRW Line Flow Channel FIT-243 Calibration	6
STP I-39	Plant Vent Flow Measurement Calibration	10

Release Permits

Liquid

Unit-2 Liquid Radwaste Batch, 12/09/08

Unit-1 High Conductivity Tank, 12/09/08

Gaseous

Unit-2 Plant Vent, 12/09/08
Unit-2 Containment Atmosphere, 12/10/08
Unit-1 Containment Atmosphere, 12/10/08

In-Place Filter Testing Surveillances

Unit-1 Control Room Ventilation HEPA Filter In-Place Leak Test, 10/10/06
Unit-1 Auxiliary Ventilation HEPA Filter In-Place Leak Test, 10/10/06
Unit-1 Auxiliary Ventilation HEPA Filter In-Place Leak Test, 01/28/08

Audits and Assessments

2007 Radiation Protection Program Audit, 08/09/07
Quality Performance Assessment Report, 2nd Period 2007
Quality Performance Assessment Report, 1st Period 2008
2008 Radioactive Effluents Control Program, Off-Site Dose Calculation Manual, and
Radioactive Environmental Monitoring Program Audit, 02/22/08
2008 Radiation Protection Program, and Solid Radioactive Waste Management (Process
Control) and Transport Program, 07/17/08

Corrective Action Documents

A0538530	A0568568	A0602113	A0636102	A0636111	A0636380
A0637725	A0647992	A0657423	A0658599	A0658602	A0661736
A0661892	A0671312	A0671526	A0671647	A0671649	A0671526
A0672688	A0672790	A0673112	A0673734	A0673851	A0672688
A0675430	A0674479	A0678765	A0679137	A0679478	A0680589
A0680600	A0683712	A0687728	A0688249	A0693049	A0699529
A0708356	A0719510	A0705928	A0708356	A0710226	
50034248	50035253	50036595	50037248	50038341	50040219
50040286	50041035	50043651	50087079	60009023	

Miscellaneous

2006 and 2007 Annual Radiological Effluent Release Reports
Inter-laboratory Comparison Program 2007
Results of the Radiochemistry Cross Check Program 2007

Section 2PS2: Radioactive Material Processing and Transportation (IP71122.02)

Audits and Assessments

2007 Radiation Protection Program Audit

2008 Radiation Protection Program, Solid Radioactive Waste Management Control, and Transportation Program

Quality Performance Assessment Report – Second Period

Procedures

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
RCP D-631	Radioactive Material Shipments	8
RCP RW-3	Radioactive Waste Nuclide Fractions and Correlation Factors Determination	18
RCP RW-4	Solid Radioactive Waste Shipments	28
RCP RW-5	Receiving, Loading, and Releasing of Transport Vehicle for Radioactive Waste Shipment	14

Shipping Packages

RMS-07-006 RMS-07-053 RMS-07-056 RMS-07-062 RMS-07-063 RMS-07-067
RMS-08-024 RMS-08-030 RMS-08-032 RMS-08-035 RMS-08-036 RMS-08-059
RMS-08-120 RWS-07-001 RWS-07-005 RWS-08-001 RWS-08-002 RWS-08-003

Corrective Action Documents

A0732260 A0732728 A0692530 A0710226 A0732278 A0687253
A0720057 A0711318 A0701774 A0703362

Section 2PS3: Radiological Environmental Monitoring Program and Radioactive Material Control Program (IP71122.03)

Audits and Assessments

2007 Radiation Protection Program Audit

2008 Radioactive Effluent Control Program, Offsite-Dose Calculation Manual, and Radioactive Environmental Monitoring Program Audit

Procedures

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
CY2	Radiological Monitoring and Controls Program	6
CY2ID11	Environmental Radiological Monitoring Procedure	8
CAP A-8	Off Site Dose Calculations	31
RCP EM-1	Radiological Environmental Biological Sampling	9
RCP EM-2	Radiological Environmental Air Sampling	12
RCP EM-4	Area TLD Monitoring	2
RCP EM-5	DCPP Ground Water Sampling	2
STP I-40-M559.B	Primary Met Instrument Channels	21A
STP I-40-M569.B	Backup Met Instrument Channels	8
RCP D-614,	Release of Solid Materials from Radiologically Controlled Areas	14

Meteorological Instrument Calibration

<u>NUMBER</u>	<u>TITLE</u>	<u>DATE</u>
R0306850	Calibration of Backup Meteorological Instrumentation	1-24-08
R0310511	Calibration of Primary Meteorological Instrumentation	4-24-08

Miscellaneous Documents

2006 Annual Environmental Operating Report

2007 Annual Environmental Operating Report

Fourth Quarter 2007 Air Sampler Calibration/Function Test Record (10/03/07)

Fourth Quarter 2008 Air Sampler Calibration/Function Test Record (10/22/08)

Corrective Action Documents

A0696953	A0717940	A0726562	A0735113	A0737235	A0741456
A0700022	A0719329	A0732643	A0735567	A0737959	A0741786
A0703362	A0721867	A0734529	A0736787	A0738519	

Section 40A5 Temporary Instruction (TI) 2515/173

Miscellaneous Documents

Sampling and Analysis of Various Tritium Sampling at DCPD Due to the NEI 8/25/08
Ground Water Protection Initiative

Tritium Occurrence in Ground Water at Diablo Canyon Power Plant by S.M. Stoller
Corporation

Corrective Action Documents

A0538530	A0568568	A0602113	A0636102	A0647992	A0657423
A0658599	A0658602	A0670660	A0673871	A0674479	A0708346
A0708356					
50035253	50037248	50037249	50040219	50034248	50116884
5003659					