Modeling of the Dynamics of the Nucleation and Growth of Cracks Initiated due to Deformation in Materials

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Similarly If the Damage is Due to Corrosion

→ Why > Corrosion

→ How > Damage due to

- Pits
 Coalescence of pits under Stress
 - Exfoliation of layers
 - Selective Leaching of One Species can Introduce Surface Stress Build up

Material Science Logic



Radiation Induced Materials Degradation



Build up of Stresses within the Irradiated materials

Mono crystalline Nickel (110) Irradiated with Helium Ions (100 keV, Dose 1 C/cm²)



Defect Structure

Blisters

Bacon and Rao, J. Nucl. Mater., 91(10, 178 (1980)



Helium bubbles in Mono crystalline Nickel (110) Irradiated with 100 keV ions

Corrosion Induced Materials Degradation



Aqueous Corrosion



5054 Al Alloy Sensitized at 80C for 1 day



5054 Al alloy Sensitized at 80C for 29 days



5054 Al alloy Sensitized at 80C for 29 days



5054 Al alloy Sensitized at 80C for 74 days

Corrosion Induced Materials Degradation



If Modeling Can Predict

Such Information will be Valuable for Practical Application









5054 Aluminum Sample Sensitized for 74 Days Accelerated Corrosion 1 Day



5054 Aluminum Sample Sensitized for 74 Days Accelerated Corrosion 1 Day

Pidaparti and Rao, Corrosion Science, 50[7], 1932 (2008)



5054 Aluminum alloy sensitized at 80°C for 74 days and was subjected to accelerated corrosion for 1 day.





Pits in 5054 series aluminum alloys sensitized at 80°C for 74 days and subjected to accelerated corrosion for 1 day.

Von Mises Stress Contours

SEM of Corroded Surface

.235E+07

.302E+07

(pa)



Pits in 5054 series aluminum alloys sensitized at 80°C for 74 days and subjected to accelerated corrosion for 1 day.

.168E+07

Pidaparti and Rao, J. of Computational Mechanics., (In Print)

340594

.101E+07

Conclusion of the FEA Analysis Results



General Conclusion from FEA Analysis



Modeling Based on Numerical Analysis of Stresses





Required Inputs for Numerical Analysis

➔ Defect Size (i.e. Average Pit Diameter) and Density (Number of Pits & Separation Distance)

→ Boundary Conditions → Limits of the Stress (Effective Zones) Lower Limit → Sample Length (L) and Width (W) Upper Limit → Sample Length (L) and Half the Sample Thickness (t/2)



Rao, Accepted for Publication in Materials Science Transactions

Lower Boundary – Critical Along the Sample Surface Area



Upper Boundary – Critical Along the Sample Thickness





Inference from the Material Integrity Plot $\ln \{\ln(1/\rho_s)\}$ versus $\ln(\sigma)$

Deviation from → Linear Behavior of
→ In {In(1/p_s)} vs In{(σ)}
◆ Significant Change → in the Stress pattern
→ (i.e. Change in Structure
→ May be Cracking
→ May be Gross Deformation (Plastic Collapse)



SEM Obtained from Aluminum Sample with Defects

Equal partitioning of observed area with Defects



SEM Obtained from Aluminum Sample with Defects

Equal partitioning of observed area with Defects

Integrity of Nickel Aluminum Bronze (NAB) Surface Layers During Corrosion: Results Obtained from the Present Numerical Analysis Based Model



•The NiAl precipitates have some Fe and Cu in them and the Fe₃Al have some Ni and Cu in them and everything has Mn in it.

•NiAl has two primary morphologies, spherical, often with a Fe₃Al center, and lamellar, outlining the prior beta grains.

•Fe₃Al is usually cuboidal or cruciform but may be dendritic.

Typical microstructure of as-received nickel aluminum bronze (NAB) sample. As received NAB is rolled, creep formed and in service material







The probability of failure of corroded NAB sample surface layers under applied external stress. NAB samples were subjected to corrosion in 10%amminia – 90% seawater under stress for 2 days.

Applied Stress (ksi)	Average Defect Diameter (d) (µm)	Average Defect - Defect Separation Distance(S) (µm)	Depth of the Defects (d $_{t}$) (μ m)			
			Probability of Failure Corroded Surface (P _f) (%)			
			Lower	Upper	Lower	Upper
			Bound	Bound	Bound	Bound
			~ (d/2)		~ (d/10)	
0	1.5	2.4	3	27	1	6
7.5	1.76	1.67	5	32	2	8
15	2.6	0.92	15	58	8	15



Determination by Numerical Methods



Schematic Diagram of the Possible Failure Mode Due to Defect Coalescence in Materials Under Stress





NAB under 45 ksi stress in 10% Ammonia and 90% seawater – After 35 Days



Resolved Stress Direction Normal to the Applied Stress



Relevance of Present Model for Nuclear Applications

Radiation Introduces Stresses due to

- → Mechanical Deformation (eg. Twinning in Graphite)
- → Lattice Parameter and Dimensional Changes
- ➔ Void Swelling
- → Gas Bubbles
- Blisters

One can Adopt the Present Modeling Approach to

- Estimate the Stresses Introduced Due to the Irradiation Induced Defects and
- → Make Predictions on the Probability of Failure of the Material Integrity Under Continued Irradiation.
- Propose the Possible Failure Mode (viz. Exfoliation or Coalescence of Blisters etc.) Based Local Strain Estimation

Mechanism of Radiation Blistering in Materials



 $P_{bubble} < 2\gamma / r_{bubble}$ $P_{bubble} > 2\gamma / r_{bubble}$

Excess Pressure (ΔP) = P _{bubble} - 2 γ / r _{bubble}

Total Pressure > Yield Stress Blister will Form



RT

300°C



Inconel –625 irradiated with 100 keV Helium Ions to a Dose of 10¹⁸ ions/cm² (a) at room temperature and (c) at 300°C.



Mono crystalline Nickel (110) Irradiated with Helium Ions (100 keV, Dose 1 C/cm²)

Titanium Carbide (Deuterium ions, Dose 5 X 10¹⁸ ions/cm²)



Rao and Kaminsky, Thin Solid Films, 83(1), 93 (1981)

Conclusion

- New Generalized Model to Estimate the Probability of Survival a Material is Subjected to Dynamic Stress and a Degrading Environment has been Proposed.
- New Methodology to Determine the Dynamic Critical Stress Required to Estimate the Failure of Materials Under Applied Stress.

A New Approach to Determine the Dynamic Stress Required for Coalescence of Defects has been Proposed.

Relevance of NRC Activity

- This generalized model can be used tailored to estimate the Probability of a Material Integrity under defined Radiation Induced / Assisted Damage.
- This methodology to determine the "local" dynamic critical stress due to the presence of voids can provide valuable information on the degradation due to IASCC in reactor internals.
- If we know the rate of neutron dose that is received by a reactor internal during normal operation, it is possible to determine the onset of neutron dose (saturation limit) for Coalescence of Defects (from the estimated local dynamic Stress).