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Nuclear

10 CFR 72.140(d)

RS-09-012 January 21, 2009

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555-0001

Braidwood Station, Units 1 and 2
Facility Operating License Nos. NPF-72 and NPF-77

NRC Docket Nos. STN 50-456 and STN 50-457

Subject:

Notification Pursuant to 10 CFR 72.140(d) of Intent to Apply Previously Approved 10 CFR 50, Appendix B, Quality Assurance Program to Independent Spent Fuel

Storage Installation Activities

References:

- Letter from P. B. Cowan (Exelon Generation Company, LLC) to U. S. NRC, "Submittal of Exelon Generation Company, LLC and AmerGen Energy Company, LLC Quality Assurance Topical Report, NO-AA-10, Revision 80," dated February 6, 2008
- 2) Letter from M. P. Gallagher (Exelon Generation Company, LLC) to U. S. NRC, "Request for Approval of Quality Assurance Program Changes," dated April 9, 2002
- 3) Letter from U. S. NRC to J. L. Skolds (Exelon Generation Company, LLC), "Approval of Proposed Revision 70 of Quality Assurance Topical Report EGC-1A, Rev. 70 in accordance with 10 CFR 50.54(a) Requirements for Exelon/AmerGen Plants," dated December 24, 2002

In accordance with 10 CFR 72.140(d), "Previously-approved programs," Exelon Generation Company, LLC (EGC) is notifying the NRC of our intent to apply the previously approved 10 CFR 50, Appendix B, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Processing Plants," quality assurance program to independent spent fuel storage installation (ISFSI) and spent fuel storage cask activities at Braidwood Station, Units 1 and 2. The ISFSI structures, systems, and components that are important to safety will be categorized in accordance with NUREG/CR-6407, "Classification of Transportation Packaging and Dry Spent Fuel Storage System Components According to Importance to Safety."

The Braidwood Station Quality Assurance Program, which satisfies the criteria of 10 CFR 50, Appendix B, is maintained in the Quality Assurance Topical Report (QATR) (NO-AA-10), Revision 82. Revision 80 was submitted to the NRC in Reference 1. Reference 2 includes the previously approved quality assurance program submittal date and Reference 3 includes the most recent date of formal NRC quality assurance program approval for the docket numbers listed above. QATR, Revision 81, was processed in accordance with 10 CFR 50.54(a)(3) to document that the 10 CFR 50, Appendix B program applies to Braidwood ISFSI and spent fuel

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cask activities consistent with the activity's importance to safety as defined in NUREG/CR-6407. Revision 81 became effective on June 16, 2008.

There are no commitments contained with this letter. Should you have any questions, please contact Ms. Michelle Yun at (630) 657-2818.

Respectfully /

Jeffrey/L/ Hansen Manager – Licensing