



DEC 31 2008

10CFR50.73

LR-N08- 0296

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington DC 20555-001

Salem Nuclear Generating Station Unit 1
Facility Operating License No. DPR-70
NRC Docket No. 50-272

SUBJECT: Inadvertent Start of an Emergency Diesel Generator During Testing

This Licensee Event Report (LER) number 272/2008-001 "Inadvertent Start of an Emergency Diesel Generator During Testing" is being submitted pursuant to the requirements of the Code of Federal Regulations 10CFR50.73(a)(2)(iv)(A).

The attached LER contains no commitments. Should you have any questions or comments regarding this submittal, please contact Mr. E. H. Villar at 856-339-5456.

Sincerely,

A handwritten signature in black ink, appearing to be "R. Braun", with a long horizontal flourish extending to the right.

Robert Braun
Site Vice President - Salem

Attachments (1)

JE22
NRR

Document Control Desk

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cc Mr. S. Collins, Administrator - Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. R. Ennis, Licensing Project Manager - Salem
U. S. Nuclear Regulatory Commission
Mail Stop 08B1
Washington, DC 20555-0001

USNRC Senior Resident Inspector - Salem (X24)

Mr. P. Mulligan, Manager IV
Bureau of Nuclear Engineering
PO Box 415
Trenton, NJ 08625

LICENSEE EVENT REPORT (LER)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME Salem Generating Station - Unit 1	2. DOCKET NUMBER 05000272	3. PAGE 1 of 4
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4. TITLE Inadvertent Start of an Emergency Diesel Generator During Testing

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
11	05	2008	2008	0 0 1 0		12	31	2008		DOCKET NUMBER

9. OPERATING MODE 5	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR§: (Check all that apply) <table style="width:100%; border: none;"> <tr> <td><input type="checkbox"/> 20.2201(b)</td> <td><input type="checkbox"/> 20.2203(a)(3)(i)</td> <td><input type="checkbox"/> 50.73(a)(2)(i)(C)</td> <td><input type="checkbox"/> 50.73(a)(2)(vii)</td> </tr> <tr> <td><input type="checkbox"/> 20.2201(d)</td> <td><input type="checkbox"/> 20.2203(a)(3)(ii)</td> <td><input type="checkbox"/> 50.73(a)(2)(ii)(A)</td> <td><input type="checkbox"/> 50.73(a)(2)(viii)(A)</td> </tr> <tr> <td><input type="checkbox"/> 20.2203(a)(1)</td> <td><input type="checkbox"/> 20.2203(a)(4)</td> <td><input type="checkbox"/> 50.73(a)(2)(ii)(B)</td> <td><input type="checkbox"/> 50.73(a)(2)(viii)(B)</td> </tr> <tr> <td><input type="checkbox"/> 20.2203(a)(2)(i)</td> <td><input type="checkbox"/> 50.36(c)(1)(i)(A)</td> <td><input type="checkbox"/> 50.73(a)(2)(iii)</td> <td><input type="checkbox"/> 50.73(a)(2)(ix)(A)</td> </tr> <tr> <td><input type="checkbox"/> 20.2203(a)(2)(ii)</td> <td><input type="checkbox"/> 50.36(c)(1)(ii)(A)</td> <td><input checked="" type="checkbox"/> 50.73(a)(2)(iv)(A)</td> <td><input type="checkbox"/> 50.73(a)(2)(x)</td> </tr> <tr> <td><input type="checkbox"/> 20.2203(a)(2)(iii)</td> <td><input type="checkbox"/> 50.36(c)(2)</td> <td><input type="checkbox"/> 50.73(a)(2)(v)(A)</td> <td><input type="checkbox"/> 73.71(a)(4)</td> </tr> <tr> <td><input type="checkbox"/> 20.2203(a)(2)(iv)</td> <td><input type="checkbox"/> 50.46(a)(3)(ii)</td> <td><input type="checkbox"/> 50.73(a)(2)(v)(B)</td> <td><input type="checkbox"/> 73.71(a)(5)</td> </tr> <tr> <td><input type="checkbox"/> 20.2203(a)(2)(v)</td> <td><input type="checkbox"/> 50.73(a)(2)(i)(A)</td> <td><input type="checkbox"/> 50.73(a)(2)(v)(C)</td> <td><input type="checkbox"/> OTHER</td> </tr> <tr> <td><input type="checkbox"/> 20.2203(a)(2)(vi)</td> <td><input type="checkbox"/> 50.73(a)(2)(i)(B)</td> <td><input type="checkbox"/> 50.73(a)(2)(v)(D)</td> <td>Specify in Abstract below or in NRC Form 366A</td> </tr> </table>	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)	<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER	<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A
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12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME Enrique Villar, Regulatory Assurance Engineer	TELEPHONE NUMBER (Include Area Code) (856) 339 -5456
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13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX
A	-	-	-	-					

14. SUPPLEMENTAL REPORT EXPECTED <input type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO
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15. EXPECTED SUBMISSION DATE

MONTH	DAY	YEAR
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ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On November 5, 2008, Salem unit 1 was shutdown in Mode 5 completing its nineteenth refueling outage (1R19) activities. During the performance of an overlap test to complete the Technical Specification 4.8.1.1.2 required Mode Operation surveillance test for the 11 containment fan coil unit, the 1A vital bus unexpectedly de-energized and the 1A safeguards equipment controller actuated in the blackout mode starting the 1A emergency diesel generator.

The apparent cause of the event was a technical rigor / human performance error made in the test plan development that was not identified and corrected in the subsequent reviews. Corrective actions taken included: an extent of condition review to identify similar test plan errors, no additional errors were identified; shared lessons learned with the plant engineering department and station personnel. Additionally the Mode Operations surveillance test procedures will be revised to include specific requirements for the use of alternate test methodologies in the event a component(s) is not available for Mode Operations testing or fails to properly actuate during the test.

This report is being made in accordance with 10CFR50.73(a)(2)(iv)(A), "any event or condition that resulted in manual or automatic actuation of any of the systems listed in paragraph (a)(2)(iv)(A)...."

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NARRATIVE

PLANT AND SYSTEM IDENTIFICATION

Westinghouse – Pressurized Water Reactor (PWR/4)

Containment Fan Coil Unit (CFCU) {BK},
Emergency Diesel Generator (EDG) {EK}.

* Energy Industry Identification System {EIS} codes and component function identifier codes appear as {SS/CCC}

IDENTIFICATION OF OCCURRENCE

Event Date: November 05, 2008

Discovery Date: November 05, 2008

CONDITIONS PRIOR TO OCCURRENCE

Salem Unit 1 was in Operational Mode 5 at 0% reactor power.

No additional structures, systems or components were inoperable at the time that contributed to the event.

DESCRIPTION OF OCCURRENCE

On November 5, 2008, Salem unit 1 was shutdown in Mode 5 completing its nineteenth refueling outage (1R19) activities. During the performance of an overlap test to complete the Technical Specification 4.8.1.1.2 Mode Operation surveillance test for the 11 containment fan coil unit (CFCU) {BK}, the 1A vital bus unexpectedly de-energized and the 1A safeguards equipment controller (SEC) actuated in the blackout mode starting the 1A emergency diesel generator (EDG) {EK}.

All equipment functioned as designed and control room personnel responded appropriately and in accordance with procedures and policies. Once the operating personnel determined that the actuation was unwarranted, all unnecessary actuated components were stopped, and the 1A vital bus was restored to its normal lineup. The 1A EDG was stopped and placed in its normal standby alignment.

This report is being made in accordance with 10CFR50.73(a)(2)(iv)(A), "any event or condition that resulted in manual or automatic actuation of any of the systems listed in paragraph (a)(2)(iv)(A)..."

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CAUSE OF OCCURRENCE

The apparent cause of the event was a technical rigor / human performance error made in the test plan development that was not identified or corrected in the subsequent reviews. The engineer who developed the test plan and the engineer who performed the independent review of the test plan failed to identify that depressing the B1 test button on the 1A SEC would actuate backup trips on the 1A 4KV vital bus infeed breakers, which would de-energize the bus and start the EDG.

PREVIOUS OCCURRENCES

Salem Generating Station LERs for years 2008 back to 2005 were reviewed for similar occurrences of an inadvertent Engineered Safeguard Feature (ESF) actuation during testing. The following two (2) LERs were identified:

- LER 272/2007-001 "ESF Actuation of Auxiliary Feedwater Pumps in Mode 3"
- LER 311/2006-002 "Automatic Start of Auxiliary Feedwater Pumps in Mode 4"

The apparent causes and corrective actions associated with these LERs were different and specific to these events and they would have not prevented this occurrence.

SAFETY CONSEQUENCES AND IMPLICATIONS

There were no safety consequences associated with this event. All equipment functioned as designed. Once the operating personnel determined that the actuation was unwarranted, all unnecessary actuated components were stopped and the plant restored to its normal configuration.

A review of this event determined that a Safety System Functional Failure (SSFF) as defined in NEI 99-02, Regulatory Assessment Performance Indicator Guidelines, did not occur. This event did not prevent the ability of a system to fulfill its safety function to either shutdown the reactor, remove residual heat, control the release of radioactive material, or mitigate the consequences of an accident.

CORRECTIVE ACTIONS

1. An extent of condition review was performed to identify similar test plan errors; no additional errors were identified.
2. The lessons learned were shared with the plant engineering department and station personnel.
3. The Mode Operations surveillance test procedures will be revised to include specific requirements for the use of alternate test methodologies in the event a component(s) is not available for Mode Operations testing or fails to properly actuate during the test. The methodologies will be subject to the same review and approval process rigor as the original surveillance test procedure.

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NARRATIVE

COMMITMENTS

No commitments are made in this LER.