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U S Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Prairie Island Nuclear Generating Plant Unit 2
Docket 50-306
License No. DPR-60

60-Day Report Pursuant to EA-03-009 Paragraph E for 2008 Prairie Island Nuclear
Generating Plant Unit 2 Reactor Pressure Vessel (RPV) Head Inspection

- Reference:
- 1) Letter from NRC to Specified Pressurized Water Reactor Licensees, "Issuance of First Revised NRC Order (EA-03-009) Establishing Interim Inspection Requirements for Reactor Pressure Vessel Heads at Pressurized Water Reactors," dated February 20, 2004 (ADAMS Accession Number ML040220181).
 - 2) Letter from NMC to NRC, "Response to Revised Order EA-03-009, 'Issuance of First Revised NRC Order (EA-03-009) Establishing Interim Inspection Requirements for Reactor Pressure Vessel Heads at Pressurized Water Reactors'," dated March 8, 2004 (ADAMS Accession Number ML041610280).

The Nuclear Regulatory Commission (NRC) issued First Revised Order EA-03-009 on February 20, 2004 (Reference 1). Nuclear Management Company, LLC, consented to the Order as written on March 8, 2004 (Reference 2).

Consistent with the requirements of the Order, the Unit 2 RPV head was not inspected during the 2008 Unit 2 refueling outage (2R25). The RPV head was replaced in 2005.

Paragraph D of the Order requires visual inspection each refueling outage to identify potential boric acid leaks from pressure-retaining components above the RPV head. Paragraph E of the Order requires a report detailing inspection results within 60 days after returning the plant to operation for each inspection required in Paragraph D of the Order if a leak or boron deposit was found during the inspection. In response to this requirement, Northern States Power Company, a Minnesota corporation (NSPM) notes the following with respect to the reactor pressure vessel head inspection conducted during 2R25:

Inspection of components above the Unit 2 RPV head was performed during 2R25. The inspection was performed by qualified VT-2 examiners who identified two minor boric acid indications. No boric acid was in contact with the vessel head.

The first component with an indication was valve 2RC-8-5, a reactor vessel head vent valve. A trace of white boric acid film was found between the stem and the gland follower from a prior packing leak. The indication was confined to the valve and boric acid had not come into contact with the RPV bare metal. The indication was cleaned and the location re-inspected while at normal operating pressure with no leakage observed.

The second component with an indication was the core exit thermocouple nozzle assembly (CETNA). A trace of boric acid flakes had fallen down the thermocouple tubing sleeve and was also seen in the area between the compression collar and CETNA column. Boroscopic examination of the connectors underneath the sleeves revealed no leakage. The component had no indication of degradation due to exposure to boric acid. All three CETNAs were disassembled, cleaned and reassembled during 2R25. Subsequent re-inspection of the CETNAs at normal operating pressure identified no observed leakage.

Summary of Commitments

This letter contains no new commitments and no revisions to existing commitments.

I declare under penalty of perjury that the foregoing is true and correct. Executed on



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Michael D. Wadley
Site Vice President
Prairie Island Nuclear Generating Plant
Northern States Power Company - Minnesota

cc: Administrator, Region III, USNRC
Project Manager, Prairie Island, USNRC
Resident Inspector, Prairie Island, USNRC