



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
 WASHINGTON, D.C. 20555-0001

June 29, 2000

MEMORANDUM TO: ACRS Members
Noel Dudley
 FROM: Noel Dudley, Senior Staff Engineer
 SUBJECT: NRC SUMMARY OF MEETING BETWEEN WESTINGHOUSE
 OWNERS GROUP AND NRR SENIOR MANAGEMENT

Senior Managers from the Office of Nuclear Reactor Regulation and representatives of the Westinghouse Owners Group (WOG) met on April 26, 2000, to discuss emerging issues and concerns. The NRC summary of this meeting is attached.

Issues discussed at the meeting included:

- the status of probabilistic safety assessment peer reviews,
- redefinition of large break loss-of-coolant accidents,
- risk-informed anticipated transient without scram model,
- reactor coolant pump seal model,
- status of the WOG baffle barrel bolt program,
- license renewal of Turkey Point and associated topical reports,
- status of containment sump debris generation and coatings failure,
- industry's review of the spent fuel pool risk report, and
- fees charged to the WOG for ACRS review of the post-accident sampling system.

The meeting summary included a list of the action items that came out of the meeting.

Attachment: Memorandum dated June 2, 2000, from Steven D. Bloom, NRR, to Stuart A. Richards, NRR, Subject: Summary of Meeting Held on April 26, 2000, with Westinghouse Owners Group and the Office of Nuclear Reactor Regulation (NRR) Senior Management

cc via e-mail w/o att.:

J. Larkin
 H. Larson
 S. Duraiswamy
 ACRS Fellows and Staff

June 2, 2000

MEMORANDUM TO: Stuart A. Richards, Director
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

FROM: Steven D. Bloom, Project Manager, Section 2
Project Directorate IV & Decommissioning /RA/
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

SUBJECT: SUMMARY OF MEETING HELD ON APRIL 26, 2000, WITH
WESTINGHOUSE OWNERS GROUP AND THE OFFICE OF
NUCLEAR REACTOR REGULATION (NRR) SENIOR MANAGEMENT

On April 26, 2000, representatives of the Westinghouse Owners Group (WOG) met with NRR management and staff. The purpose of the semi-annual senior management meeting (SMM) is to encourage communication between NRR management and the WOG, discussing emerging issues and concerns, and give the status of important activities of NRR and the WOG. The attachment contains the list of meeting attendees. Copies of the slides used during the meeting are available under ADAMS accession number ML003713446.

The WOG started the meeting by showing an organizational chart indicating the various changes in the Steering Committee and the Executive Management Group. After the introduction of the meeting participants, Sam Collins, Director of NRR, gave the opening remarks, stressing that this meeting should be more than just information. He stated that the meeting should be used by the NRC staff to help direct, prioritize and schedule work that was discussed.

The WOG began the meeting by discussing the various issues that they have classified as risk-informed issues. The first one being probabilistic safety assessment peer review status, where the WOG discussed the reviews that have been completed and the future reviews that are scheduled. The WOG stated that they were incorporating the American Society of Mechanical Engineers standard criteria into their peer review checklist and that they have only been looking up to Level 1. The NRC stated that we have not certified the peer review process. Brian Sheron, the Associate Director for Project Licensing and Technical Analysis, raised the issue of the necessity for the use of the MAPP code for a review beyond Level 1, and that the NRC and EPRI needed to get together to discuss the applicability of the code.

The next issue was the status of large break loss-of-coolant accident (LBLOCA) redefinition, as it related to the limiting break size required for the plant licensing basis. Brian Sheron stated that the discussion of LBLOCA redefinition would need to incorporate a discussion on the effects on the containment and equipment qualification. The WOG discussed the meeting that

was held on March 17, 2000, and the future meeting on May 18, 2000, where the NRC and the WOG discussed the WOG strategy and scope of work on the subject. The NRC stated that we were developing a faster relaxation than Option 3, for 10 CFR 50.46 and 10 CFR 50.44, with respect to decay heat. The WOG program mainly deals with focusing on break size.

The WOG next discussed the risk-informed anticipated transient without scram model and evaluation used in WCAP-11992. The WOG stated that during a meeting held in December 1998, the NRC raised several concerns/issues that the WOG will respond to in May 2000. The WOG requested that once the NRC reviews the responses, a followup meeting would be held in June 2000 to further discuss this issue. The WOG stated that the lead plant for this issue was Byron/Braidwood and that they intended on submitting their application for review by the end of 2000.

The next agenda item was the reactor coolant pump (RCP) seal model (Topical Report WCAP-10541) submittal and the NRC's recent response. The WOG requested some clarifications on our denial of their topical report. Jerry Wermiel stated that the NRC did not agree with the seal failure frequency numbers presented in the report as they related to RCP seal failures, however, the NRC would have future dialogues with industry to discuss the future of this issue.

The WOG commented on the importance of the risk-Informed technical specification program and their interest in the staff completing the reviews of tasks 2 and 3. This review is still underway.

The next major topic of the agenda dealt with deterministic issues. The first topic was the WOG baffle baffle bolt (BBB) program status, where the current status of the inspections that have been completed was discussed. The WOG stated that the number of bolt degradation indications is less than anticipated from European experience. The WOG stated that one reason may be the use of load-following used by the French reactors that have had indications of this problem. The WOG indicated that they would like to treat BBB degradation as a potential aging management issue for license renewal.

The next item on the agenda was license renewal. The WOG indicated that they had a lead plant in Turkey Point and that they would be referencing 3 of the 5 topical reports on license renewal that the NRC is currently reviewing. Chris Grimes indicated that he is striving to get the remaining three draft safety evaluations out by the end of June. It was noted that there are some problems with the topical report on reactor internals, since some of the information is needed on the aging management program. The last report needed for Turkey Point, on the pressurizer, is close to review completion. The NRC stated that some of the license renewal submittals are coming in at a "higher level" than did Calvert Cliffs, therefore possibly causing more requests for additional information.

The next issue was containment sump/coating issue status as it applied to debris generation and coatings failure. The WOG stated that they were following the work being done by the NRC and that the lines of communication between themselves and the NRC were very good. The NRC stated that the issue was the risk of generation, not the risk caused by the generation. The NRC stated that they would be issuing a response to the WOG letter on the application of

GDC-4 and leak before break (LBB) to debris generation. The NRC also stated that we may consider use of the phenomenon importance ranking table (PIRT) panel once again. The WOG next discussed the issue of their break opening time (BOT) topical report (WCAP-14748). The NRC had reviewed the topical report and approved its use for BBB, however the staff had not been able to allow generic application. Dick Wessman stated that the two major issues were crack propagation speed and the use of the Multiflex 3.0 Code. He stated that the NRC has expressed these concerns and that the WOG has not presented any additional information. He further stated that the NRC would issue a letter with these concerns and would set up a future meeting on this topical report.

The next major topic area was related to process issues. The first item was the post accident sampling system (PASS) elimination process. The WOG indicated that they did not want to use PASS as the first use of the consolidated line item improvement program. The WOG stated that they would like to receive the safety evaluation by June to allow licensees to implement the elimination by their fall outages.

The next issue was the Mode 3/4 LOCA topical report (WCAP-12476). The WOG, in a letter dated April 28, 1999, requested that the NRC withdraw the topical report from review and that the NRC concur that the NRC did not need to review this type of report. The NRC stated that we would send out another letter to indicate our concurrence on this request. Jerry Wermiel stated that the Commission has directed the staff to provide oversight of voluntary programs on shutdown issues like this via the inspection process.

The WOG next discussed the industry review of the spent fuel pool risk report. The WOG indicated that in the future they would like to get the report directly instead of getting it from NEI. They also indicated that they would still like to comment on the report and hoped to submit their comments within three weeks.

The next discussion related to fees charged to the WOG for ACRS time on the PASS review. The NRC stated that it has been the policy of the NRC to charge for ACRS time since 1985. The various program office charges are shown on the attachments to the invoices. Any request for clarification of charges is referred back to the program office and their response is provided to the licensees. NRR stated that they had the lead to coordinate with ACRS to then alert the licensee.

The WOG next presented a table with the submittals that the NRC is reviewing along with a list of their future submittals.

Brian Sheron stated that we are reviewing the lessons learned from the Indian Point 2 (IP2) event and that we may need to update Generic Letter 97-06. Sam Collins explained that NRR asked the Office of Research to perform an independent review of the handling of IP2 steam generator issues and that we were reviewing the comments that we had received.

Next the NRC discussed the industry initiatives paper that will be submitted to the Commission by the end of May. The NRC stated that the BBB program was a good example of this. In the paper is an explanation of how industry initiatives fit into the regulatory process.

The action items that came out of the meeting are:

1. Set up meeting on RI ATWS for June following WOG May submittal.
2. Set up meeting on RCP seal model.
3. Send out letter on GDC-4 and LBB application to debris generation.
4. Set up meeting on BOT following issuance of letter on issues still outstanding for generic application of BOT.
5. Issuance of PASS SE.
6. Letter on closeout of Mode 3/4 LOCA.
7. Look into comment period for OGs and NEI of 45 to 60 days, and sending out information to OGs and NEI, not just NEI.
8. Look into informing the OGs when one of the issues is on the ACRS agenda.
9. Sending out information to the OGs on the fees that they could be charged.

Project No. 694

Attachment: Attendance List

cc w/att: See next page