

Br. 2

October 23, 2008

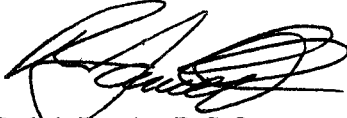
Licensing Assistant Section

Re: Terminate license and close out reports of Nuclear Medicine room

To Whom It May Concern:

03036530
This letter is to inform the N.R.C. that on November 12, 2008 the Nuclear Medicine Department will close. We request that you terminate N.R.C license number 29-30895-01. A disposition of our CS 137 vial for dose calibrator and forms for close out surveys of Nuclear Medicine room.

Respectfully yours,



Dr. Ralph Dauito R.S.O


Evelyn O'Donnell RTR

RECEIVED
REGION 1
2008 NOV 13 PM 2:32

143000
NPS/RGNI MATERIALS-002

October 23, 2008

William Csaszar, Supervisor
Radioactive Materials Section

Re: Terminate license and close out reports of nuclear room

Dear Mr. Csaszar,

This letter is to inform the state of N.J. that on November 12, 2008, the Nuclear Medicine Department will close. We request that you terminate N.J. State license NJSL 20804/01/000. A disposition of the Co57 sheet source and form for the close out surveys of Nuclear Medicine room.

Respectfully yours,


Dr. Dauito R.S.O.


Evelyn O'Donnell RTNR

RECEIVED
REGION 1

2008 NOV 13 PM 2:32

CERTIFICATE OF DISPOSITION OF MATERIALS

PLEASE READ THESE INSTRUCTIONS BEFORE COMPLETING NRC FORM 314.

Subpart E of 10 CFR Part 20 establishes the radiological criteria for license terminations/decommissioning of facilities licensed under 10 CFR Parts 30, 40, 50, 60, 61, 70, and 72, as well as other facilities subject to the Commission's jurisdiction under the Atomic Energy Act of 1954, as amended, and the Energy Reorganization Act of 1974, as amended.

INSTRUCTIONS

Section B, Item 2.

Licensees should describe the specific radioactive material transfer actions. If radioactive wastes were generated in terminating this license, the licensee should describe the disposal actions taken, including the disposition of low-level radioactive waste, mixed waste, greater-than-Class-C waste, and sealed sources.

Section B, Item 2.a.

The information provided concerning the transfer of radioactive material to another licensee should specify the date of the transfer, the name of the licensee recipient, an individual contact name and telephone number for the licensee recipient, and the recipient's NRC or Agreement State license number.

Section B, Item 2.b.

For disposal of radioactive materials, licensees should describe the specific disposal method or procedure (e.g., decay-in-storage). For those cases when radioactive materials are disposed of by a licensed disposal site or by a waste contractor, the licensee should specify the name, address, and telephone number of the licensed disposal site operator or waste contractor.

Section B, Item 2.c.

"Residual radioactivity," as defined in 10 CFR 20.1003, means radioactivity in 'areas' (structures, materials, soils, etc.) remaining as a result of activities (licensed and unlicensed) under the licensee's control from sources used by the licensee, excluding background radiation. ALARA is defined in 10 CFR 20.1003.

FILE CERTIFICATES AS FOLLOWS:

IF YOU ARE LOCATED IN:

CONNECTICUT, DELAWARE, DISTRICT OF COLUMBIA,
MAINE, MARYLAND, MASSACHUSETTS, NEW HAMPSHIRE,
NEW JERSEY, NEW YORK, PENNSYLVANIA, RHODE
ISLAND, OR VERMONT, SEND CERTIFICATES TO:

LICENSING ASSISTANT SECTION
NUCLEAR MATERIALS SAFETY BRANCH
U.S. NUCLEAR REGULATORY COMMISSION, REGION I
475 ALLENDALE ROAD
KING OF PRUSSIA, PA 19406-1415

ALABAMA, FLORIDA, GEORGIA, KENTUCKY, MISSISSIPPI,
NORTH CAROLINA, PUERTO RICO, SOUTH CAROLINA,
TENNESSEE, VIRGINIA, VIRGIN ISLANDS, OR WEST
VIRGINIA, SEND CERTIFICATES TO:

NUCLEAR MATERIALS SAFETY SECTION
U. S. NUCLEAR REGULATORY COMMISSION, REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET, S.W., SUITE 23T85
ATLANTA, GEORGIA 30303-8931

IF YOU ARE LOCATED IN:

ILLINOIS, INDIANA, IOWA, MICHIGAN, MINNESOTA,
MISSOURI, OHIO, OR WISCONSIN, SEND CERTIFICATES
TO:

MATERIALS LICENSING SECTION
U.S. NUCLEAR REGULATORY COMMISSION, REGION III
801 WARRENVILLE RD.
LISLE, IL 60532-4351

ALASKA, ARIZONA, ARKANSAS, CALIFORNIA,
COLORADO, HAWAII, IDAHO, KANSAS, LOUISIANA,
MONTANA, NEBRASKA, NEVADA, NEW MEXICO, NORTH
DAKOTA, OKLAHOMA, OREGON, PACIFIC TRUST
TERRITORIES, SOUTH DAKOTA, TEXAS, UTAH,
WASHINGTON, OR WYOMING, SEND CERTIFICATES TO:

MATERIAL RADIATION PROTECTION SECTION
U. S. NUCLEAR REGULATORY COMMISSION, REGION IV
611 RYAN PLAZA DRIVE, SUITE 400
ARLINGTON, TX 76011-8064

RADIOACTIVE SEALED SOURCE LEAK TEST REPORT

INSTITUTION: EASTLANKE Diagnostic Inst.

NRC LICENSE #: 29-30895-0 **STATE LICENSE #:** NYSL-20804

SOURCES TESTED: Co 57 Bm 01103681 sheet
Source

Cs 137 (28792) vial

DATE(S) OF TEST: 11-12-08

TYPE OF TEST: ALCOHOL SWAB ✓

DETECTION EQUIPMENT: CAPRAC Capintec

APPROXIMATE EFFICIENCY: Cs 137 - 2.87×10^5
Co 57 - 1.22×10^6

RESULTS:

<u>Source</u>	<u>Net CPM</u>	<u>Removable Activity</u> <u>(uCi)</u>
<u>Co 57</u>	<u>2</u>	<u>20.001</u>
<u>Cs 137</u>	<u>0</u>	<u>20.001</u>

The above test satisfies the leak test requirements as specified under the conditions of the NRC and State Licenses.



Radiation Safety Officer

*RadQual
9/18/08*

Test and Measurement Report

Model #: BM01- 10 Serial #: BM01103681 Reference Date: August 10, 2008

Radioactive Isotope Information at Reference Date

Primary Nuclide:	Co-57	10 mCi	370 MBq	
Principle Contaminants:	Co-56	0.002 mCi	0.0555 MBq	0.02%
	Co-58	0.001 mCi	0.037 MBq	0.01%

Leak Test Results

The subject source was tested for radioactivity leakage and surface contamination as described by ANSI Standard N43.6-1997. Any leakage and/or contamination detected did not exceed 5×10^{-3} microcuries.

By: *[Signature]* Date: 7/14/08

Physical Properties (for reference only)

Total Dimensions: 17.9" (45.5 cm) x 25.4" (64.5 cm)
Active Dimensions: 16.4" (41.7 cm) x 23.9" (60.7 cm)

Physical Properties as Tested for Uniformity (for reference only)

Total Field of View: 15.2" (38.6 cm) x 23.9" (60.7 cm)
Useful Field of View: 13.7" (34.8 cm) x 21.5" (54.6 cm)

Source Decay Characteristics¹

Cobalt-57 half-life is 271.7 days

Photon Energies (keV)	Abundance (%)
γ_1 14.4	9.5
γ_2 122	85.6
γ_3 136	10.6
γ_9 692	0.16

(1) "A Handbook of Radioactivity Measurement Procedures", NRC Report No. 58, Second Edition, (February 1985).

Most gamma cameras tend to resolve the γ_2 and γ_3 as a single non-Gaussian peak for window alignment during quality control procedures. The medium and high-energy gamma emissions from the trace Co-56 and Co-58 (0.5 MeV — 3.0 MeV) impurities decrease relative to the Co-57 with an effective half-life of 105 days.



Decay Reference Chart for Cobalt-57 Flood Source

Source Serial No. **BM01103681**

Source Reference Date **10-Aug-08**

Labeled Activity **10 mCi**
370 MBq

Date	mCi*	MBq*
10-Aug-08	10.00	370.0
17-Aug-08	9.82	363.5
24-Aug-08	9.65	357.0
31-Aug-08	9.48	350.7
7-Sep-08	9.31	344.5
14-Sep-08	9.15	338.4
21-Sep-08	8.98	332.4
28-Sep-08	8.83	326.5
5-Oct-08	8.67	320.8
12-Oct-08	8.52	315.1
19-Oct-08	8.36	309.5
26-Oct-08	8.22	304.0
2-Nov-08	8.07	298.6
9-Nov-08	7.93	293.4
16-Nov-08	7.79	288.2
23-Nov-08	7.65	283.1
30-Nov-08	7.52	278.1
7-Dec-08	7.38	273.1
14-Dec-08	7.25	268.3
21-Dec-08	7.12	263.6
28-Dec-08	7.00	258.9
4-Jan-09	6.87	254.3
11-Jan-09	6.75	249.8
18-Jan-09	6.63	245.4
25-Jan-09	6.51	241.0
1-Feb-09	6.40	236.8
8-Feb-09	6.29	232.6
15-Feb-09	6.18	228.5
22-Feb-09	6.07	224.4
1-Mar-09	5.96	220.5
8-Mar-09	5.85	216.6

Date	mCi*	MBq*
15-Mar-09	5.75	212.7
22-Mar-09	5.65	209.0
29-Mar-09	5.55	205.3
5-Apr-09	5.45	201.6
12-Apr-09	5.35	198.1
19-Apr-09	5.26	194.6
26-Apr-09	5.17	191.1
3-May-09	5.07	187.7
10-May-09	4.98	184.4
17-May-09	4.90	181.2
24-May-09	4.81	177.9
31-May-09	4.72	174.8
7-Jun-09	4.64	171.7
14-Jun-09	4.56	168.7
21-Jun-09	4.48	165.7
28-Jun-09	4.40	162.7
5-Jul-09	4.32	159.9
12-Jul-09	4.24	157.0
19-Jul-09	4.17	154.3
26-Jul-09	4.10	151.5
2-Aug-09	4.02	148.8
9-Aug-09	3.95	146.2
16-Aug-09	3.88	143.6
23-Aug-09	3.81	141.1
30-Aug-09	3.75	138.6
6-Sep-09	3.68	136.1
13-Sep-09	3.61	133.7
20-Sep-09	3.55	131.4
27-Sep-09	3.49	129.0
4-Oct-09	3.43	126.8
11-Oct-09	3.37	124.5

Date	mCi*	MBq*
18-Oct-09	3.31	122.3
25-Oct-09	3.25	120.1
22-Nov-09	3.02	111.9
29-Nov-09	2.97	109.9
6-Dec-09	2.92	107.9
13-Dec-09	2.87	106.0
20-Dec-09	2.81	104.2
27-Dec-09	2.77	102.3
3-Jan-10	2.72	100.5
10-Jan-10	2.67	98.7
17-Jan-10	2.62	97.0
24-Jan-10	2.57	95.3
31-Jan-10	2.53	93.6
7-Feb-10	2.48	91.9
14-Feb-10	2.44	90.3
21-Feb-10	2.40	88.7
28-Feb-10	2.35	87.1
7-Mar-10	2.31	85.6
14-Mar-10	2.27	84.1
21-Mar-10	2.23	82.6
28-Mar-10	2.19	81.1
4-Apr-10	2.15	79.7
11-Apr-10	2.12	78.3
18-Apr-10	2.08	76.9
25-Apr-10	2.04	75.5
2-May-10	2.01	74.2
9-May-10	1.97	72.9
16-May-10	1.93	71.6

* mCi and MBq values are reference values only, this is not a calibrated source

Radiation Safety Instructions and Recommendations For Use and Storage of Cobalt 57 (Co-57) Flood Sources

The Co-57 flood source has been manufactured to provide maximum safety and longevity of service. The flood source has been designed to satisfy the safety performance requirements of ANSI Standard N43.6-1997 for classification C22312, as recommended for calibration sources.

Recommended Use and Service

The Co-57 flood source provides a uniform radiation field for performing quality control testing of a gamma camera flood field. A flood field study provides a record of camera field uniformity performance and subsequent need for correction of a less than optimal performing camera prior to use in a diagnostic procedure.

When performing a study using a Co-57 flood source, you must be sure that the energy window for the instrument includes the 122 keV gamma ray (100 keV — 150 keV is recommended). The use of a narrow Technetium 99 (Tc-99) window (140 keV) with Co-57 may

result in your flood field image appearing faulty. Refer to your instrument manual to ascertain the correct number of counts to acquire for your flood study with the source directly on the collimator.

The Co-57 flood source should be used and stored at 10-40° C, ambient air pressure, and 10% to 80% relative humidity. Care should be taken to avoid contact with organic solvents, hot surfaces, and excessive mechanical stress.

Radiation Safety Recommendation

Generally, radiation protection procedures used for the measurement and handling of radio-pharmaceutical products is applicable to the handling of a Co-57 flood source. The following chart is provided to aid in effective radiation exposure control, actual readings may vary depending upon measurement instrumentation and technique. Consult with your Radiation Protection Office for any additional handling requirements.

Model Number	Surface Dose Rate Unshielded	(1) Surface Dose Rate Shielded
BM01-10, BM01-15, BM01-20, BM01-99	Approx. 2.9 mR/hr/mCi	Approx. 0.10 mR/hr/mCi
BM02-05, BM02-10, BM02-99	Approx. 3.8 mR/hr/mCi	Approx. 0.15mR/hr/mCi
BM04-10, BM04-15, BM04-99	Approx 3.6 mR/hr/mCi	Approx. 0.15 mR/hr/mCi
BM05-10, BM05-99	Approx. 9.0 mR/hr/mCi	Approx. 0.10 mR/hr/mCi
BM07-10, BM07-75, BM07-99	Approx. 7.0 mR/hr/mCi	Approx. 0.10 mR/hr/mCi

(1) Shielded with Rad Shield

Technical Data

Test and Measurement Report

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By: 

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1-Mar-09	5.96	220.5
8-Mar-09	5.85	216.6

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12-Apr-09	5.35	198.1
19-Apr-09	5.26	194.6
26-Apr-09	5.17	191.1
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31-May-09	4.72	174.8
7-Jun-09	4.64	171.7
14-Jun-09	4.56	168.7
21-Jun-09	4.48	165.7
28-Jun-09	4.40	162.7
5-Jul-09	4.32	159.9
12-Jul-09	4.24	157.0
19-Jul-09	4.17	154.3
26-Jul-09	4.10	151.5
2-Aug-09	4.02	148.8
9-Aug-09	3.95	146.2
16-Aug-09	3.88	143.6
23-Aug-09	3.81	141.1
30-Aug-09	3.75	138.6
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13-Sep-09	3.61	133.7
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6-Dec-09	2.92	107.9
13-Dec-09	2.87	106.0
20-Dec-09	2.81	104.2
27-Dec-09	2.77	102.3
3-Jan-10	2.72	100.5
10-Jan-10	2.67	98.7
17-Jan-10	2.62	97.0
24-Jan-10	2.57	95.3
31-Jan-10	2.53	93.6
7-Feb-10	2.48	91.9
14-Feb-10	2.44	90.3
21-Feb-10	2.40	88.7
28-Feb-10	2.35	87.1
7-Mar-10	2.31	85.6
14-Mar-10	2.27	84.1
21-Mar-10	2.23	82.6
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11-Apr-10	2.12	78.3
18-Apr-10	2.08	76.9
25-Apr-10	2.04	75.5
2-May-10	2.01	74.2
9-May-10	1.97	72.9
16-May-10	1.93	71.6

* mCi and MBq values are reference values only, this is not a calibrated source

CERTIFICATE OF CALIBRATION

MODEL MED3400 GAMMA REFERENCE STANDARD

Radionuclide:	Cs-137	Activity:	37.66 kBq (1.018 μ Ci)
Serial Number:	17417	Reference Date:	1200 PST April 1, 2002
Half Life ⁽¹⁾ :	30.0 \pm 0.2 years		

PRINCIPAL EMISSIONS⁽¹⁾

Type	Energy (keV)	Intensity (%)
gamma	661.66	85.21

SOURCE DESCRIPTION

Active Diameter:	1 mm	Nature of Active Deposit:	Cesium Chloride in resin matrix
Overall Diameter:	12.7 mm	Position of Active Bead:	8 mm from the end of rod
Overall Length:	102 mm		

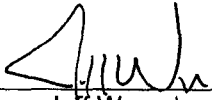
METHOD OF CALIBRATION

The source was calibrated on a high purity germanium detector against a Cs-137 standard at 661.66 keV. The activity of the standard was determined using an efficiency established and verified through ongoing intercomparisons with the National Institute of Standards and Technology. This standard is indirectly (implicitly) traceable to the National Institute of Standards and Technology.

North American Scientific, Inc. actively participates in the Radioactivity Measurements Assurance Program conducted by the National Institute of Standards and Technology in cooperation with the Nuclear Energy Institute.

TOTAL UNCERTAINTY (99% Confidence Level)

Systematic uncertainty	3.22%
Random uncertainty	1.06%
Total uncertainty (quadratic sum)	\pm 3.39%


Jeff Wagner
Calibration Laboratory

March 28, 2002
Date

REFERENCES

- (1) Table of Radioactive Isotopes, 7th edition, 1986.

• LEAK TEST CERTIFICATION ON REVERSE •

TECHNICAL DATA SHEET

MODEL NO.: MED3503 Spot Marker

QUANTITY: 1

<u>NUCLIDE</u>	<u>S/N OR LOT NO.</u>	<u>ACTIVITY</u>	<u>UNCERTAINTY</u>	<u>REFERENCE DATE</u>	<u>HALF-LIFE</u>
Co-57	29603	2.0 MBq (55 uCi)	±15%	November 1, 2002	271.77 days

NATURE OF ACTIVE DEPOSIT: Evaporated CoCl₂

BACKING: n/a

COVER: n/a

ACTIVE DIAMETER/AREA: 3 mm active diameter

OVERALL DIAMETER/DIMENSIONS: 25.4 mm diameter x 6.4 mm thick

• LEAK TEST CERTIFICATION ON REVERSE •

North American Scientific, Inc. 7435 Greenbush Ave., North Hollywood, CA 91605 (818) 734-8600 Fax (818) 734-5200

CERTIFICATE OF CALIBRATION

MODEL MED3550 GAMMA REFERENCE STANDARD

Radionuclide:	Cs-137	Activity:	7.683 MBq (207.7 μ Ci)
Serial Number:	28792	Reference Date:	1200 PDT October 1, 2002
Half Life ⁽¹⁾ :	30.0 \pm 0.2 years		

PRINCIPAL EMISSIONS⁽¹⁾

Type	Energy (keV)	Intensity (%)
gamma	661.660	85.21

SOURCE DESCRIPTION

The activity is uniformly distributed throughout approximately 10 milliliters of epoxy resin and covered with 10 milliliters of inactive epoxy resin in a 30 milliliter polyethylene vial with an epoxy sealed cap.

METHOD OF CALIBRATION

The standard was calibrated by direct comparison to 10 milliliters of standardized solution traceable to the National Institute of Standards and Technology, in an identical geometry, using a pressurized ion chamber. Therefore, the activity value provided is equivalent to 10 milliliters of standardized solution. This standard is indirectly (implicitly) traceable to the National Institute of Standards and Technology.

North American Scientific, Inc. actively participates in the Radioactivity Measurements Assurance Program conducted by the National Institute of Standards and Technology in cooperation with the Nuclear Energy Institute.

TOTAL UNCERTAINTY (99% Confidence Level) \pm 5.00%



Seyed Miri
Calibration Laboratory

September 24, 2002

Date

REFERENCES

- (1) Table of Radioactive Isotopes, 7th edition, 1986.

• LEAK TEST CERTIFICATION ON REVERSE •

SHIPPING PAPER FOR DANGEROUS GOODS

CARRIER: Cardinal Health

ORIGIN:

Bill No.:

SHIPPER:

Eastlantic Vineland Diagnostic
1470 S. Main Rd.
Vineland, NJ 08360
856-794-9674

CONSIGNEE NAME AND ADDRESS:

Cardinal Health
650 Elmwood Ave.
Sharon Hill, PA 19079
610-461-7070

Proper Shipping Name

Radioactive Material, Type A Package

Hazard Class

7

UN No.

UN 2915

NATURE AND QUANTITY OF DANGEROUS GOODS

NO. PIECES
WITHIN BOX

RADIONUCLIDE

CHEMICAL FORM/PHYSICAL STATE

ACTIVITY - GBq

ACTIVITY - mCi

2

Co-57

solid epoxy

0.285

(7.702)

3

Cs-137

solid epoxy

0.006

(0.175)

0.000

0.000

0.000

0.000

0.000

0.000

Total Activity Contained in one (1) box:

0.291

(7.877)

RADIOACTIVE LABEL CATEGORY:

~~White-I~~

Yellow-II

Transport Index:

0.2

Emergency Spill-Leak-Fire-Exposure or Accident Contact: CHEMTREC 800-424-9300

This is to certify that the above named materials are properly classified, described, packaged, marked and labeled, and are in proper condition for transportation according to applicable regulations for the Department of Transportation.

Certified By:

Courier Signature:

Consignee Signature:

Evelyn O'Donnell

Signatures

Date:

Time:

Date:

Time:

Date:

Time:

10/1/08

12:21

Run:

Route:

Box:

Evelyn O'Donnell
DAILY SURVEY AND WEEKLY WIPE AREAS

11 TO
2008

Ludlum I mr/hr

AREA	MONDAY	TUESDAY	WEDNESDAY	THURSDAY	FRIDAY	WEEKLY WIPES (DPM)
BACKGROUND CPMs	11-11-08					100 cpm
1) NUC MED TABLE	CS-137 ml					410 cpm
2) NUC MED FLOOR	BKGD = 0.2 mr/hr					0 dpm
3) INJECTION AREA	surface = 0.2 mr/hr.					
4) HOT LAB FLOOR						
5) HOT LAB COUNTER	(126.3 dpm)					
6) BEHIND L BLOCK	wipe					
7) WELL COUNTER	Dose Calibrator					EO'D
TECHNOLOGIST	RSO					

11 TO 2008

Liedlum ✓ I m m / h

yellow MON

MONDAY | TUESDAY | WEDNESDAY | THURSDAY | FRIDAY | WEEKLY WIPES (DPW)

BACKGROUND *cpm's*

~~ms II~~

CO 57

Shut Sauce 410 gms

~~Depms~~
Co:ID

50.10

$$B_{\text{Grd}} = 0.2 \text{ mol/L}$$

surface = 0.2 m²/m

126.3 cps

Dose
Calibrator

RSO.

Room Close Out

terminate LICENSES

Evelyn O'Donnell
DAILY SURVEY AND WEEKLY WIPE AREAS

11 TO 13
2008

Ludlum ☒ I mru/lu

AREA	MONDAY	TUESDAY	WEDNESDAY	THURSDAY	FRIDAY	WEEKLY WIPES (CPM's)
BACKGROUND CPM's			11-12-08 .02			11-12-08-1000 406 cpm
1) NUC MED TABLE			.02			0 done
2) NUC MED FLOOR			.02			0
3) INJECTION AREA			.02			0
4) HOT LAB FLOOR			.02			2.7
5) HOT LAB COUNTER			.02			37.3
6) BEHIND L BLOCK			.02			123.6
7) WELL COUNTER ^{Dose Calibrator}			.02			0

TECHNOLOGIST

E.O.D.

RSO

[Signature]

E.O.D.

1 0 9 5 1 0 0 1

		2ND	Pieces: 1/1
FM: EASTLAITIC DIAGHSTIC INSTITUTE BRITTANY SMITH 1470 SOUTH MAIN ROAD VINELAND, NJ 08360 UNITED STATES Phone: (856) 690-0300		ORIGIN: AIY	
To: LICENSING ASSISTANT SECTION NUCLEAR MATERIALS SAFETY BRANCH US NUC. REG. COMMISSION, REGION 1 475 ALLENDALE RD. KING OF PRUSSIA, PA 19406 UNITED STATES		POSTCODE 19406 TEL: 856-690-0300	
		14FR Day	
Description			
Weight: Letter Date: 2006-11-12			
DHL standard terms and conditions apply.			
		KPDT 2G ABH	
(2L)US19406			
			
WAYBILL: 29673538650		(Non-Negotiable)	

This is to acknowledge the receipt of your letter/application dated

10/23/08, and to inform you that the initial processing which includes an administrative review has been performed.

☒ Termination C29-30895-01
There were no administrative omissions. Your application was assigned to a technical reviewer. Please note that the technical review may identify additional omissions or require additional information.

☐ Please provide to this office within 30 days of your receipt of this card

A copy of your action has been forwarded to our License Fee & Accounts Receivable Branch, who will contact you separately if there is a fee issue involved.

Your action has been assigned **Mail Control Number** 143000.
When calling to inquire about this action, please refer to this control number.
You may call us on (610) 337-5398, or 337-5260.

NRC FORM 532 (RI)
(6-96)

Sincerely,
Licensing Assistance Team Leader