



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**  
WASHINGTON, D.C. 20555-0001

November 20, 2008

Mr. Charles G. Pardee  
President and Chief Nuclear Officer  
Exelon Nuclear  
4300 Winfield Road  
Warrenville, IL 60555

**SUBJECT: LASALLE COUNTY STATION, UNITS 1 AND 2 – REQUEST FOR  
SUPPLEMENTAL INFORMATION REGARDING RELIEF REQUEST CR-26,  
INSERVICE INSPECTION PROGRAM RELIEF REGARDING EXAMINATION  
COVERAGE FOR SECOND 10-YEAR INSERVICE INSPECTION INTERVAL  
(TAC NOS. MD9817 AND MD9818)**

Dear Mr. Pardee:

By letter to the Nuclear Regulatory Commission (NRC), dated October 1, 2008, Exelon Generation Company, LLC (the licensee), submitted a relief request (RR) for LaSalle County Station (LSCS), Units 1 and 2. The proposed relief requested NRC approval to use a proposed alternative to the existing American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," on the basis that compliance with the specified requirements is impractical due to plant design. The purpose of this letter is to provide the results of the NRC staff's acceptance review of this RR. The acceptance review was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical review. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Pursuant to Section 50.55a(g)(5)(iii) of Title 10 of the *Code of Federal Regulations* (10 CFR), if the licensee has determined that conformance with certain code requirements is impractical for its facility, the licensee shall notify the Commission and submit, as specified in § 50.4, information to support the determination.

Based on the review conducted by the NRC staff, it has been determined that there is insufficient technical information in scope and depth to allow NRC to complete its technical review. Therefore, the NRC staff has requested additional information be provided in order to make an independent assessment regarding the acceptability of the RR in terms of regulatory requirements and the protection of public health and safety.

In order to make the request complete, the NRC staff requests that you supplement your October 1, 2008, submittal to address the information requested below, by November 21, 2008. This will enable the NRC staff to begin its detailed technical review. If the information responsive to the NRC staff's request is not received by this date, the application will not be accepted for review pursuant to 10 CFR 2.101, and the NRC staff will cease its review activities associated with the request. If the request is subsequently accepted for review, you will be advised of any further information needed to support the NRC staff's detailed technical review

by separate correspondence. The information requested and associated time-frame in this letter was discussed with members of your staff on November 6, 2008.

INFORMATION NEEDED

Provide further detailed information to support the basis for each limited examination in RR CR-26, and therefore, demonstrate impracticality. This information should include detailed descriptions (with sufficient explanation, and lay-out or cross-sectional drawings/sketches) to enable the NRC staff to fully understand the causes of the inspection coverage limitations and their impact on examination volume coverage.

If you have any questions, please contact Stephen P. Sands, at (301) 415-3154.

Sincerely,

A handwritten signature in black ink, appearing to read "Stephen P. Sands", followed by a stylized flourish.

Stephen P. Sands, Project Manager  
Plant Licensing Branch III-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-373 and 50-374

cc: Distribution via Listserv

C. Pardee

- 2 -

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**/RA by MDavid for/**

Stephen P. Sands, Project Manager  
Plant Licensing Branch III-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-373 and 50-374

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\* w/ comments

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