UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
612 EAST LAMAR BLVD, SUITE 400
ARLINGTON, TEXAS 76011-4125

November 10, 2008

UniTech Services Group
ATTN: Glen Roberts
Corporate Health Physicist
295 Parker Street
P.O. Box 51957
Springfield, MA 01151

SUBJECT: NRC INSPECTION REPORT 030-06869/08-002

Dear Mr. Roberts:

This report refers to the inspection conducted on September 23, 2008, at your former nuclear laundry facility located in Honolulu, Hawaii. The inspection was an examination of activities as they relate to safety and compliance with the Commission's rules and regulations. Within these areas, the inspection consisted of the performance of a confirmatory radiological survey. Details of the inspection were presented to you at the conclusion of the onsite inspection. Following the initial review of the final status survey report and confirmatory survey results, the final inspection results were discussed with you by telephone on October 27, 2008. No violations were identified; therefore, no response to this letter is required.

During the inspection, the NRC conducted a confirmatory radiological survey including the measurement of ambient gamma exposure rates and surface contamination levels. The NRC inspector compared the confirmatory sample results to the radiological criteria for license termination as specified in Subpart E to Title 10 of the Code of Federal Regulations (CFR) Part 20 and related regulatory guidance documents. All sample results were less than the screening criteria specified in NUREG-1757, Volume 1, Revision 2, "Consolidated Decommissioning Guidance," Appendix B, "Screening Values." The results of the inspection indicate that you have effectively remediated your former nuclear laundry facility.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response, if you choose to provide one, will be made available electronically for public inspection in the NRC Public Document Room or from the NRC's document system (ADAMS), accessible from the NRC's Web site at http://www.nrc.gov/reading-rm/adams.html. To the extent possible, your response should not include any personal privacy, proprietary, or safeguards information so that it can be made available to the Public without redaction.

Should you have any questions concerning this inspection, please contact the undersigned at (817) 860-8197 or Mr. Robert J. Evans, Senior Health Physicist, at (817) 860-8234.

Sincerely,

/RA/

Jack E. Whitten, Chief
Nuclear Materials Safety Branch B

Docket No.: 030-06869
License No.: 53-13668-01

Enclosure:
NRC Inspection Report
030-06869/08-002

cc w/enclosure:
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cc w/enclosure:
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SUNSI Review Completed: RRM ADAMS: ■ Yes □ No Initials: RRM
■ Publicly Available □ Non-Publicly Available □ Sensitive ■ Non-Sensitive

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U.S. NUCLEAR REGULATORY COMMISSION
REGION IV

Docket No.:	030-06869
License No.:	53-13668-01
Report No.:	030-06869/08-002
Licensee:	UniTech Services Group
Facility:	Former Nuclear Laundry
Location:	3050 Ualena Street, Suite C Honolulu, Hawaii
Dates:	September 23, 2008
Inspector:	Rick Muñoz, Health Physicist Nuclear Materials Safety Branch A
Approved by:	Jack E. Whitten, Chief Nuclear Materials Safety Branch B
Attachment:	Supplemental Inspection Information

ENCLOSURE

EXECUTIVE SUMMARY

UniTech Services Group
NRC Inspection Report 030-06869/08-002

This inspection included a review of site status and decommissioning activities, as well as a closeout inspection and survey. In summary, the licensee completed decommissioning in accordance with license and regulatory requirements.

Closeout Inspection and Survey

- The inspector conducted a confirmatory survey which included measurement of ambient gamma exposure rates and building surface contamination levels. None of the survey results exceeded the respective guideline values for the release of building surfaces (Section 2).

Report Details

1 Site Status

The licensee completed decommissioning in phases. During the first phase of decommissioning, the licensee surveyed and free-released non-contaminated equipment from the site. The equipment released included tools, conduit, pipe, boxes, wood, buckets/drums, tables, and debris. The first phase of decommissioning was completed in November 2007.

During the second phase, the licensee surveyed, demolished, and free-released a concrete wall from within the facility. The licensee also removed potentially contaminated equipment from service and packaged the equipment into one of two intermodal containers that were staged onsite. The equipment removed from service included the nuclear laundry washers, dryers, and lint collection system. The second phase was completed in February 2008.

The third phase of decommissioning included remediation of the washer discharge water trench and the two in-ground water storage pits. In addition, the waste water treatment system piping and filters were permanently removed from service. Other work completed included decontamination of building surfaces and placement of the remainder of potentially contaminated material inside of the two intermodal containers. The third phase was completed in May 2008.

The fourth phase of decommissioning included surveying the remaining drain lines, performance of the final status survey, packaging and shipment of the remaining potentially contaminated wastes from the facility. This activity was completed in September 2008. The waste water pit void spaces will be backfilled after the license has been terminated.

The licensee submitted the final status survey report to the NRC by letter dated September 4, 2008. The report was still under formal NRC review at the end of the onsite inspection.

2 Closeout Inspection and Survey (83890)

2.1 Inspection Scope

The inspector conducted a confirmatory radiological survey to help establish whether the site had been decontaminated to acceptable levels for unrestricted use.

2.2 Observations and Findings

At the time of the inspection, the licensee had completed all decommissioning activities. The inspector conducted a confirmatory survey of the laydown area, the area where the intermodal containers were previously situated, and the lateral drain lines adjacent to the intermodal location. The survey completed by the inspector included measurement of ambient gamma radiation exposure rates and surface contamination levels.

Prior to conducting the survey, the inspector collected background exposure rate and surface radioactivity measurements. The inspector also calculated a minimum

detectable concentration (MDC) for the particulate contamination survey meter. The survey meters used during the inspection and their respective calibration due dates are provide in the Attachment to this Enclosure.

The ambient gamma radiation exposure rates were measured using a Ludlum Model 19 microRoentgen meter. The background exposure rates ranged from 2-4 microRoentgens per hour (μR/hr). The exposure rates of areas surveyed ranged from 3-4 μR/hr. The inspector noted only one very small area that exhibited an elevated exposure rate of 6 μR/hr. This measurement was obtained in a corner area within the drain trench. Although there is no regulatory limit for ambient gamma exposure rate for free-release of decommissioned sites, the measured exposure rates were less than the guidance value, 10 μR/hr above background, provided in NRC Policy and Guidance Directive FC 83-23, "Termination of Byproduct, Source, and Special Nuclear Materials Licenses."

The inspector conducted surface contamination measurements using an Eberline E600 survey meter coupled to an alpha-beta probe. The inspector collected 84 1-minute counts for alpha and beta contamination on building and equipment surfaces. No sample results exceeded the MDC of the survey meter. The highest sample result was 205 dpm/100 cm^2 above background. This sample result was still below the licensee's surrogate derived concentration guideline level of approximately 6,000 dpm/100 cm^2.

2.3 Conclusions

The inspector conducted a confirmatory survey which included measurement of ambient gamma exposure rates and building surface contamination levels. None of the survey results exceeded the respective guideline values for the release of building surfaces.

3 Exit Meeting Summary

The inspector reviewed the scope and findings of the inspection during an exit meeting that was conducted at the conclusion of the onsite inspection on September 23, 2008. Following the review of the final status survey report and confirmatory survey results, a final exit briefing was conducted telephonically with the corporate radiation safety officer on October 27, 2008. The licensee did not identify as proprietary any information provided to, or reviewed, by the inspector.

SUPPLEMENTAL INSPECTION INFORMATION

Partial List of Persons Contacted

UniTech Services Group

Glenn Roberts, Corporate Health Physicist

Items Opened, Closed, and Discussed

Open

None

Closed

None

Discussed

None

Inspection Procedures Used

IP 83890	Closeout Inspection and Survey

List of Acronyms and Abbreviations Used

dpm/100 cm^2	disintegrations per minute per 100-square centimeters
IP	inspection procedures
µR/hr	microRoentgens per hour
MDC	minimum detectable concentration

Radiological Survey Instrumentation Used

Ludlum Model 19 MicroRoentgen Meter, Serial No. 33541, NRC No. 015525, calibration due date of February 14, 2009

Eberline E600 count rate meter, Serial No. 00763, NRC No. 063472, calibration due date of August 04, 2009

SHP380AB alpha-beta probe, Serial No. 01108, NRC No. 079976, calibration due date of August 04, 2009