Entergy

Entergy Operations, Inc.
P.O. Box 756
Port Gibson, MS 39150
Tel 601 437 2800

GNRO-2008/00070

October 28, 2008

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555-0001

Subject: Cycle 17 Update to the Core Operating Limits Report (COLR)

Grand Gulf Nuclear Station
Docket No. 50-416
License No. NPF-29

Dear Sir or Madam:

Attached is the Cycle 17 update to the Grand Gulf Nuclear Station Core Operating Limits Report (COLR).

This letter does not contain any commitments.

Yours truly,

Michael J. Larson
Acting-Manager Licensing

MJL/JEO
attachment: Cycle 17 Update to the GGNS COLR
cc: (See Next Page)

cc:

NRC Senior Resident Inspector
Grand Gulf Nuclear Station
Port Gibson, MS 39150

U. S. Nuclear Regulatory Commission
ATTN: Mr, Elmo E. Collins (w/2)
Regional Administrator, Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-4005

U. S. Nuclear Regulatory Commission
ATTN: Mr. Carl F. Lyon, NRR/DOLR (w/2)
ATTN: ADDRESSEE ONLY
ATTN: Courier Delivery Only
Mail Stop OWFN/8 B1
11555 Rockville Pike
Rockville, MD 20852-2378

Attachment to

GNRO-2008/00070

CYCLE 17 UPDATE TO THE GRAND GULF NUCLEAR STATION (COLR)

Grand Gulf Nuclear Station

Core Operating Limits Report

CORE OPERATING LIMITS REPORT

REASON FOR REVISION

This revision provides the Cycle 17 core operating limits. These limits are based on a core power of 3898 MWt.

TABLE OF CONTENTS

CORE OPERATING LIMITS REPORT

1.0 PURPOSE

On October 4, 1988, the NRC issued Generic Letter 88-16 [3.1.1] encouraging licensees to remove cycle-specific parameter limits from Technical Specifications and to place these limits in a formal report to be prepared by the licensee. As long as the parameter limits were developed with NRC-approved methodologies, the letter indicated that this would remove unnecessary burdens on licensee and NRC resources.

On October 29, 1992, Entergy Operations submitted a Proposed Amendment to the Grand Gulf Operating License requesting changes to the GGNS Technical Specifications to remove certain reactor physics parameter limits that change each fuel cycle [3.1.2]. This amendment committed to placing these operating limits in a separate Core Operating Limits Report (COLR) which is defined in Technical Specifications. This PCOL was approved by the NRC by SER dated January 21, 1993 [3.1.3].

The COLR is controlled as a License Basis Document and revised accordingly for each fuel cycle or remaining portion of a fuel cycle. Any revisions to the COLR must be submitted to the NRC for information as required by Tech Spec 5.6.5 and tracked by LCTS 29132. This COLR reports the Cycle 17 core operating and stability limits.

2.0 SCOPE

As defined in Technical Specification 1.1, the COLR is the GGNS document that provides the core operating limits for the current fuel cycle. This document is prepared in accordance with Technical Specification 5.6.5 for each reload cycle using NRC-approved analytical methods.

The Cycle 17 core operating and stability limits included in this report are:

- the Average Planar Linear Heat Generation Rate (APLHGR),
- the Minimum Critical Power Ratio (MCPR) (including EOC-RPT inoperable),
- the Linear Heat Generation Rate (LHGR) limit, and
- the E1A stability limits.

3.0 REFERENCES

This section contains the background, cycle-specific, and methodology references used in the safety analysis of Grand Gulf Cycle 17.

3.1 Background References

3.1.1 MAEC-88/0313, Generic Letter 88-16, "Removal of Cycle-Specific Parameter Limits from Technical Specifications", October 4, 1988.

3.1.2 GNRO-92-00093, Proposed Amendment to Grand Gulf Operating License, PCOL-92/07, dated October 29, 1992.

3.1.3 GNRI-93-0008, Amendment 106 to Grand Gulf Operating License, January 21, 1993.

3.1.4 GEXI 2000-00116, K.V. Walters to J.B. Lee, "Technical Specification and COLR References for Grand Gulf Nuclear Station and River Bend Station," November 3, 2000.

3.2 Current Cycle References

3.2.1 ECH-NE-08-00029 Revision 1, Supplemental Reload Licensing Rport for Grand Gulf Nuclear Station Reload 16 Cycle 17, dated October 2008.

3.2.2 ECH-NE-08-00031 Revision 1, GE14 Fuel Design Cycle-Independent Analyses for Entergy Grand Gulf Nuclear Station, dated October 2008.

3.2.3 ECH-NE-08-00030 Revision 1, Fuel Bundle Information Report for Grand Gulf Nuclear Station Reload 16 Cycle 17, dated October 2008.

3.2.4 NEDC-32910P, Revision 1, Grand Gulf Nuclear Station SAFER/GESTR-LOCA Accident Analysis With Relaxed ECCS Parameters, dated October 1999.

3.2.5 GGNS-SA-08-00005 Revision 0, Grand Gulf Nuclear Station ATRIUM-10 ECCS-LOCA Evaluation, dated July 2008.

3.2.6 GGNS-SA-08-00006 Revision 0, Grand Gulf Nuclear Station GE14 ECCS-LOCA Evaluation, dated July 2008.

3.2.7 CEO 2000-00094, Jim Head to M.D. Withrow, "Revised E1A Related COLR Input," dated April 20, 2000.

3.3 Methodology References

The Technical Specifications (TS) supported by each methodology reference are provided in brackets.

3.3.1 XN-NF-81-58(P)(A) Revision 2 and Supplements 1 and 2, "RODEX2 Fuel Rod Thermal-Mechanical Response Evaluation Model," Exxon Nuclear Company, March 1984 [TS 3.2.1, TS 3.2.2, TS 3.2.3].

3.3.2 XN-NF-85-67(P)(A) Revision 1, "Generic Mechanical Design for Exxon Nuclear Jet Pump BWR Reload Fuel," Exxon Nuclear Company, September 1986 [TS 3.2.3].

3.3.3 EMF-85-74(P) Revision 0 Supplement 1 (P)(A) and Supplement 2 (P)(A), "RODEX2A (BWR) Fuel Rod Thermal-Mechanical Evaluation Model, Siemens Power Corporation," February 1998 [TS 3.2.3].

3.3.4 ANF-89-98(P)(A) Revision 1 and Supplement 1, "Generic Mechanical Design Criteria for BWR Fuel Designs," Advanced Nuclear Fuels Corporation, May 1995 [TS 3.2.3].

3.3.5 Deleted

3.3.6 XN-NF-80-19(P)(A) Volume 1 and Supplements 1 and 2, "Exxon Nuclear Methodology for Boiling Water Reactors – Neutronic Methods for Design and Analysis, Exxon Nuclear Company," March 1983 [TS 3.2.1, TS 3.2.2, TS 3.2.3].

3.3.7 XN-NF-80-19(P)(A) Volume 4 Revision 1, "Exxon Nuclear Methodology for Boiling Water Reactors: Application of the ENC Methodology to BWR Reloads, Exxon Nuclear Company," June 1986 [TS 3.2.1, TS 3.2.2, TS 3.2.3].

3.3.8 EMF-2158(P)(A) Revision 0, "Siemens Power Corporation Methodology for Boiling Water Reactors: Evaluation and Validation of CASMO-MICROBURN-B2, Siemens Power Corporation," October 1999 [TS 3.2.2, TS 3.2.3] .

3.3.9 XN-NF-80-19(P)(A) Volume 3 Revision 2, "Exxon Nuclear Methodology for Boiling Water Reactors, THERMEX: Thermal Limits Methodology Summary Description," Exxon Nuclear Company, January 1987 [TS 3.2.2].

3.3.10 XN-NF-84-105(P)(A), Volume 1 and Supplements 1 and 2, "XCOBRA-T: A Computer Code for BWR Transient Thermal Hydraulic Core Analysis," Exxon Nuclear Company, February 1987 [TS 3.2.2].

3.3.11 ANF-524(P)(A) Revision 2 and Supplements 1 and 2, "ANF Critical Power Methodology for Boiling Water Reactors," Advanced Nuclear Fuels Corporation, November 1990 [TS 3.2.2].

3.3.12 ANF-913 (P)(A), Volume 1, Revision 1 and Volume 1 Supplements 2, 3 and 4, "COTRANSA2: A Computer Program for Boiling Water Reactor Transient Analyses," Advanced Nuclear Fuels Corporation, August 1990 [TS 3.2.2].

3.3.13 XN-NF-825(P)(A) Supplement 2, "BWR/6 Generic Rod Withdrawal Error Analysis, $MCPR_p$ for Plant Operation Within the Extended Operating Domain," Exxon Nuclear Company, October 1986 [TS 3.2.2].

3.3.14 ANF-1358(P)(A) Revision 3, "The Loss of Feedwater Heating Transient in Boiling Water Reactors," Framatome ANP, September 2005 [TS 3.2.2].

3.3 Methodology References (continued)

3.3.15 EMF-1997(P)(A) Revision 0, "ANFB-10 Critical Power Correlation," Siemens Power Corporation, July 1998 [TS 3.2.2].

3.3.16 EMF-1997(P), Supplement 1(P)(A), Revision 0, "ANFB-10 Critical Power Correlation: High Local Peaking Results, Siemens Power Corporation," July 1998 [TS 3.2.2].

3.3.17 EMF-2209(P)(A) Revision 2, "SPCB Critical Power Correlation, Siemens Power Corporation," September 2003 [TS 3.2.2].

3.3.18 EMF-2245(P)(A) Revision 0, "Application of Siemens Power Corporation's Critical Power Correlations to Co-Resident Fuel," Siemens Power Corporation, August 2000 [TS 3.2.2].

3.3.19 EMF-2361 (P)(A) Revision 0, "EXEM BWR-2000 ECCS Evaluation Model," Framatome ANP Richland, Inc., May 2001 [TS 3.2.1].

3.3.20 Deleted

3.3.21 Deleted

3.3.22 NEDC-33383P, Revision 1, "GEXL97 Correlation Applicable to ATRIUM-10 Fuel," June, 2008 [TS 3.2.2].

3.3.23 EMF-2292(P)(A) Revision 0, "ATRIUM-10: Appendix K Spray Heat Transfer Coefficients, Siemens Power Corporation," September 2000 [TS 3.2.1].

3.3.24 EMF-CC-074(P)(A) Volume 4 Revision 0, "BWR Stability Analysis-Assessment of STAIF with Input from MICROBURN-B2," Siemens Power Corporation, August 2000 [TS 3.2.4].

3.3.25* NEDE-24011-P-A, General Electric Standard Application for Reactor Fuel (GESTAR-II) [TS 3.2.1, TS 3.2.2, TS 3.2.3].

* Note: These references are applicable when GE fuel is in the reactor.

4.0 DEFINITIONS

4.1 Average Planar Linear Heat Generation Rate (APLHGR) - the APLHGR shall be applicable to a specific planar height and is equal to the sum of the linear heat generation rates for all the fuel rods in the specified bundle at the specified height divided by the number of fuel rods in the fuel bundle.

4.2 Average Planar Exposure - the Average Planar Exposure shall be applicable to a specific planar height and is equal to the sum of the exposure of all the fuel rods in the specified bundle at the specified height divided by the number of fuel rods in the fuel bundle.

4.3 Critical Power Ratio (CPR) - the ratio of that power in the assembly, which is calculated by application of the fuel vendor's appropriate boiling correlation, to cause some point in the assembly to experience boiling transition, divided by the actual assembly operating power.

4.4 Core Operating Limits Report (COLR) - The Grand Gulf Nuclear Station specific document that provides core operating limits for the current reload cycle in accordance with Technical Specification 5.6.5.

4.5 Linear Heat Generation Rate (LHGR) - the LHGR shall be the heat generation per unit length of fuel rod. It is the integral of the heat flux over the heat transfer area associated with the unit length.

4.6 Minimum Critical Power Ratio (MCPR) - the MCPR shall be the smallest CPR which exists in the core.

4.7 MCPR Safety Limit - the minimum value of the CPR at which the fuel could be operated with the expected number of rods in boiling transition not exceeding 0.1% of the fuel rods in the core.

4.8 Aligned Drive Flow - Adjusted FCTR card input drive flow signal that accounts for actual variations in the core flow to drive flow relationship.

4.9 Monitored Region - The area of the core power and flow operating domain where the reactor may be susceptible to reactor instabilities under conditions exceeding the licensing basis of the current reactor system.

4.10 Restricted Region - The area of the core power and flow operating domain where the reactor is susceptible to reactor instabilities in the absence of restrictions on core void distributions.

4.11 Setpoint "Setup" - A FCTR card feature that sets the normal "non-setup" E1A APRM flow-biased scram and control rod block trip reference setpoints associated with the Exclusion and Restricted Regions higher to permit required reactor maneuvering in the Restricted Region when stability controls are in effect.

4.12 Middle of Cycle (MOC) - The Cycle 17 MOC Core Average Exposure (CAE) is 26,019 MWd/ST [3.2.1].

4.13 End of Cycle (EOC) - The Cycle 17 EOC CAE is 29,750 MWd/ST [3.2.1].

5.0 GENERAL REQUIREMENTS

5.1 Average Planar Linear Heat Generation Rates

Consistent with Technical Specification 3.2.1, all APLHGRs shall not exceed the exposure-dependent limits reported in Figures 1-1 and 1-2 [3.2.1].

5.2 Minimum Critical Power Ratio

Consistent with Technical Specification 3.2.2, the MCPR shall be equal to or greater than the limits reported in Figure(s) 2 as functions of power, flow, and exposure [3.2.1,3.2.2].

Additional MCPR operating limits are provided to support operation with EOC-RPT inoperable as described in Technical Specification 3.3.4.1.

5.3 Linear Heat Generation Rate

Consistent with Technical Specification 3.2.3, the LHGRs for any GE14 rod at any axial location shall not exceed the nodal exposure-dependent limits reported in Reference 3.2.3 multiplied by the smaller of either the power-dependent or flow-dependent LHGR factors reported in Figures 3-2a and 3-3, respectively [3.2.2].

The LHGR for ATRIUM-10 bundles shall not exceed the exposure-dependent limits reported in Figure 3-1 multiplied by the smaller of either the power-dependent or flow-dependent LHGR factors reported in Figures 3-2b and 3-3, respectively [3.2.2].

5.4 Stability

The stability regions and allowable values specified in Technical Specifications are reported in Figure(s) 4 [3.2.7].

5.5 Applicability

The following core operating limits are applicable for operation in the Maximum Extended Operating Domain (MEOD), with Feedwater Heaters Out of Service (FHOOS), and EOC-RPT inoperable. For operation with EOC-RPT inoperable, the alternate MCPR limits described in Section 5.2 above must be implemented. Since the maximum licensed GGNS feedwater temperature reduction is 50 °F at rated power operation, an alternate set of stability limits is not required. For single-loop operation (SLO), the following additional requirements must be satisfied.

1. A SLO multiplier on LHGR and MAPLHGR of 0.83 is required [3.2.1].

2. The MCPR shall be equal to or greater than the limits determined in accordance with Section 5.2 above increased by 0.02 to account for the difference between the two-loop and single-loop MCPR safety limits for the allowable range of single-loop operation [3.2.1].

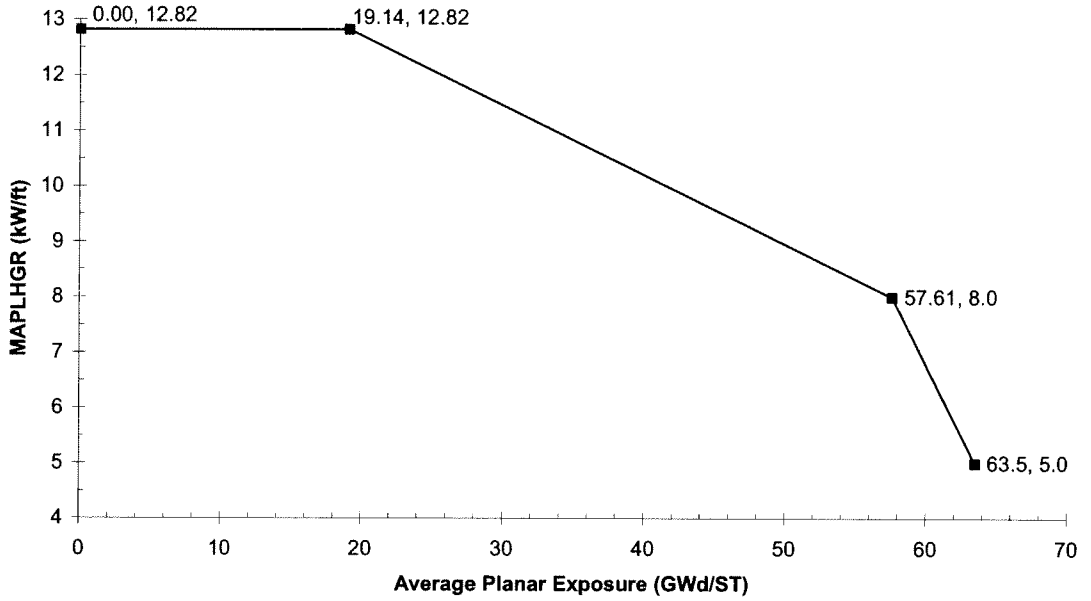

Figure 1-1
Maximum Average Planar Linear Heat generation Rate for GE14
Note: Actual Limits described in Sections 5.1 and 5.5

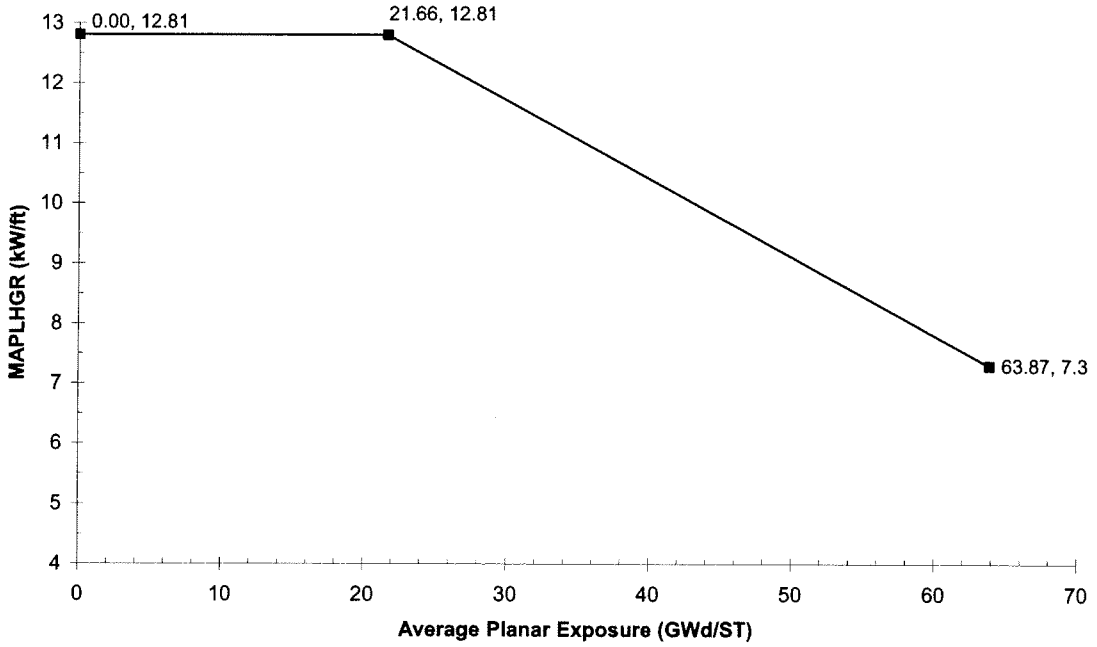

Figure 1-2
Maximum Average Planar Linear Heat generation Rate for ATRIUM-10
Note: Actual Limits described in Sections 5.1 and 5.5

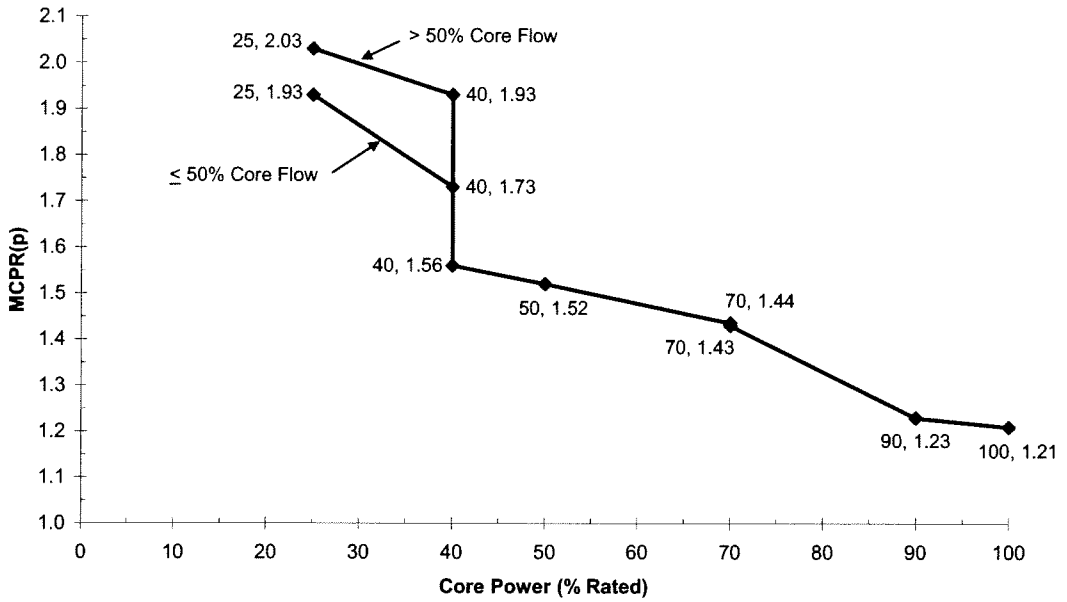

Figure 2-1a
Cycle 17 Power-Dependent MCPR Limits for GE14
BOC to MOC

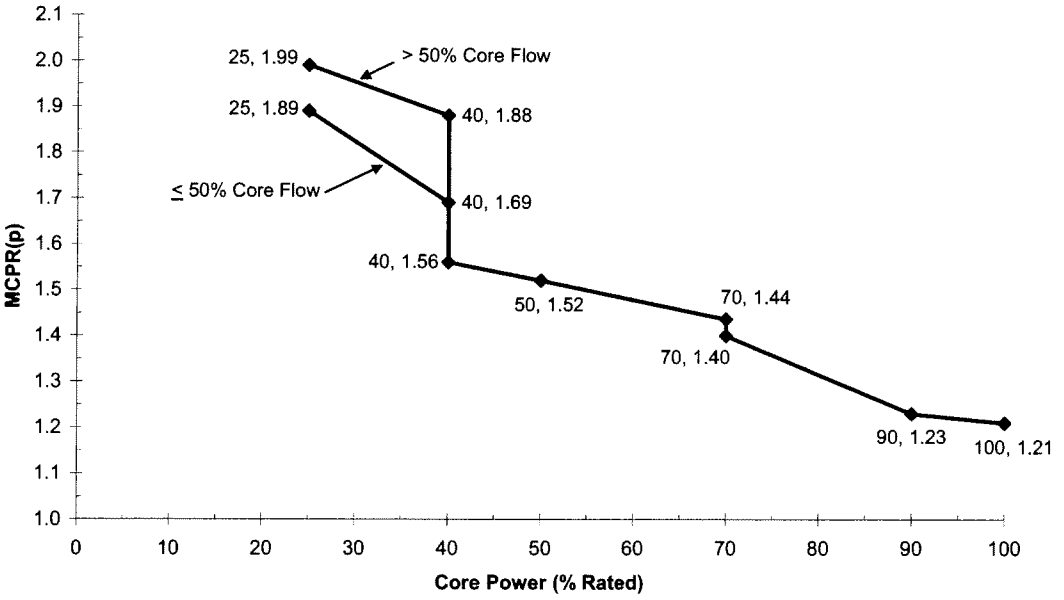

Figure 2-1b
Cycle 17 Power-Dependent MCPR Limits for ATRIUM-10
BOC to MOC

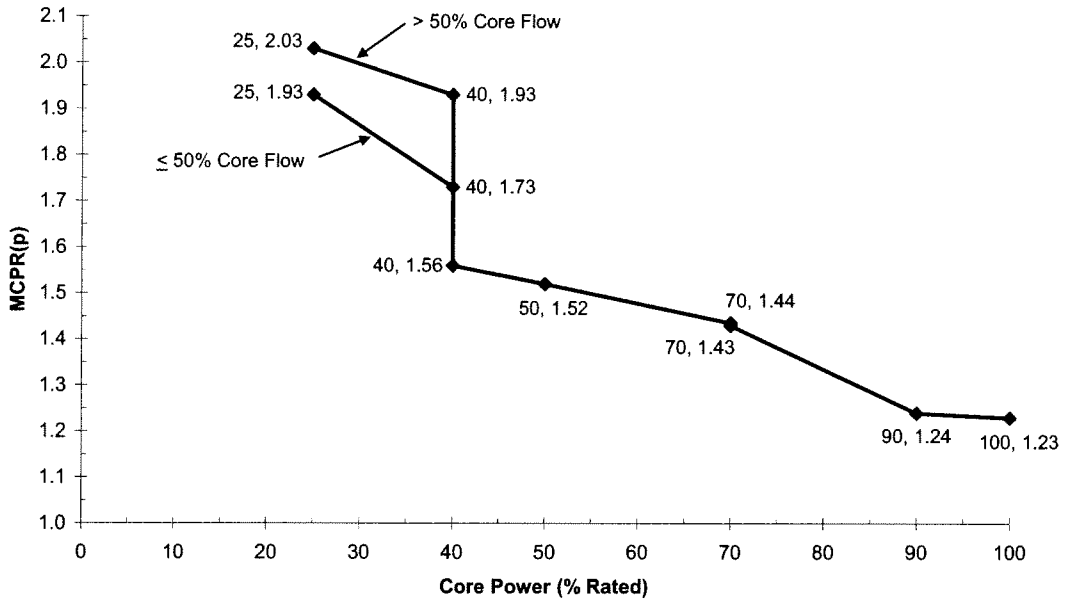

Figure 2-2a
Cycle 17 Power-Dependent MCPR Limits for GE14
BOC to MOC with EOC-RPT Inoperable

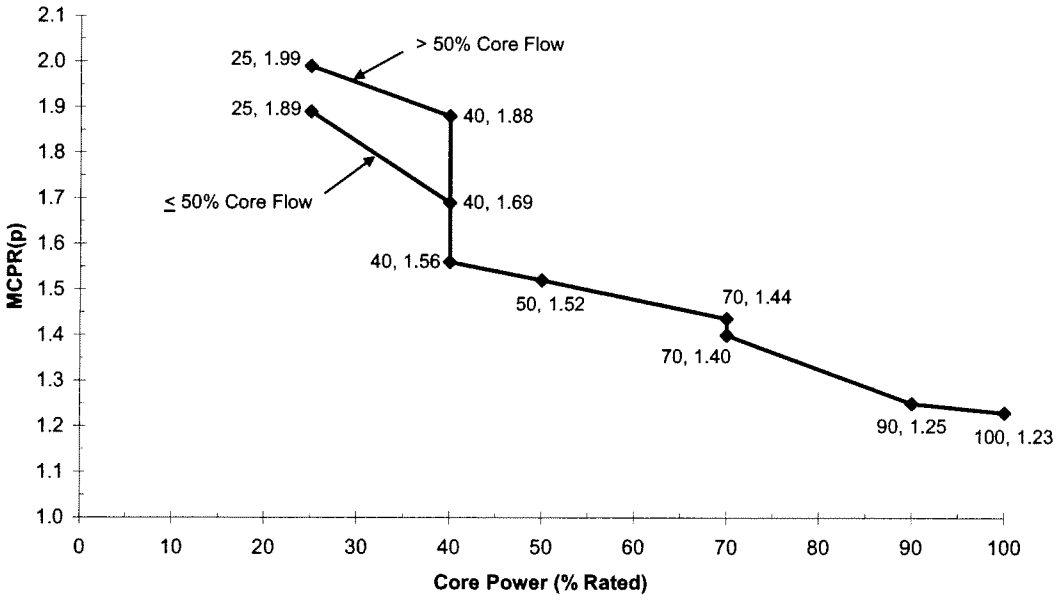

Figure 2-2b
Cycle 17 Power-Dependent MCPR Limits for ATRIUM-10
BOC to MOC with EOC-RPT Inoperable

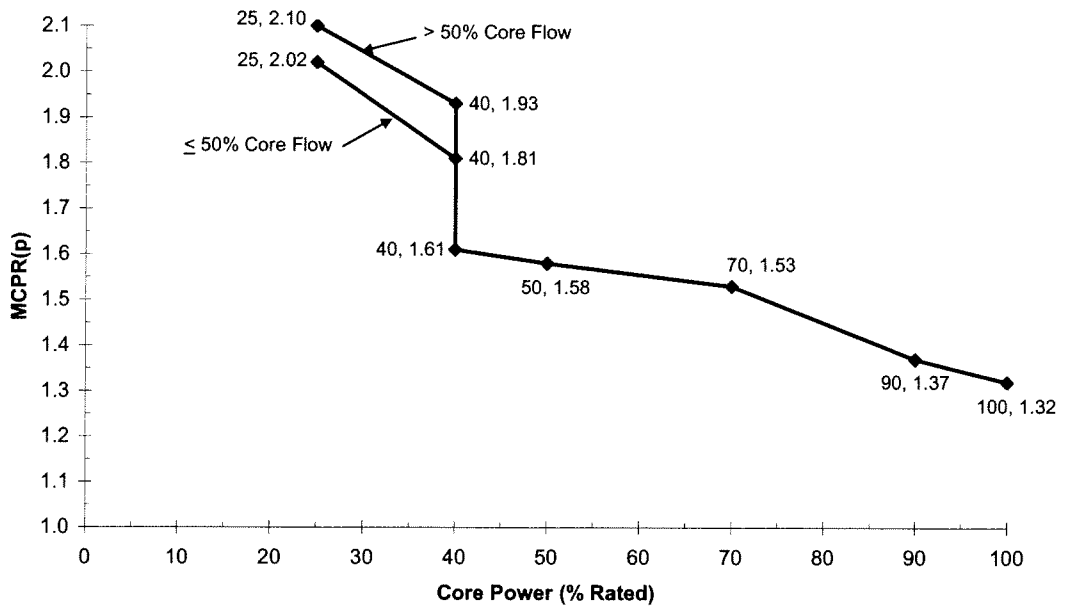

Figure 2-3a
Cycle 17 Power-Dependent MCPR Limits for GE14
MOC to EOC

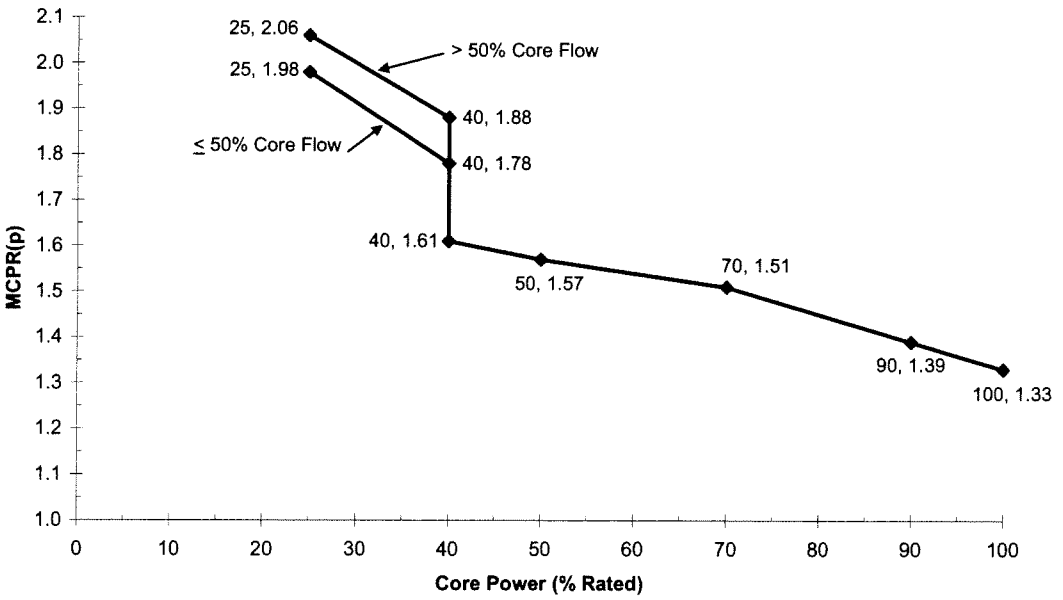

Figure 2-3b
Cycle 17 Power-Dependent MCPR Limits for ATRIUM-10
MOC to EOC

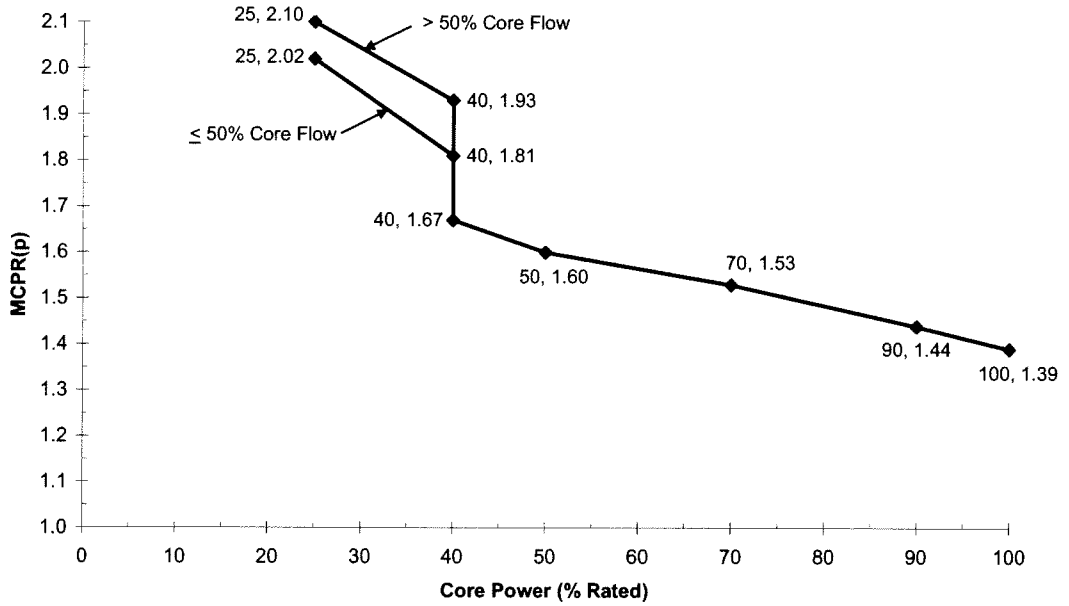

Figure 2-4a
Cycle 17 Power-Dependent MCPR Limits for GE14
MOC to EOC with EOC-RPT Inoperable

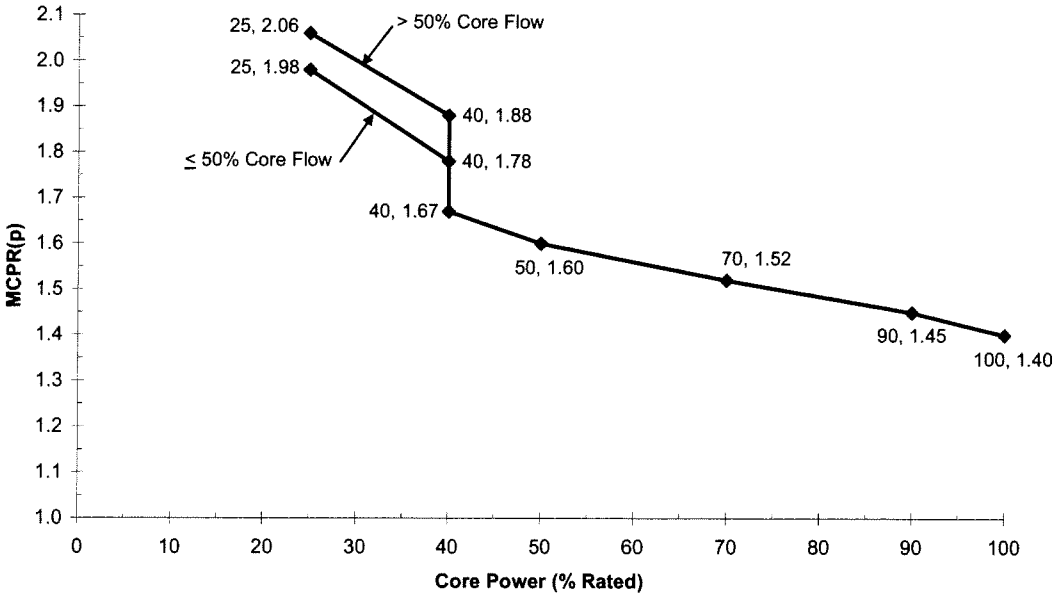

Figure 2-4b
Cycle 17 Power-Dependent MCPR Limits for ATRIUM-10
MOC to EOC with EOC-RPT Inoperable

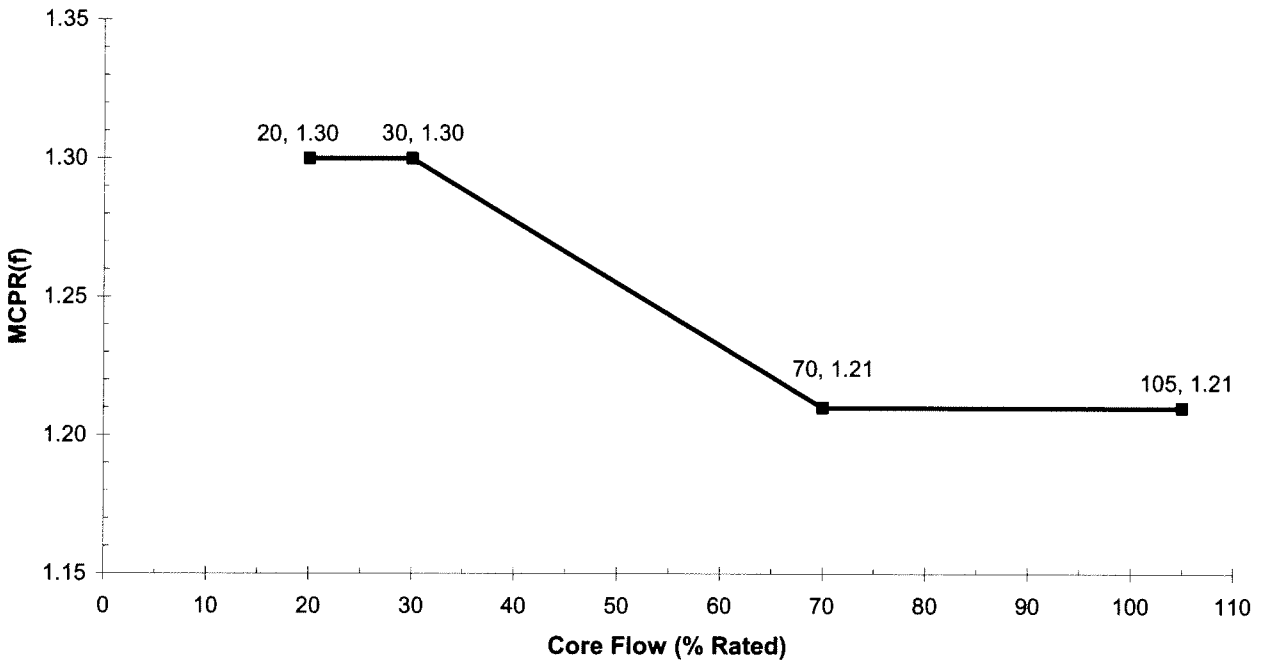

Figure 2-5
Cycle 17 Flow-Dependent MCPR Limits for GE14 and ATRIUM-10

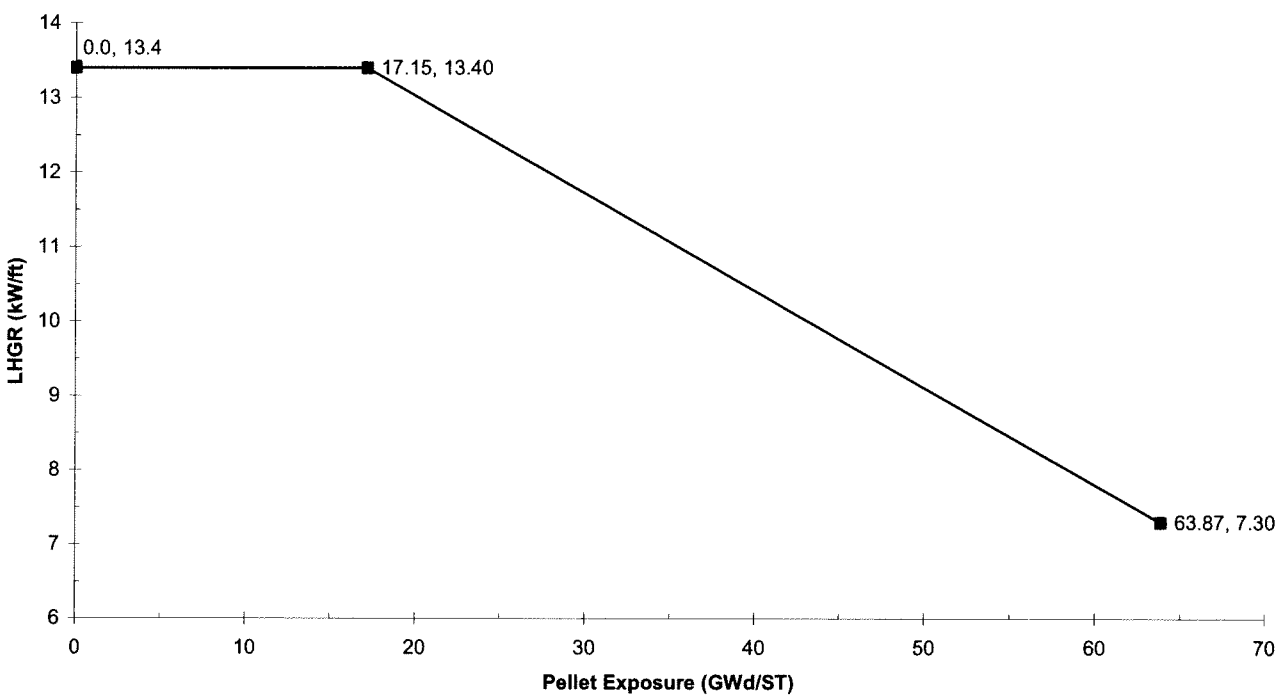

Figure 3-1
Maximum Linear Heat Generation Rate for ATRIUM-10
Note: Actual limits described in Sections 5.3 and 5.5

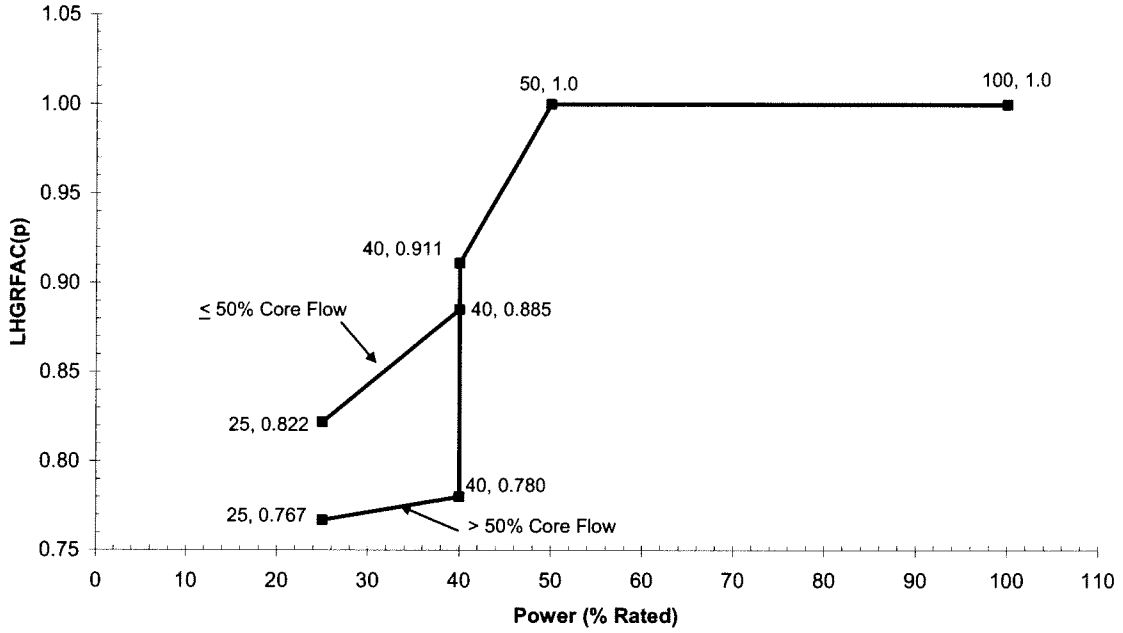

Figure 3-2a
Cycle 17 Power-Dependent LHGR Factor BOC-EOC for GE14
Note: These factor to be applied to the exposure-dependent limits as described in Section 5.3

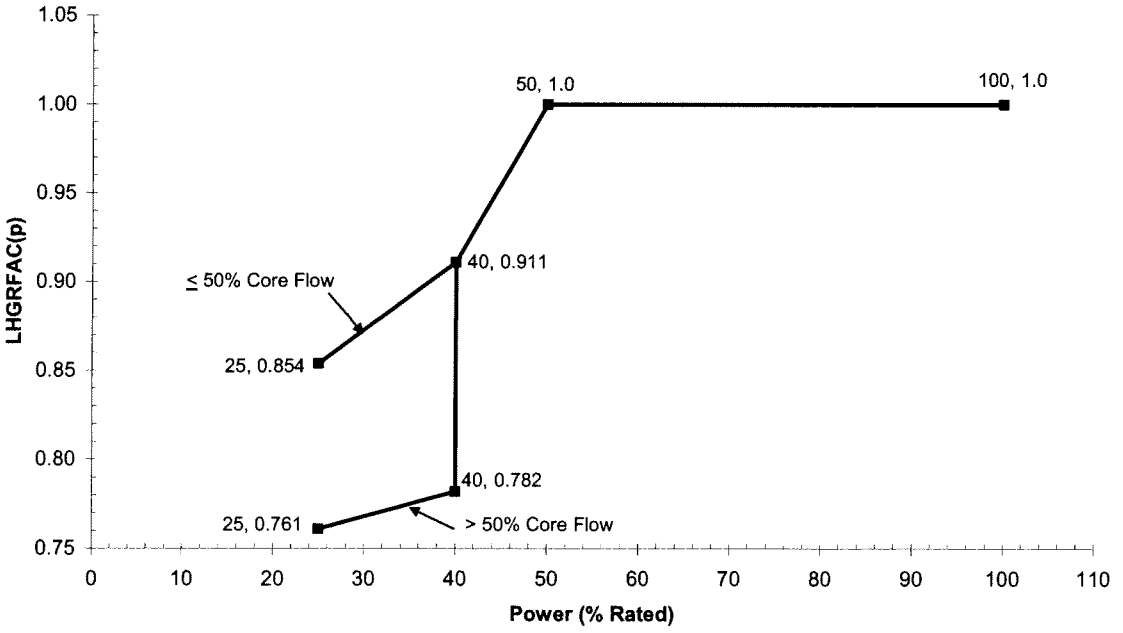

Figure 3-2b
Cycle 17 Power-Dependent LHGR Factor BOC-EOC for ATRIUM-10
Note: These factor to be applied to the exposure-dependent limits as described in Section 5.3

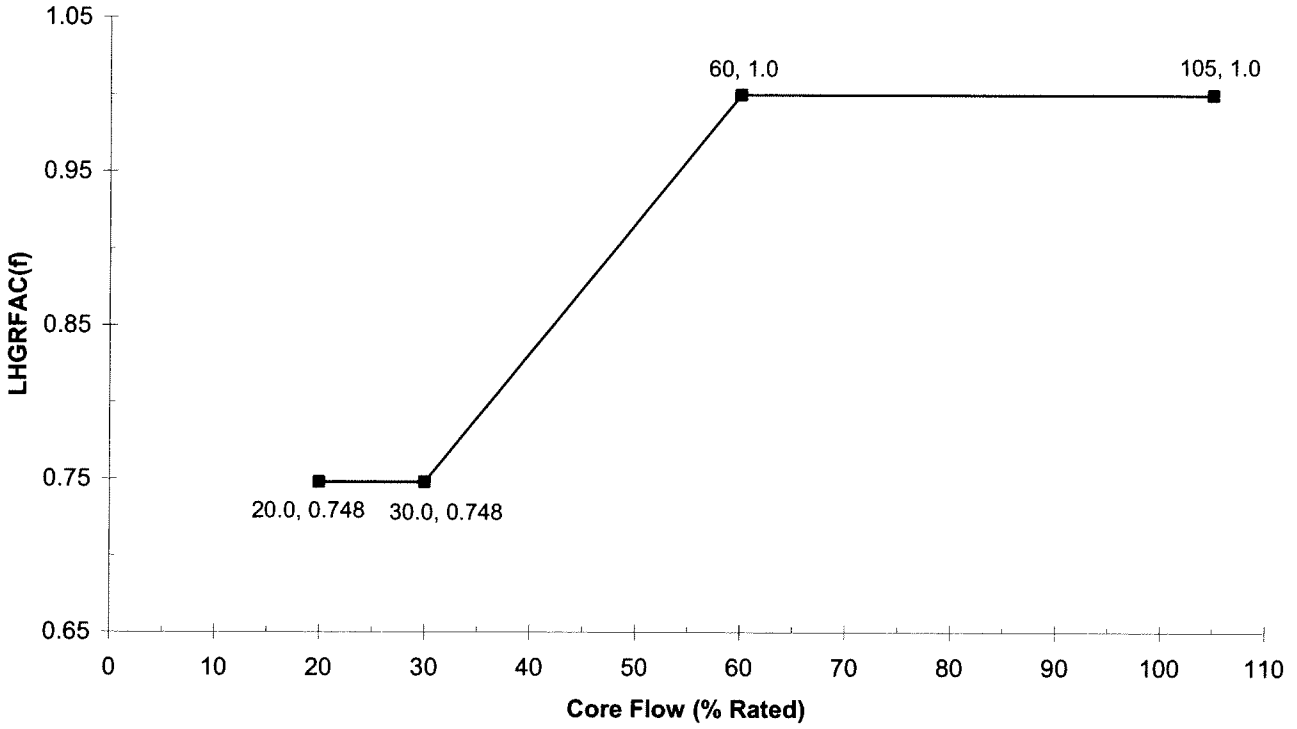

Figure 3-3
Cycle 17 Flow-Dependent LHGR Factor for GE14 and ATRIUM-10
Note: These factor to be applied to the exposure-dependent limits as descibed in Section 5.3

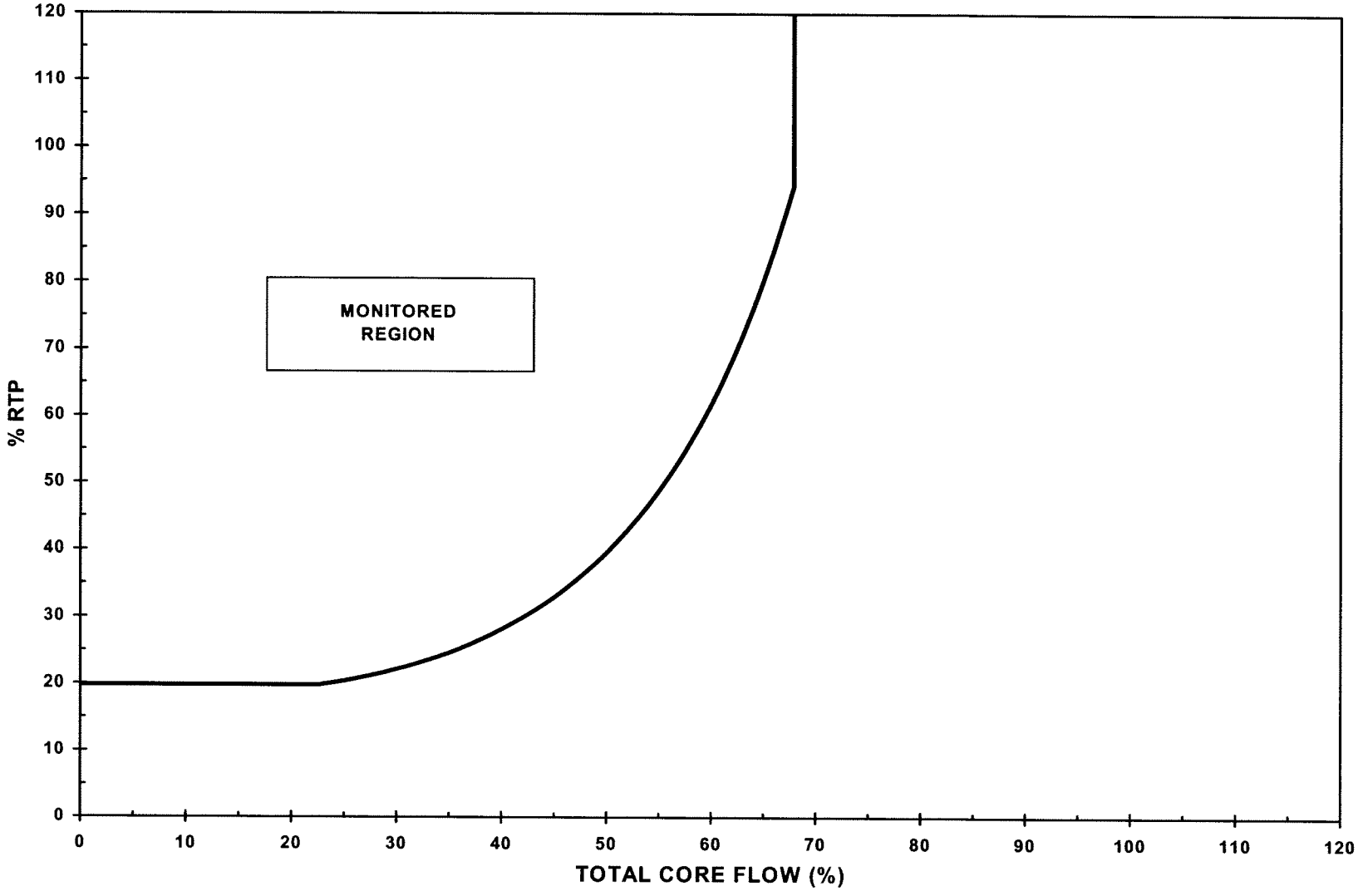

Figure 4-1 Monitored Region Boundary

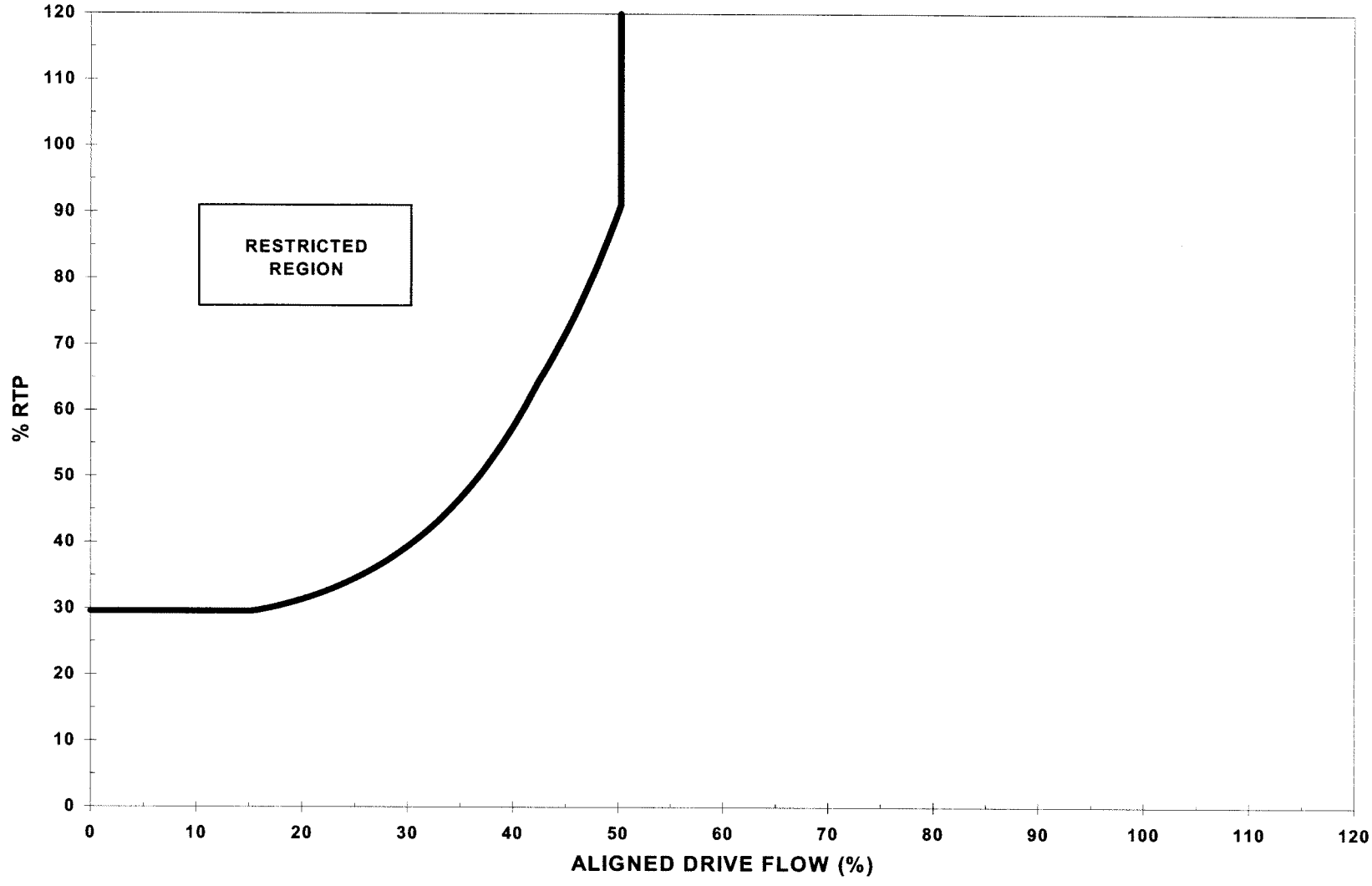

Figure 4-2 Restricted Region Boundary for Two-Loop Operation

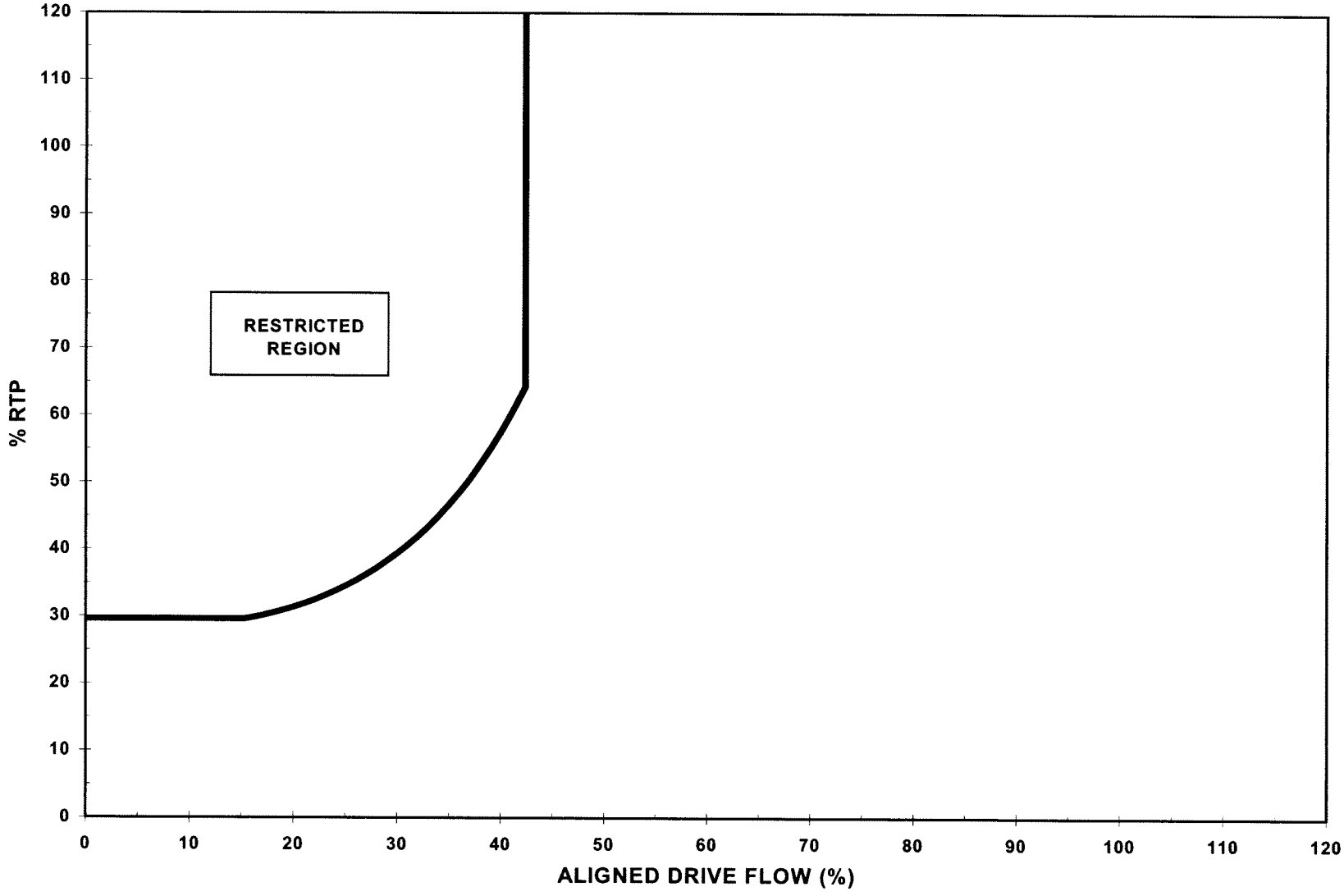

Figure 4-3 Restricted Region Boundary for Single-Loop Operation

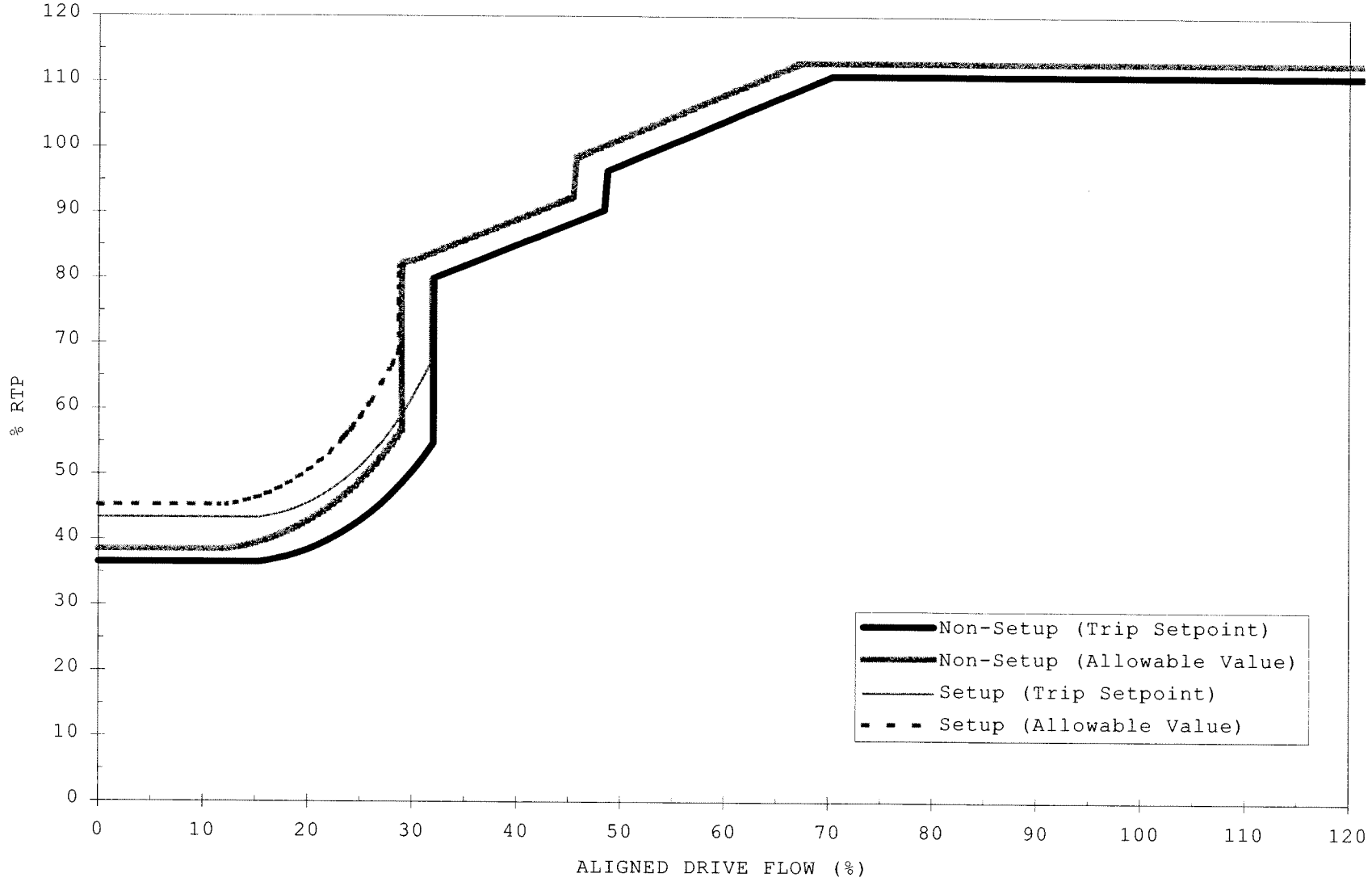

Figure 4-4 APRM Flow-Biased Simulated Thermal Power - High Scram Allowable Values for Two-Loop Operation

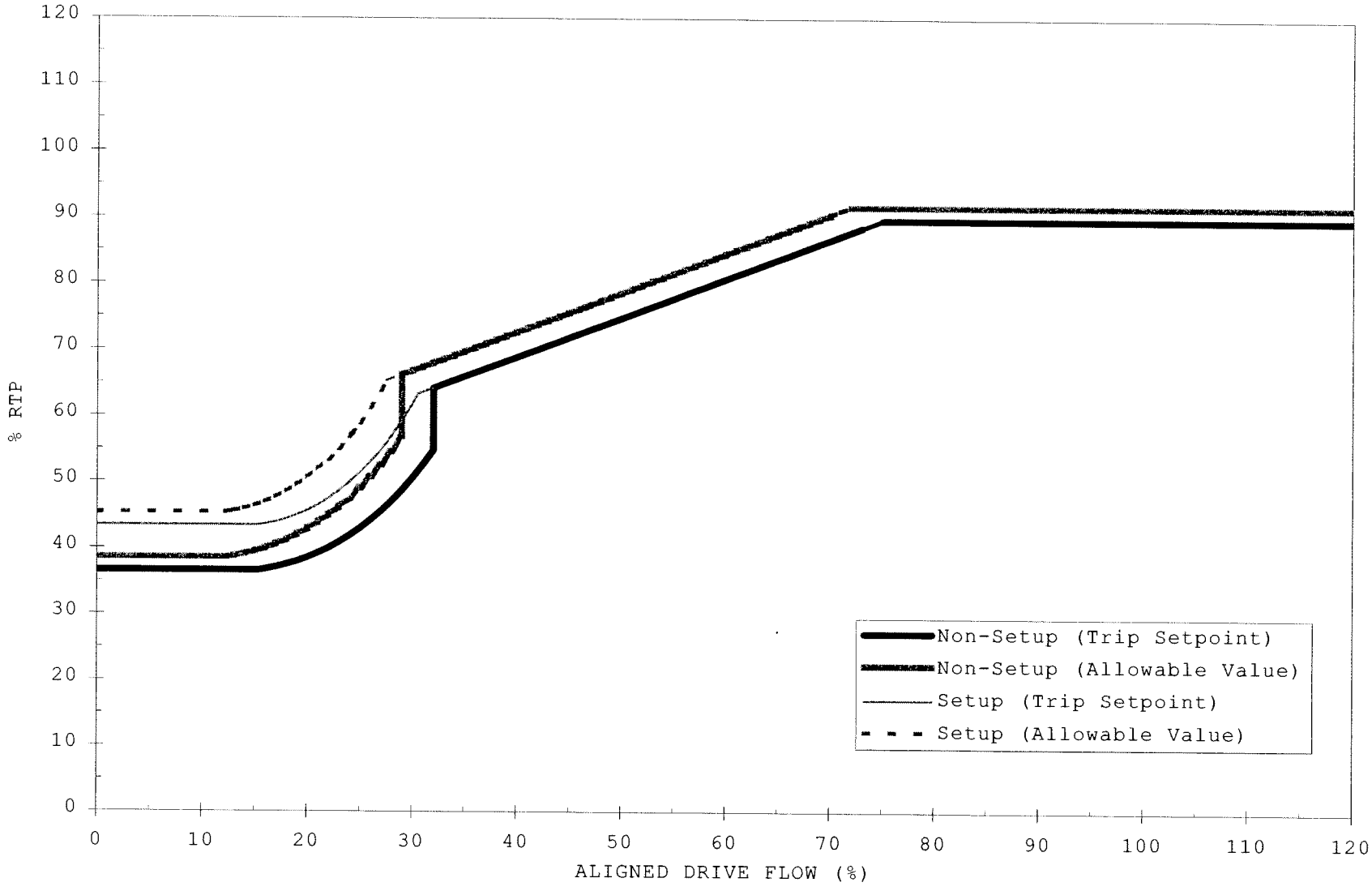

Figure 4-5 APRM Flow-Biased Simulated Thermal Power - High Scram Allowable Values for Single-Loop Operation