



**UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, DC 20555 - 0001**

October 13, 2008

MEMORANDUM TO: Sherry Meador, Technical Secretary
Advisory Committee on Reactor Safeguards

FROM: Cayetano Santos, Chief */RA/*
Reactor Safety Branch
Advisory Committee on Reactor Safeguards

SUBJECT: MINUTES OF THE 555th MEETING OF THE ADVISORY
COMMITTEE ON REACTOR SAFEGUARDS (ACRS),
September 4-6, 2008

I certify that based on my review of the minutes from the 555th ACRS Full Committee meeting, and to the best of my knowledge and belief, I have observed no substantive errors or omissions in the record of this proceeding subject to the comments noted below.

OFFICE	ACRS	ACRS:RSB
NAME	SMeador	CSantos/sam
DATE	10/ 13 /08	06/ 13 /08

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Date Certified: 10/13/2008

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- I. *Federal Register Notice*
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During its 555th meeting, September 4-6, 2008, the Advisory Committee on Reactor Safeguards (ACRS) discussed several matters and completed the following reports, letters, and memoranda

REPORTS

Reports to Dale E. Klein, Chairman, NRC, from William J. Shack, Chairman, ACRS:

- Development of the TRACE Thermal-Hydraulic System Analysis Code, dated September 24, 2008
- Report on the Safety Aspects of the License Renewal Application for the Wolf Creek Generating Station, Unit 1, dated September 17, 2008

MEMORANDA

Memoranda to R. W. Borchardt, Executive Director for Operations, NRC, from Edwin M. Hackett, Executive Director, ACRS:

- Draft Final Revision to Regulatory Guides 10.7, 10.8, and 10.9, dated September 9, 2008
- Draft Regulatory Guides DG-1205, DG-1187, DG-1197, DG-1196, DG-3031, DG-0020, DG-5026, DG-1141, DG-3035, DG-5027, and DG-1203, dated September 9, 2008
- Proposed Interim Staff Guidance (ISG) DC/COL-ISG-06, dated September 9, 2008
- Withdrawal of Regulatory Guide (RG) 8.1, "Radiation Symbol," dated September 9, 2008

MINUTES OF THE 555th MEETING OF THE
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
September 4-6, 2008
ROCKVILLE, MARYLAND

The 555th meeting of the Advisory Committee on Reactor Safeguards (ACRS) was held in Conference Room 2B3, Two White Flint North Building, Rockville, Maryland, on September 4-6, 2008. Notice of this meeting was published in the *Federal Register* on August 22, 2008 (72 FR 49713-49714) (Appendix I). The purpose of this meeting was to discuss and take appropriate action on the items listed in the meeting schedule and outline (Appendix II). The meeting was open to public attendance.

A transcript of selected portions of the meeting is available in the NRC's Public Document Room at One White Flint North, Room 1F-19, 11555 Rockville Pike, Rockville, Maryland. Copies of the transcript are available for purchase from Neal R. Gross and Co., Inc., 1323 Rhode Island Avenue, NW, Washington, DC 20005. Transcripts are also available at no cost to download from, or review on, the Internet at <http://www.nrc.gov/ACRS/ACNW>.

ATTENDEES

ACRS Members: Dr. William J. Shack (Chairman), Dr. Mario V. Bonaca (Vice-Chairman), Dr. Said Abdel-Khalik (Member-at-Large), Dr. George E. Apostolakis, Dr. Sam Armijo, Dr. Sanjoy Banerjee, Dr. Dennis Bley, Mr. Charles Brown, Dr. Michael Corradini, Mr. Otto L. Maynard, Dr. Dana A. Powers, Mr. Harold Ray, Dr. Michael Ryan, and Mr. John Sieber. Mr. John Stetkar did not attend the meeting. For a list of other attendees, see Appendix III.

I. Chairman's Report (Open)

[Note: Mr. Sam Duraiswamy was the Designated Federal Official for this portion of the meeting.]

Dr. William J. Shack, Committee Chairman, convened the meeting at 8:30 a.m. In his opening remarks he announced that the meeting was being conducted in accordance with the provisions of the Federal Advisory Committee Act. He reviewed the agenda items for discussion and noted that no written comments or requests for time to make oral statements from members of the public had been received. Dr. Shack also noted that a transcript of the open portions of the meeting was being kept and speakers were requested to identify themselves and speak with clarity and volume. Dr. Shack announced that Dr. Edwin Hackett will replace Mr. Frank Gillespie as the ACRS Executive Director. Also, two employees joined the ACRS staff, Natalie Mitchell-Funderburk and Banu Goldfeiz.

II. License Renewal Application and Final SER for the Wolf Creek Generating Station, Unit 1

[Note: Mr. Peter Wen was the Designated Federal Official for this portion of the meeting.]

The Committee met with representatives of the Wolf Creek Nuclear Operating Corporation (WCNOC) (the applicant) and the NRC staff to discuss the license renewal application (LRA) for the Wolf Creek Generating Station (WCGS) and the associated NRC staff's final Safety Evaluation Report (SER). The operating license for WCGS expires on March 11, 2025. The applicant has requested approval for continued operation for a period of 20 years beyond the current license expiration date.

The applicant discussed the resolution of the five open items, of which, two items were related to scoping boundary of station blackout (SBO) recovery paths and the remaining three were related to metal fatigue. For closure of the SBO recovery paths related open items, the applicant submitted an amendment to the LRA by including: (a) a breaker at transmission system voltage on both the East and West switchyard bus and (b) an underground medium voltage switchyard cable. The staff reviewed this amendment and found it to be acceptable. For the metal fatigue issues, the applicant has committed to update the fatigue monitoring program baseline fatigue analyses as follows: (a) for the surge line hot leg nozzle, the applicant will account for the additional insurge and outsurge cycles accumulated in the early years of plant operation, during which thermal cycle counts were not collected in a systematic and rigorous manner, and (b) for the charging nozzles, the applicant will account for differential contribution of fatigue for each category of charging event. Based on the commitments made by the applicant, the staff concludes that the applicant has provided an acceptable basis for managing aging effect of environmentally assisted metal fatigue of surge line hot leg nozzle and charging nozzles in accordance with 10 CFR 54.21(c)(1)(iii).

The staff described its review and inspection of the applicant's scoping, screening, and aging management programs; the program implementation at WCGS; and resolution of the open items. The staff concluded that the requirement of 10 CFR 54.29(a) has been met.

The Committee issued a report to the NRC Chairman on this matter, dated **September 17, 2008**. The Committee concluded that the programs established by the applicant to manage age-related degradation provide reasonable assurance that the WCGS can be operated in accordance with the current licensing basis for the period of extended operation without undue risk to the health and safety of the public. The Committee recommended that the WCNOC application for renewal of the operating license for WCGS should be approved.

III. Draft Final Revision 1 to Regulatory Guide 1.131, "Qualification of Safety-Related Cables and Field Splices for Nuclear Power Plants"

[Note: Ms. Christina Antonescu was the Designated Federal Office for this portion of the meeting.]

The Committee met with representatives of the NRC staff to discuss Revision 1 of Draft Final Regulatory Guide 1.131, "Qualification of Safety-Related Cables and Field Splices for Nuclear Power Plants." The staff noted that the final Regulatory Guide will be issued as a new regulatory guide bearing the number 1.211.

This Guide endorses IEEE Standard 383-2003, "Standard for Qualifying Class 1E Electric Cables and Field Splices for Nuclear Power Generating Stations," with some minor clarifications and/or exceptions. This Guide describes a method that the NRC staff considers acceptable for complying with the Commission's regulations for the qualification of safety-related cables and field splices for nuclear power plants.

Some Committee members suggested that clarifications be added in the Guide regarding the definition of risk-significant safety-related equipment (e.g. cable). Also, it should be made clear: that (a) the scope of the Guide is limited to the safety-related cables; (b) the cables under Appendix R "Fire Protection Program for Nuclear Power Facilities" are not within the scope of this Guide; and (c) that there is a need for testing specialty cables with connectors.

In addition, some members noted that the last Regulatory Position in this Guide does not offer any specific condition monitoring techniques to assess physical and operating conditions of the cable. It only mentions that some condition monitoring should be incorporated. The requirement for condition monitoring is being imposed without any condition monitoring techniques being endorsed by RG 1.131, Rev. 1. This introduction of cable monitoring establishes a requirement for testing with no defined test methodology or acceptance criteria. Also, the members expressed concern about the lack of adequate time to review the changes made to this Guide the day before the ACRS meeting.

A representative from the industry focus group on equipment qualification also made oral remarks at the meeting. The nuclear industry generally supports the proposed draft final Regulatory Guide except it opposes the requirements for condition monitoring of safety-related power, control, instrumentation, and control cables.

The Committee recommended that a revised copy of this Guide, concurred in by other Offices, be provided to the Committee for its review in a future ACRS meeting.

IV. TRACE Computer Code Peer Review

[Note: Mr. David Bessette was the Designated Federal Official for this portion of the meeting.]

The Committee met with representatives of the NRC staff to discuss the development of the TRACE thermal-hydraulic system analysis code and the outcome of the recently completed peer review. The peer reviewers identified no major deficiencies that preclude the use of TRACE for confirmatory analyses of postulated loss-of-coolant accidents (LOCAs) in current light water reactors (LWRs). Several improvements have been recommended by the peer reviewers and the staff has proposed a plan to address them. The Committee agreed with the recommended improvements and endorsed the staff's plan.

The staff noted that TRACE is now being used for performing small-break and large-break LOCA confirmatory analyses for the Browns Ferry Nuclear Plant extended power uprate (EPU). Plant decks are also being prepared to assist EPU reviews of other BWR designs, as well as Westinghouse, Combustion Engineering, and Babcock & Wilcox PWRs. Analyses of anticipated operational occurrences (AOOs) and chimney instabilities for the Economic Simplified Boiling Water Reactor (ESBWR), and assessments of applicability of TRACE to other new reactor designs are in progress.

The Committee issued a report to the NRC Chairman on this matter, dated September 24, 2008. The Committee concluded that significant progress has been made toward the incorporation of TRACE into the regulatory process. The Committee recommended that further peer review be conducted to evaluate the applicability of TRACE to new LWR designs, as well as for analysis of coupled reactor physics-thermal hydraulics issues related to EPU's and expanded operating domains. The Committee also recommended that the capability to evaluate uncertainties in the TRACE code predictions be incorporated into TRACE. The Committee noted that the continued development of TRACE is necessary to keep pace with the evolving industry capabilities.

V. Anticipated Advanced Reactor Research Needs

[Note: Ms. Maitri Banerjee was the Designated Federal Official for this portion of the meeting.]

The Committee members discussed anticipated research needs in the area of advanced reactors and identified items to be discussed during the January 2009 Future Plant Designs Subcommittee and the February 2009 full Committee meetings. The Future Plant Designs Subcommittee Chair presented a chronology of recent developments in the area of advanced reactors and an outline of what should be addressed at these two meetings. The following subjects received considerable discussion: use of PRA to establish the licensing basis events; radiological consequence and source term research; and safety system performance and qualification. Some specific issues, unique to HTGR, will also be discussed at the above meetings. Following Commission guidance, HTGR is the current focus of the NRC advanced reactor research plan. The Committee plans to review the NRC advanced reactor research plan during its February 2009, meeting.

VI. Quality Assessment of Selected Research Projects

[Note: Dr. Hossein Nourbakhsh was the Designated Federal Official for this portion of the meeting.]

The Committee discussed the status of the quality assessment of the research projects on: "Assessment of Predictive Bias and the Influence of Manufacturing, Model, and Power Uncertainties in NRC Fuel Performance Code Predictions," and NUREG/CR - 6943, "A Study of Remote Visual Methods to Detect Cracking in Reactor Components." The Committee discussed the results of panel review and the numerical rating scores for these projects. The Committee plans to complete its report on the quality assessment of the research projects noted above during its October 2-4, 2008, meeting.

VI. Executive Session

[Note: Mr. Edwin M. Hackett was the Designated Federal Official for this portion of the meeting.]

A. Reconciliation of ACRS Comments and Recommendations/EDO Commitments

- The Committee considered the EDO's response of July 8, 2008, to comments and recommendations included in the May 19, 2008, ACRS report concerning the draft NUREG/CR-6962, "Approaches for Using Traditional Probabilistic Risk Assessment Methods for Digital Systems." The Committee decided that it was satisfied with the EDO's response.
- The Committee considered the August 21, 2008, EDO response to the June 3, 2008, ACRS report on Susquehanna extended power uprate application. The EDO response states that the recommended void fraction uncertainty analyses are being performed. The Committee lauds the staff's effort to quantify the impact of void fraction uncertainties on the safety analyses. For any future inquiries or concerns on the Susquehanna EPU application, the EDO proposes that the Committee communicate directly with the Division of Safety Systems. Considering the Committee's mandate is to provide advice to the Commission as an independent advisory body, the Committee disagrees with the EDO's proposal. The ACRS full Committee and Subcommittee meetings are the appropriate forum for holding technical discussions and communications with specific Divisions and Offices of the NRC, including NRR, and other stakeholders.
- The Committee considered the August 27, 2008, EDO response to the July 23, 2008, ACRS report on Millstone Unit 3 stretch power uprate. The Committee decided that it was satisfied with the EDO's response.
- The Committee considered the August 15, 2008, EDO response to comments and recommendations included in the July 21, 2008, ACRS interim letter 4 on Chapter 3 of the NRC Staff's SER related to the ESBWR design. The Committee decided that it was satisfied with the EDO's response.

B. Report of the Planning and Procedures Subcommittee Meeting

Review of the Member Assignments and Priorities for ACRS Reports and Letters for the September ACRS Meeting

Member assignments and priorities for ACRS reports and letters for the September ACRS meeting were discussed. Reports and letters that would benefit from additional consideration at a future ACRS meeting were also discussed.

Anticipated Workload for ACRS Members

The anticipated workloads for ACRS members through November 2008 were discussed and the objectives were:

- Review the reasons for the scheduling of each activity and the expected work product and to make changes, as appropriate
- Manage the members' workload for these meetings
- Plan and schedule items for ACRS discussion of topical and emerging issues

Containment Overpressure Credit Issue

During the July 2008 meeting, the ACRS was briefed by representative of the Tennessee Valley Authority (TVA) regarding the containment overpressure credit issue related to the Browns Ferry Nuclear Plant. During the meeting, members provided feedback on the information presented by TVA. Due to a lack of time, the Committee did not discuss its position on this issue at the July meeting. Consequently, Dr. Bonaca suggested that the Committee discuss this matter during the September meeting and decide on a course of action.

In addition, in the June 26, 2008 Staff Requirements Memorandum (SRM), stemming from the ACRS meeting with the Commission on June 5, 2008, the Commission directed the staff to continue working to address Committee concerns on containment overpressurization and, as necessary and appropriate, provide timely policy decision paper to the Commission to resolve the disagreement. The Committee should discuss this matter and propose a future course of action.

We understand that the NRR staff is preparing White Paper on the Containment Overpressure issue. If it is provided to the ACRS in a timely manner, it will be scheduled for discussion at the October meeting.

Summary Matrix of ACRS Reports and Letters Issued During FY 2008

The ACRS Office should submit to the Commission the annual ACRS Operating Plan and Self-Assessment on October 31, 2008. In accordance with the Commission direction in the August 6, 1999 SRM, a summary matrix of ACRS reports and letters issued during FY-2008 should also be submitted along with the Operating Plan and Self-Assessment. In order to preclude violation of the ACRS Bylaws, the Committee should authorize the ACRS Executive Director and/or his designee to summarize the FY-2008 ACRS reports and letters.

Proposed Regulatory Guides

The staff issued the following Draft Regulatory Guides (DGs) for public comment:

- Proposed Revision 1 to Regulatory Guide 1.47 (DG-1205), "Bypassed and Inoperable Status Indication for Nuclear Power Plant Safety Systems"

Proposed Revision 1 to Regulatory Guide 1.47 (DG-1205) endorses the Institute of Electrical and Electronics Engineers (IEEE) Standard 603-1991, "Criteria for Protection Systems for Nuclear Power Generating Stations," in place of IEEE Standard 279-1971 version. The IEEE Standard 603-1991 is more specific than IEEE Standard 279-1971 regarding bypassed and inoperable status indication.

- Proposed Revision 1 of Regulatory Guide 1.69, (DG-1187), "Concrete Radiation Shields and Generic Shield Testing for Nuclear Power Plants"

Proposed Revision 1 to Regulatory Guide 1.69 (DG-1187) subsumes the provisions of Regulatory Guide 2.1 "Shield Test Program for Evaluation of Installed Biological Shielding in Research and Training Reactors," as the staff plans to withdraw Regulatory Guide 2.1.

- Proposed Revision 2 of Regulatory Guide 1.90, (DG-1197), “Inservice Inspection of Prestressed Concrete Containment Structures with Grouted Tendons”

Proposed Revision 2 to Regulatory Guide 1.90 (DG-1197) cites the appropriate regulations and incorporates the appropriate level of pre-stress and inspection requirements for a 60-year plant life instead of a 40-year plant life. DG-1197 provides two acceptable alternative methods of inspecting containment structures with grouted tendons: (1) an in-service inspection (ISI) program based on monitoring the pre-stress level by means of instrumentation and (2) an ISI program based on pressure-testing the containment structure.

- Proposed Revision 2 of Regulatory Guide 1.107, (DG-1196), “Qualification for Cement Grouting for Prestressing Tendons in Containment Structures”

Proposed Revision 2 to Regulatory Guide 1.107 (DG-1196) cites the appropriate regulations, incorporates the latest technology advances in the American Society of Mechanical Engineers (ASME) code, and identifies an acceptable method to demonstrate that the proposed system (grouting of pre-stressing tendons) will provide a high level of reliability in the design and installation of the system. DG-1196 provides quality standards for using Portland cement grout to protect pre-stressing steel from corrosion.

- Proposed Revision 2 of Regulatory Guide 3.52, (DG-3031), “Standard Format and Content for the Health and Safety Sections of License Renewal Applications for Uranium Processing and Fuel Fabrication”

Proposed Revision 2 to Regulatory Guide 3.52 (DG-3031) endorses the procedure contained in NUREG-1520, “Standard Review Plan for the Review of a License Application for a Fuel Cycle Facility.” While the regulations provide general information for filing license renewal applications, NUREG-1520 identifies the specific information to be submitted by an applicant and evaluated by the staff. This NUREG provides guidance on the information to be included in licensing applications and establishes a format for presenting the information. Using this standard format helps to ensure uniformity and completeness in the preparation of licensing applications.

- Proposed Revision 3 of Regulatory Guide 10.4, (DG-0020), “Guide for the Preparation of Applications for Licenses to Process Source Material”

Proposed Revision 3 to Regulatory Guide 10.4 (DG-0020) endorses the license application and review procedure to process source material as discussed in NUREG-1556, “Consolidated Guidance about Material Licenses.” While the regulations provide general information for filing license renewal applications, NUREG-1556 identifies the specific information to be submitted by an applicant and evaluated by the staff. This NUREG provides guidance on the information to be included in licensing applications and establishes a format for presenting the information. Using this standard format helps to ensure uniformity and completeness in the preparation of licensing applications.

- Proposed new Regulatory Guide DG-5026, "Fatigue Management for Nuclear Power Plant Personnel"

DG-5026 is a proposed new Regulatory Guide developed in support of 10 CFR Part 26. The draft regulatory guide endorses (with modifications) the Nuclear Energy Institute (NEI) document NEI 06-11, Revision E, "Managing Personnel Fatigue at Nuclear Power Reactor Sites," dated June 2008.

- Proposed Revision 4 to Regulatory Guide 1.105 (DG-1141), "Setpoints for Safety-Related Instrumentation,"

Proposed Revision 4 to Regulatory Guide 1.105 (DG-1141) endorses the American National Standard Institute (ANSI)/ Instrumentation, Systems, and Automation Society (ISA) Standard 67.04.01-2006. This standard incorporates the current industry consensus on instrument setpoints, and the information in the Regulatory Information Summary (RIS) 2006-017, which presents the NRC staff position on the requirements of 10 CFR 50.36, "Technical Specifications," regarding limiting safety system settings during periodic testing and calibration of instrument channels.

- Proposed Revision 1 to Regulatory Guide 3.16 (DG-3035), "General Fire Protection Guide for Plutonium Processing and Fuel Fabrication Plants,"

This proposed revision is a complete re-write that refers to NUREG-1718, "Standard Review Plan for the Review of an Application for a Mixed Oxide (MOX) Fuel Fabrication Facility," Chapter 7, "Fire Protection," regarding what information an applicant should provide with their license application.

- Proposed Revision 1 to Regulatory Guide 5.12 (DG-5027), "General Use of Locks in Protection and Control of Facilities and Special Nuclear Materials."

This proposed revision endorses new and revised standards, including several General Services Administration (GSA) Standards such as Federal Specification FF-L-2740A, "Locks, Combination," FF-L-2890A, "Lock Extension (Pedestrian Door, Deadbolt)," and FF-L-2937, Amendment 1, "Combination Lock, Mechanical" as well as several ANSI and ASTM Standards.

- Proposed New Regulatory Guide DG-1203, "Containment Performance for Pressure Loads,"

DG-1203 is a proposed new Regulatory Guide Which describes acceptable methods for demonstrating containment performance in nuclear power plants, in accordance with the regulatory requirements of 10 CFR Part 50, Appendix A, General Design Criteria 16, "Containment Design," and 50, "Containment Design Basis," and 10 CFR 50.44, "Combustible Gas Control for Nuclear Power Reactors."

Proposed Interim Staff Guidance

The staff issued proposed Interim Staff Guidance DC/COL-ISG-06, "Evaluation and Acceptance Criteria for 10 CFR 20.1406," for public comment.

DC/COL-ISG-06 is a new Interim Staff Guide which describes the evaluation and acceptance criteria that will be used by NRC staff in reaching a reasonable assurance finding that a Design Certification (DC) or Combined License (COL) applicant has demonstrated compliance with the regulatory requirements of 10 CFR Part 20.1406, "Minimization of Contamination," in accordance with the guidance provided in Regulatory Guide 4.21, "Minimization of Contamination and Waste Generation: Life Cycle Planning."

Withdrawal of Regulatory Guide 8.1, "Radiation Symbol"

The NRC is withdrawing the Regulatory Guide 8.1, issued in February 1973, because it is no longer required. Regulatory Guide 8.1 references 10 CFR 20.203 which has been deleted. It also endorses ANSI Standard N2.1-1969 which was withdrawn in 1999. The Regulatory Guide is no longer required because the current 10 CFR 20.1901, "Caution Signs," provides a specific description of the approved trefoil radiation symbol and no additional guidance is required.

Draft Final Regulatory Guides

- Draft Final Revision 2 to Regulatory Guide 10.7 (DG0017), "Guide for the Preparation of Applications for Licenses for Laboratory and industrial Use of Small Quantities of Byproduct Material"

This Guide provides guidance on the type of information to be submitted to the staff to evaluate an application for a specific license for laboratories and industries to use small quantities of byproduct material. This Guide endorses the methods and procedures for applying for a license for laboratory and industrial use of small quantities of by product material contained in the current version of NUREG-1556, Volume 7, "Consolidated Guidance About Materials Licenses: Program-Specific Guidance About Academic, Research and Development, and Other Licenses of Limited Scope."

- Draft Final Revision 3 to Regulatory Guide 10.8 (DG018), "Guide for the Preparation of Applications for Medical Use Programs"

This Guide provides guidance on the type of information to be submitted to the staff for reviewing an application for material use license. This Guide endorses the methods and procedures for medical licensing applications contained in the current revision of NUREG-1556, Volume 9, "Consolidated Guidance About Material Licenses: Program – Specific Guidance About Medical Use Licenses."

- Draft Final Revision 2 to Regulatory Guide 10.9 (DG019), "Guide for the Preparation of Applications for the Use of Self-Contained Dry Source-Storage Gamma Irradiators"

This guide provides guidance on the type of information to be submitted to the staff for reviewing applications of self-contained dry source-storage gamma irradiators. This Guide endorses the methods and procedures contained in NUREG-1556, Volume 5, "Consolidated guidance about materials Licenses: Program-Specific guidance about Self-shielded Irradiator Licenses."

The Committee decided not to review the proposed versions of these Guides and requested an opportunity to review the draft final versions of these guides after reconciliation of public

comments. Based on the review of these Guides, which incorporate public comments, Dr. Powers recommended that the Committee not review these Guides.

Informal Meetings with the NRC Staff/Interaction with the Industry

Informal meetings are being arranged by the NRC staff through direct contact with the members. This is not a good practice, since it compromises the independence of the ACRS. All meetings should be arranged by the appropriate ACRS staff engineer. In accordance with the Federal Advisory Committee Act, these meetings should be used solely to gather information for use by the Committee in its decisionmaking process. During such meetings:

- The members should not make recommendations or provide advice.
- The members could provide their views, but it should be stated clearly that those are their personal views and do not necessarily reflect those of the full Committee.
- An ACRS staff member should be present at these meetings.
- No significant technical issues should be discussed. To the extent feasible, these meetings should be limited to planning purpose.

In addition, members should not contact a licensee or applicant to discuss any Committee proposed position or an individual member view on matters being or expected to be reviewed by the ACRS Committee. The appropriate ACRS staff engineer should be notified, to gather information for use by the Committee in its deliberations.

Meeting with the Commission

The ACRS will meet with the Commission on Friday, November 7, 2008. After consulting with the ACRS Chairman, we informed the Office of SECY that there are no significant topics to discuss with the Commission in November and that the Commission consider postponing the meeting to March 2009.

On August 27, 2008, we received an email from SECY stating that the Commission definitely wants to meet with the ACRS in November and some of the topics the Commission would like to discuss include:

- Management of the Committee, including strategies for getting qualified candidates for membership and increasing diversity among the membership.
- Challenges in the coming year
- Committee views on power uprates for BWRs

The ACRS Chairman feels that the first topic should be discussed during a meeting with individual Commissioners.

The proposed topics for meeting with the Commission are as follows:

1. Overview
 - Accomplishments
 - Ongoing/Future ACRS Activities, including challenges in the coming year

2. PWR Sump Performance Issues
3. Committee views on Power uprates for BWRs
4. TRACE Computer Code Development

The Commission has requested to meet with the ACRS twice a year. To support two ACRS meetings with the Commission, the first meeting should be held in March/April and the second in October/November. SECY is seeking the Commissioners' views on this proposal.

Quadripartite Working Group Meeting

France's Groupe Permanent chargé de Réacteurs Nucleaires (GPR) will host the second Quadripartite Working Group (WG) meeting in France on October 9-10, 2008 on the general topic of "EPR". Drs. Bonaca and Powers, and Mr. Stetkar will be attending the meeting on behalf of ACRS. The proposed agenda, suggested topics for ACRS presentations and available draft presentations were discussed.

13) ACRS Retreat in 2009

The last ACRS retreat was held on January 26-27, 2006. The Committee should decide whether it would like to hold a retreat in 2009.

Impact of Continuing Resolution on FY2009 ACRS Activities

The Agency is preparing to operate under a Continuing Resolution (CR) beginning October 1, 2008, and continuing March 31, 2009. If the CR does remain in effect through FY2009, all NRC Offices have been asked to identify anticipated travel related costs. ACRS travel funds will be allocated bi-weekly and travel authorizations will be approved only a week prior to actual travel start dates.

Members Issue

Travel Request

- Dr. Ryan requests Committee approval and support to attend the NRC staff's Workshop on the Security and Continued Use of Cesium-137 Chloride Sources scheduled for September 29-30, 2008 at the Bethesda North Marriott Hotel.

The meeting was adjourned at 7:00 p.m. on September 5, 2008.