

GNRO-2008/00069

October 22, 2008

U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Attention: Document Control Desk

Subject:

t: Technical Specification Bases and Technical Requriements Manual Update to the NRC for Period Dated October 22, 2008

> Grand Gulf Nuclear Station Docket No. 50-416 License No. NPF-29

Dear Sir and Madam:

Pursuant to Grand Gulf Nuclear Station (GGNS) Technical Specification 5.5.11, Entergy Operations, Inc. hereby submits an update of all changes made to GGNS Technical Specification Bases and Technical Requirements Manual since the last submittal (GNRO-2008/00025 dated March 5, 2008 and GNR O-2008/00005 dated Januar y 10, 2008 to the NRC from GGNS). This update is consistent with update frequency listed in 10CFR50.71(e).

## This letter does not contain any commitments.

Should you have any questions, please contact Michael Larson at (601) 437-6685.

Sincerely,

Michael J. Larson Acting-Manager Licensing

MJL attachment: cc:

GGNS Technical Specification Bases (See Next Page) GNRO-2008/00069 Page 2

cc:

| NRC Senior Resident Inspector<br>Grand Gulf Nuclear Station<br>Port Gibson, MS 39150  |   |
|---|---|
| U.S. Nuclear Regulatory Commission<br>ATTN: Mr. Elmo E. Collins, Jr. (w/2)<br>611 Ryan Plaza Drive, Suite 400<br>Arlington, TX 76011-4005 | ALL LETTERS   |
| U.S. Nuclear Regulatory Commission<br>ATTN: Mr. Carl F. Lyon, NRR/DORL (w/2)<br>ATTN: ADDRESSEE ONLY                                      | ALL LETTERS -<br>COURIER DELIVERY<br>(FEDEX, ETC.)<br>ADDRESS ONLY -            |
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# ATTACHMENT to GNRO-2008/00069

# Grand Gulf Technical Specification Bases Revised Pages

| LDC#  | BASES PAGES AFFECTED | TOPIC of CHANGE   |
|-------|----------------------|---|
| 07044 |                      | CHANGE TO REFLECT USE OF NEW RCS LEAKAGE<br>DETECTION INSTRUMENTATION |

# **Grand Gulf Technical Requirements Manual Revised Pages**

| LDC#  | TRM PAGES AFFECTED  | TOPIC of CHANGE   |
|-------|---------------------|---|
| 08026 | 6.3-5               | COMPLIANCE WITH 10CFR50.68 CRITICALITY<br>MONITORING REQUIREMENTS                         |
| 08023 | 7-8, 7-9, and TRB-7 | CHANGE TO SNUBBER PROGRAM TO CHANGE TO<br>COMPLIANCE WITH ASME OM CODE SUBSECTION<br>ISTD |

BACKGROUND (continued) The Drywell floor drain in-leakage is monitored by the P45 floor drain sump level transmitters and associated instrumentation. The leakage and change in leakage can be determined by monitoring the associated computer point or recorders which calculate leakage based on the level rate of change.

(continued)

| FUNCTION  | APPLICABLE<br>MODES OR<br>OTHER<br>SPECIFIED<br>CONDITIONS | REQUIRED<br>CHANNELS | CONDITION<br>REFERENCED<br>FROM<br>REQUIRED<br>ACTION B.1 | ALARM<br>SETPOINT | MEASUREMENT RANGE           | SURVE ILLANCE<br>REQUIREMENTS          |
|---|--|----------------------|---|-------------------|-----------------------------|--|
| 9. Area Monitors<br>a. Fuel Handling Area<br>Monitors |  |                      |   |                   |                             |  |
| 1) New Fuel<br>Storage Vault                          | (b)  | 1                    | D   | ≤2.5 mR/hr        | $10^{-2}$ to $10^3$ mR/hr   | SR 6.3.1.1<br>SR 6.3.1.2<br>SR 6.3.1.5 |
| 2) Spent fuel<br>Storage Pool                         | (c)  | 1(f)(h)              | D   | ≤2.5 mR/hr        | $10^{-2}$ to $10^3$ mR/hr   | SR 6.3.1.1<br>SR 6.3.1.2<br>SR 6.3.1.5 |
| 3) Dryer Storage<br>Area                              | (d)  | 1(g)                 | D   | ≤15 mR/hr         | $10^{-2}$ to $10^3$ mR/hr   | SR 6.3.1.1<br>SR 6.3.1.2<br>SR 6.3.1.5 |
| b. Control Room<br>Radiation Monitor                  | At all times   | 1                    | D   | ≤0.5 mR/hr        | $10^{-2}$ to $10^{3}$ mR/hr | SR 6.3.1.1<br>SR 6.3.1.2<br>SR 6.3.1.5 |

#### TABLE 6.3.1-1 RADIATION MONITORING INSTRUMENTATION

- \*\* When irradiated fuel is being handled in the primary or secondary containment.
- # When CCW System is in operation.
- ## With ADHR heat exchangers in operation.
- (a) With irradiated fuel in spent fuel storage pool.
- (b) With fuel in the new fuel storage vault.
- (c) With fuel in the spent fuel storage pool.
- (d) With fuel in the dryer storage area.
- (e) The actuation and isolation caused by the Function is covered under other LCOs, except Function 5. Two upscale Hi Hi, one upscale Hi Hi and one downscale, or two downscale signals from the same trip system actuate the trip system and initiate isolation of the associated isolation valves. A channel may be placed in an inoperable status for up to 6 hours for required surveillance without placing the trip system in the tripped condition, provided at least one other OPERABLE channel in the same trip system is monitoring that parameter.
- (f) Two monitors are required to meet the requirements of 10CFR72.124.
- (g) Monitors that are credited to comply with 10CFR50.68 are 1D21-K626, K627 and K629.
- (h) One monitor is required to comply with 10CFR50.68.

LDC 08023

<sup>\*</sup> With RHR heat exchangers in operation.

b. a determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.

Shall become effective upon review and acceptance by the OSRC and the approval of the General Manager, Plant Operations.

## 7.6.3.9 OFFSITE DOSE CALCULATION MANUAL (ODCM)

Licensee initiated changes to the ODCM shall become effective upon review and acceptance by the OSRC.

### 7.6.3.10 SNUBBER PROGRAM

- 7.6.3.10.1 Snubber testing and visual examinations will be performed in accordance with the ASME OM Code Subsection ISTD and applicable addenda as required.
- 7.6.3.10.2 Deleted
- 7.6.3.10.3 Deleted

### 7.7 <u>REPORTING REQUIREMENTS</u>

## 7.7.1 ROUTINE REPORTS

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Nuclear Regulatory Commission pursuant to Section 50.4 of 10 CFR Part 50.

### 7.7.1.1 <u>STARTUP REPORTS</u>

Deleted

## 7.7.2 SPECIAL REPORTS

Special reports shall be submitted to the Nuclear Regulatory Commission pursuant to Section 50.4 of 10 CFR 50 within the time period specified for each report.

#### 7.7.2.1 ECCS SYSTEM ACTUATIONS

In the event an ECCS system is actuated and injects water into the Reactor Coolant System during Modes 1, 2, or 3, a Special Report shall be prepared and submitted to the Commission within 90 days describing the circumstances of the actuation and the total accumulated actuation cycles to date. The current value of the usage factor for each affected safety injection nozzle shall be provided in this Special Report whenever its value exceeds 0.70.

## 7.7.2.2 DIESEL GENERATOR FAILURES

Deleted

### 7.8 <u>RECORD RETENTION</u>

Deleted

## GGNS TRM

## B 7.6.3.10 SNUBBERS

The snubber testing and visual inspection program will be performed in accordance with ASME OM Code Subsection ISTD and applicable addenda as required by 10CFR50.55a.