

IPRenewal NPEmails

From: Bo Pham
Sent: Thursday, June 21, 2007 12:56 PM
To: Kimberly Green
Subject: Fwd: Re: Response from "Contact the NRC Web Site Staff"

something via the OPA web on aging management. Not sure if it makes sense though.

>>> Scott Burnell 6/21/07 12:17 PM >>>
Bo;

Enjoy. I get the feeling this guy's not going to be happy with "we're here to discuss the process, not debate the acceptability of the application."

Scott

>>> NRCWEB 06/21/2007 10:52 AM >>>

Thank you for your request. We have forwarded your concern to our Office of Public Affairs for response directly back to you.

--NRC Web Team

>>> Karl <nukekarl@aol.com> 6/21/2007 9:05:02 AM >>>
Below is the result of your feedback form. It was submitted by

Karl (nukekarl@aol.com) on Thursday, June 21, 2007 at 09:05:02

comments: Why is Entergy's Indian Point Unit 3 license renewal application being considered especially since the first line of defense the Reactor Pressure Vessel has a lower shell plate (heat number B 2803-3) that does not meet 10 CFR50 Appendix G requirements for upper shelf energy (fracture toughness) for the license renewal time frame? The 10CFR50 Appendix G has a definitive limit of a minimum of 50 ft-lbs shall be maintained for the material life of the reactor vessel. Please review IP3 current licensing basis aging management program "IP3 Reactor Surveillance Capsule Program results from withdrawn surveillance capsules T, Y, Z and I think also "S" has been removed which are on file with NRC. A review of IP3 response to your Req Guide 1.99 Rev. 2 would be helpful background information for you also. During your review and extrapolation of the capsules data and the data in their response to Reg Guide 1.99 rev.2 out to 60 years this RPV beltline region plate material fall!

s significantly below the 50 ft-lb minimum requirement. I will be at the forthcoming meeting on 6/27/07 to discuss the license renewal review process, aging management issues for IP2 and IP3 with NRC representatives Mr. Bo Pham (Senior Project Manager). In addition I plan on discussing aging issues with the pressurizers (fatigue limit issues for the upper part of the pressurizer shell (only good for 44 years), the spray nozzle (only good for 49 years), manway bolts, seismic support lugs, lower head (only good for 42 years), support skirt weld to the lower head (good for only 54 years); Please note that when you impose the NRC environmental effects on these design fatigue curves for the safety and relief nozzles, have to be considered (good for 53 years) along with the instrument nozzles (51 years). In addition discussions on presuurizer materials will be addressed e.g. alloy 600 material, part of its pressure boundary. Cracking has been found in two US nuclear Power Plants!

identified as failure mechanism known as stress corrosion cracking. I

n addition I will be prepared to discuss Reactor Vessel Internals aging management issues as well. This E-mail is being sent now to help you prepare for the questions that will be asked.

Thank You

P.S. If you want to discuss more aging management issues for the NSSS components and other safety related components and buildings at the meeting such as Reactor Coolant Pumps, Steam Generators, Containment, Reactor Vessel Head replacement, Shared systems with IP1 please let me know in the return e-mail.

Thanks Again

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Hearing Identifier: IndianPointUnits2and3NonPublic_EX
Email Number: 704

Mail Envelope Properties (Bo.Pham@nrc.gov20070621125604)

Subject: Fwd: Re: Response from "Contact the NRC Web Site Staff"
Sent Date: 6/21/2007 12:56:04 PM
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From: Bo Pham

Created By: Bo.Pham@nrc.gov

Recipients:
"Kimberly Green" <Kimberly.Green@nrc.gov>
Tracking Status: None

Post Office:

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Options
Priority: Standard
Return Notification: No
Reply Requested: No
Sensitivity: Normal
Expiration Date:
Recipients Received: