

October 14, 2008

Mr. Mark Bezilla
Site Vice President
FirstEnergy Nuclear Operating Company
Perry Nuclear Power Plant
P. O. Box 97, 10 Center Road, A-PY-290
Perry, OH 44081-0097

**SUBJECT: PERRY NUCLEAR POWER PLANT – NOTIFICATION OF NRC INSPECTION
AND REQUEST FOR INFORMATION**

Dear Mr. Bezilla:

On December 1, 2008, the NRC will begin inspection of refueling and other outage activities (NRC Inspection Procedure 71111.20) prior to the 2009 refueling outage scheduled for the Perry Nuclear Power Plant. This inspection will utilize NRC Operating Experience Smart Sample: (OpESS) FY 2007-03, Revision 2, "Crane and Heavy Lift Inspection, supplemental guidance for IP-71111.20," issued September 12, 2008. This inspection will focus on the impact of load handling activities on the reactor core and its cooling and plant support systems. Specifically, this inspection will review applicable design and licensing bases, engineering design and analysis, procedures, and surveillances associated with the reactor vessel head removal and installation during refueling outages including: (1) Containment building crane preventative maintenance, inspections and testing; (2) reactor vessel head rigging equipment; (3) reactor vessel head drop analysis; and (4) reactor vessel head safe load paths. The on-site inspection is scheduled to take place on December 1, 2008 through December 5, 2008, and December 15 through December 19, 2008.

Experience has shown that inspection of these activities is resource intensive both for the NRC inspector and licensee staff. In order to minimize the inspection impact to the site and to ensure a productive inspection for both parties, we have enclosed a request for information needed for the inspection. This information should be available in Region III by November 24, 2008, (preparatory week). It is important that all of these documents are up to date and complete in order to minimize the number of additional documents requested during the preparation and/or the on-site portions of the inspection.

We have discussed the schedule for these inspection activities with your staff and understand that our regulatory contact for this inspection will be Mr. K. Russell, of your organization. If there are any questions about this inspection or the material requested, please contact the inspector, Mr. J. Bozga, at (630) 829-9613.

M. Bezilla

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In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

David E. Hills, Chief
Engineering Branch 1
Division of Reactor Safety

Docket No. 50-440
License No. NPF-58

Enclosure: Refueling and Other Outage Activities Inspection Document Request

cc w/encl: J. Hagan, President and Chief Nuclear Officer - FENOC
J. Lash, Senior Vice President of Operations and
Chief Operating Officer - FENOC
D. Pace, Senior Vice President, Fleet Engineering - FENOC
J. Rinckel, Vice President, Fleet Oversight - FENOC
P. Harden, Vice President, Nuclear Support
Director, Fleet Regulatory Affairs - FENOC
Manager, Fleet Licensing - FENOC
Manager, Site Regulatory Compliance - FENOC
D. Jenkins, Attorney, FirstEnergy Corp.
Public Utilities Commission of Ohio
C. O'Claire, State Liaison Officer, Ohio Emergency Management Agency
R. Owen, Ohio Department of Health

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C. O'Claire, State Liaison Officer, Ohio Emergency Management Agency
R. Owen, Ohio Department of Health

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DATE	10/14/08	10/14/08	10/14/08		

OFFICIAL RECORD COPY

Letter to Mr. Mark Bezilla from Mr. David E. Hills dated October 14, 2008.

SUBJECT: PERRY NUCLEAR POWER PLANT – NOTIFICATION OF NRC INSPECTION
AND REQUEST FOR INFORMATION

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INITIAL DOCUMENT REQUEST

Inspection Dates: December 1 through December 5, and December 15
through December 19, 2008

Inspection Procedures: IP 71007, "Reactor Vessel Head Replacement Inspection"
IP 71111.20, "Refueling and Other Outage Activities"

Inspector: John V. Bozga
(630) 829-9613

A. Information Requested for the In-Office Preparation Week

The following information (paper copy if practicable, unless otherwise indicated) is requested to be available in Region III on November 24, 2008. If you have any questions regarding this information, please call the inspector as soon as possible.

1. A copy of the following documents related heavy load control during removal or installation of the reactor vessel head:
 - a. Perry submittals as a result of NRC December 22, 1980, unnumbered Generic Letter (GL) (now NRC numbered as GL 80-113) and GL 81-07, "Control of Heavy Loads" regarding NUREG-0612, "Control of Heavy Loads at Nuclear Power Plants," six month response (Phase I) and nine month response (Phase II).
 - b. Reactor head drop analysis associated with above Phase II submittals, if applicable.
 - c. Perry corrective action program reports related to NRC Regulatory Issue Summary 2005-25, "Clarification of NRC Guidelines for Control of Heavy Loads," NRC Regulatory Issue Summary 2005-25, "Supplement I Clarification of NRC Guidelines for Control of Heavy Loads," Enforcement Guidance Memorandum 2007-006, "Enforcement Discretion for Heavy Load Handling Activities," and Nuclear Energy Institute 08-05, "Industry Initiative on Control of Heavy Loads."
 - d. Current reactor head drop analysis (if different from Phase II submittals) or single failure proof crane/equivalency evaluation.
 - e. Other supporting documents that demonstrate equipment to maintain safe shutdown will be unaffected (i.e., calculations associated with the reactor head drop analysis) and that potential offsite releases at the exclusion boundary will be within 10 CFR Part 100 limits after a postulated reactor vessel head drop.
 - f. Calculations for rigging and special lifting devices associated with lifting the reactor vessel head.

INITIAL DOCUMENT REQUEST

- g. Documents that establish the total lift weight for the existing reactor vessel head.
 - h. Site procedures associated with "Control of Heavy Loads," removal of the reactor vessel head during refueling operations, installation of the reactor vessel head during refueling operations, and any contingency actions taken to mitigate the consequences of a postulated reactor vessel head drop.
 - i. Containment building crane vendor recommendations regarding preventative maintenance, and inspection, and testing prior to use.
 - j. Site procedures associated with Containment building crane preventative maintenance, inspection, and testing prior to lifting the reactor vessel head.
 - k. Documentation for Containment building crane preventative maintenance, inspection, and testing performed during last refueling outage prior to lifting the reactor vessel head.
2. An electronic copy of the Perry UFSAR.

If you have questions regarding any of the information requested, please contact the lead inspector.