UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET, SW, SUITE 23T85
ATLANTA, GEORGIA 30303-8931

October 3, 2008

Mr. J. Art Stall
Senior Vice President Nuclear and Chief Nuclear Officer
Florida Power and Light Company
St. Lucie Nuclear Plant
P.O. Box 14000
Juno Beach, FL 33408-0420

SUBJECT: ST. LUCIE NUCLEAR PLANT – NRC OPERATOR LICENSE EXAMINATION REPORT 05000335/2008301 AND 05000389/2008301

Dear Mr. Stall:

During the period July 23-25 and July 28-30, 2008, the Nuclear Regulatory Commission (NRC) administered operating tests to employees of your company who had applied for licenses to operate the St. Lucie Nuclear Plant. At the conclusion of the tests, the examiners discussed preliminary findings related to the operating tests and the written examination submittal with those members of your staff identified in the enclosed report. The written examination was administered by your staff on August 1, 2008.

Eight Senior Reactor Operator (SRO) applicants passed both the operating test and written examination. One SRO applicant failed the written examination. There were three post-examination comments concerning the written examination. These comments, and the NRC resolution of these comments, are summarized in Enclosure 2. A Simulator Fidelity Report is included in this report as Enclosure 3.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosures will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of the NRC's document system (ADAMS).

ADAMS is accessible from the NRC Website at http://www.nrc.gov/reading-rm.adams.html (the Public Electronic Reading Room).

If you have any questions concerning this letter, please contact me at (404) 562-4550.

Sincerely,

/RA/

Malcolm T. Widmann, Chief
Operations Branch
Division of Reactor Safety

Docket Nos: 50-335, 50-389
License Nos: DPR-67, NPF-16

Enclosures:
1. Report Details
2. Facility Comments and NRC Resolution
3. Simulator Fidelity Report

(cc: w/encl - See page 3)

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Letter to J. Art Stall from Malcolm T. Widmann dated October 3, 2008

SUBJECT: ST. LUCIE NUCLEAR PLANT – NRC OPERATOR LICENSE EXAMINATION REPORT 05000335/2008301 AND 05000389/2008301

Distribution w/encl:
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B. Mozafari, NRR (PM: STL, TP)

October 3, 2008

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See Previous Concurrence

X PUBLICLY AVAILABLE ☐ NON-PUBLICLY AVAILABLE ☐ SENSITIVE X☐ NON-SENSITIVE

ADAMS: X☐ Yes ACCESSION NUMBER:_____ X☐ SUNSI REVIEW COMPLETE

OFFICE	RII:DRS	RII:DRS	RII:DRS	RII:DRS	RII:DRP		
SIGNATURE	/RA/	/RA/	/RA/	/RA/	/RA/		
NAME	P Capehart	RBaldwin	R Aiello	MWidmann	MSykes		
DATE	10/ /2008	10/1/2008	10/1/2008	10/3/2008	10/3/2008	10/ /2008	10/ /2008
E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO

OFFICIAL RECORD COPY DOCUMENT NAME: O:\ST. LUCIE EXAMINATIONS\INITIAL EXAM 2008-301\CORRESPONDENCE\MASTER FINAL EXAM REPORT ST. LUCIE JULY 2008 (RSB).DOC

NUCLEAR REGULATORY COMMISSION

REGION II

Docket No.:	50-335, 50-389
License No.:	DPR-67, NPF-16
Report No.:	05000335/2008301, 05000389/2008301
Licensee:	Florida Power and Light Company (FP&L)
Facility:	St. Lucie Nuclear Plant, Units 1 & 2
Location:	6351 S. Ocean Drive Jensen Beach, FL 34957
Dates:	Operating Test – July 23-25 and July 28-30, 2008 Written Examination – August 1, 2008
Examiners:	R. Baldwin, Chief Examiner, Senior Operations Engineer R. Aiello, Senior Operations Engineer P. Capehart, Chief-Under-Instruction, Operations Engineer
Approved by:	Malcolm T. Widmann, Chief Operations Branch Division of Reactor Safety

Enclosure 1

SUMMARY OF FINDINGS

ER 05000335/2008301, 05000389/2008301, 07/23-25 and 07/28-30/2008 and 08/01/2008; St. Lucie Nuclear Plant, Licensed Operator Examinations.

The NRC examiners conducted operator licensing initial examinations in accordance with the guidance in NUREG-1021, Revision 9, Supp.1, "Operator Licensing Examination Standards for Power Reactors." This examination implemented the operator licensing requirements of 10 CFR §55.41, §55.43, and §55.45.

The NRC administered the operating tests during the period of July 23-25 and July 28-30, 2008. Members of the St. Lucie Power Plant training staff administered the written examination on August 1, 2008. The written examination outline, written examination, operating test outlines and operating test details were developed by the St. Lucie Nuclear Plant training staff.

Eight Senior Reactor Operator (SRO) applicants passed both the written and operating examinations. One SRO applicant failed the written examination. Eight SRO applicants were issued operating licenses.

There were three post examination comments.

No findings of significance were identified.

REPORT DETAILS

4. OTHER ACTIVITIES

4OA5 Operator Licensing Initial Examinations

a. Inspection Scope

The St. Lucie Nuclear Plant training staff developed the written examination and operating test. NRC regional examiners reviewed the proposed examination material to determine whether it was developed in accordance with NUREG-1021, "Operator Licensing Examination Standards for Power Reactors," Revision 9, Supplement 1. Examination changes agreed upon between the NRC and the licensee were made according to NUREG-1021 and incorporated into the final version of the examination materials.

The examiners reviewed the licensee's examination security measures while preparing and administering the examinations to ensure examination security and integrity complied with 10 CFR 55.49, "Integrity of Examinations and Tests."

The examiners evaluated nine SRO applicants who were being assessed under the guidelines specified in NUREG-1021. The examiners administered the operating tests during the period of July 23-25 and July 28-30, 2008. Members of the St. Lucie Power Plant training staff administered the written examination on August 1, 2008. The evaluations of the applicants and review of documentation were performed to determine if the applicants, who applied for licenses to operate the St. Lucie Nuclear Plant, met requirements specified in 10 CFR Part 55, "Operators' Licenses."

b. Findings

The NRC determined that the details provided by the licensee for the written exam, walkthrough, and simulator tests were within the range of acceptability expected for a proposed examination.

Eight Senior Reactor Operator (SRO) applicants passed both the written and operating examinations. One SRO applicant failed the written examination.

The final RO and SRO written examinations with knowledge and abilities (K/As) question references/answers and examination references, and licensee's post examination comments may be accessed in the ADAMS system (ADAMS Accession Numbers, ML082680225 and ML082680203).

Copies of all individual examination reports were sent to the facility Training Manager for evaluation and determination of appropriate remedial training.

4OA6 Meetings

Exit Meeting Summary

On July 31, 2008, the examination team discussed generic issues associated with the operating test with Mr. C. Costanzo, Plant General Manager, and members of the St.

Lucie Nuclear Plant staff. The examiners asked the licensee whether any materials examined during the inspection should be considered proprietary. No proprietary information was identified.

PARTIAL LIST OF PERSONS CONTACTED

Licensee personnel

C. Costanzo, Plant General Manager
M. Bladek, Assistant Operations Manager – Support
S. Dunston, Training Manager
D. Lanyi, Operations – Support
J. Klauck, Assistant Operations Manger
J. Milligan, Initial License Training Supervisor
L. Rich, Examination Writer

NRC Resolution to the Facility Comment

A complete text of the licensee's post examination comments can be found in ADAMS under Accession Number ML082680203.

SRO QUESTION # 65

LICENSEE COMMENT:

In summary, the licensee requests this question be graded with two correct answers (choices "A" and "C"). The post exam comments submitted by the licensee can be viewed in their entirety in ADAMS under the above referenced ML number.

The question asks the applicant to identify the proper actions if the 1B2 Circulating Water pump tripped with Unit 1 at 34% power, the 1B1 Circulating water pump is out of service and a 31°F differential temperature exists across the condenser.

Unit 1 Procedure 1-060030, "Circulating Water System,;" Rev. 36A, off normal entry conditions are: 1) Circulating Water Pump trip and 2) High Differential Temperature (>30°F) are met. Conditions are provided in the stem of the question (31°F differential temperature) that exceeds the expected differential temperature across the condenser. Based upon entering pProcedure 1-060030procedure, steps are provided in the subsequent operator actions to control turbine power so that differential temperature limits are not exceeded.

Procedureprocedure 1-060030 states in sStep step 2 Contingency Actions: "IF both circulating water pumps in one condenser are stopped, THEN begin a turbine/reactor shutdown in a controlled manner and attempt to restore at least one circulating water pump in that condenser." This step is the bases for the original answer choice "A."

The licensee contends, because the initial differential temperature given in the stem of the question is above the entry condition (>30°F) for Pprocedure 1-060030, Sprocedure, step 9.H also applies: "Control turbine power so that differential temperature limits or discharge canal temperature limits are NOT exceeded."

NRC RESOLUTION

Recommendation accepted. Based on the additional information provided by the licensee, it has been determined that two answers are correct based on the initial conditions listed in the ofstem of the question.

The original answer, choice " B," would have been the only correct answer if the Condenser Differential Temperature was less than 31°F. Since Condenser Differential Temperature was provided as 31°F in the stem of the question as 31°F, this allows answer choice "D" to also be correct.

In accordance with NUREG 1021, Rev 9, Supplement 1, ES-403, Section D.1.c, question # 65 will be graded allowing for two correct answers. The answer key and examination will be changed to reflect that either choice "B" or choice "D" is correct.

SRO QUESTION # 70

LICENSEE COMMENT:

In summary, the licensee requests this question be graded with two correct answers (choices "A" or "C"). The post exam comments submitted by the licensee can be viewed in their entirety in ADAMS under the above referenced ML number.

The question asks the applicant to identify a plant operation or malfunction that will result in an inaccurate or invalid RCS inventory balance calculation.

Unit 2 Procedure, 2-OSP-01.03, "Reactor Coolant System Inventory Balance,""; Rev 2B, states that ANY of the following variables could result in inaccurate or invalid calculations:

1) Changes in RCS temperature
2) Adjustments to Quench Tank parameters

"A Quench Tank drain and refill is in progress" is marked as the correct choice.

The licensee contends that another choice, "Adding, 50 gallons of primary water to the Charging pump suction"," would slightly change RCS temperature and could therefore also be considered an additional correct answer.

NRC RESOLUTION

Recommendation accepted. Based on the additional information provided by the licensee, it has been determined that two answers are correct based on the list of responses to choose from.

The original answer, "C", would have been the only correct answer if this was the only item listed as a choice from the list of known items in the Reactor Coolant System (RCS) Inventory Balance Procedure. Since "Changes in RCS temperature" is listed in 2-OSP-01.03 as an item that could result in inaccurate or invalid calculations and "50 gallons of primary water added to the Charging pump suction" (choice "A") will impact RCS temperature, this allows choice "A" to also be considered as correct.

In accordance with NUREG 1021, Rev 9, Supplement1, ES-403, Section D.1.c, question # 70 will be graded allowing for two correct answers. The answer key and examination will be changed to reflect that "A" or "C" is correct.

RO QUESTION # 95

LICENSEE COMMENT:

In summary, the licensee requests this question be graded with two correct answers (choices "A" or "C"). The post exam comments submitted by the licensee can be viewed in their entirety in ADAMS under the above referenced ML number.

The question asks the applicant to identify the **required** operator response in accordance with the off normal procedure based on secondary sodium chemistry increasing to an Action Level 2 and then subsequently decreasing 3 hours later to Action level 1 values after isolating the source of the leak.

Unit 1 Procedure, 1-0610030, "Secondary Chemistry – Off Normal;" Rev. 23A, Step 5.3.6, For Action Level 2 "Required Actions:" Power de-escalation can be terminated if the source is terminated and levels are below Action Level 2. The next bulleted item goes on to state that after an Action Level 2 excursion, **consideration** should be given to further reduction in power.

NRC RESOLUTION

Recommendation not accepted. The stem of the question asks the applicant to identify what is **required** based on the conditions given and not by what should be taken into **consideration**.

The licensee's contention that choice "A", to continue the downpower, is also correct, is based on the procedure section title for this step. Substeps of Step 5.3.6, "For Action Level: 1" are listed as "Required Actions." However, the actual step defined by choice "A" states to **consider** reducing power. This implies that a downpower is not **required**. This is contradictory to the wording of the choice "A", which implies a downpower must continue.

Question # 95 is valid with only one correct answer. The only correct answer is "C." No change to the answer key is warranted.

SIMULATION FACILITY REPORT

Facility Licensee: St. Lucie Nuclear Plant

Facility Docket Nos.: 05000335/05000389

Operating Tests Administered on: July 23-25, 2008; July 28-30, 2008

This form is to be used only to report observations. These observations do not constitute audit or inspection findings and, without further verification and review in accordance with IP 71111.11, are not indicative of noncompliance with 10 CFR 55.46. No licensee action is required in response to these observations.

No simulator fidelity or configuration items were identified.