



Entergy Nuclear Northeast
Entergy Nuclear Operations, Inc.
James A. Fitzpatrick NPP
P.O. Box 110
Lycoming, NY 13093
Tel 315-342-3840

Pete Dietrich
Site Vice President - JAF

September 24, 2008
JAFP-08-0098

**Subject: Entergy Nuclear Operations, Inc.
James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333
License No. DPR-59**

James A. FitzPatrick Nuclear Power Plant – Supplement to The Application for Amendment to Technical Specifications Regarding Relocation of Pressure and Temperature (P-T) Curves to the Pressure and Temperature Limits Report (PTLR) Consistent with TSTF-419-A (TAC No. MD8556)

- References:
- 1) Entergy Letter, JAFP-08-0034, "Application for Amendment to Technical Specifications Regarding Relocation of Pressure and Temperature (P-T) Curves to the Pressure and Temperature Limits Report (PTLR) Consistent with TSTF-419-A", dated April 22, 2008 (TAC No. MD8556)
 - 2) Entergy Letter, JAFP-08-0067, "Response to Request For Additional Information Regarding Relocation of Pressure and Temperature Curves to the Pressure and Temperature Limits Report (TAC No. MD8556)", dated July 22, 2008

Dear Sir or Madam:

Entergy Nuclear Operations, Inc. (ENO), as operator of the James A. FitzPatrick Nuclear Power Plant (JAF), hereby submits this supplement to the Application for Amendment to Technical Specifications Regarding Relocation of Pressure and Temperature (P-T) Curves to the Pressure and Temperature Limits Report (PTLR) Consistent with TSTF-419-A, dated April 22, 2008 (Reference 1). In ENO's response to NRC's request for additional information (Reference 2) ENO included a specific analysis of the small diameter instrument nozzles in the belt region of the JAF reactor vessel. That analysis should be referenced in the Technical Specification 5.6.7 discussion of analytical methods, as shown on the Technical Specification pages attached to this supplement.

Attachment 1 contains a mark-up of Technical Specification page 5.6-3 and Attachment 2 contains a clean typed version of the same Technical Specification page showing the additional reference. The clean typed Technical Specification page provided in Attachment 2 should replace the clean typed page 5.6-3 provided in Reference 1.

ADD
NRR

There are no new commitments made in this letter.

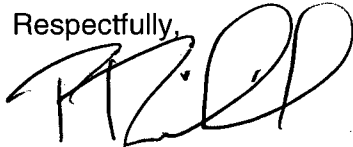
This letter does not affect the "No Significant Hazards" determination made in Reference 1.

Should you have any questions concerning this letter, please contact Mr. Gene Dorman, Acting Licensing Manager, at (315) 349-6810.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on the 24th day of September, 2008.

Respectfully,



Pete Dietrich
Site Vice President

Attachments:

1. Marked-up Technical Specification Page
2. Clean Typed Technical Specification Page

cc:

Mr. Samuel J. Collins
Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406-1415

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James A. FitzPatrick Nuclear Power Plant
U. S. Nuclear Regulatory Commission
P. O. Box 136
Lycoming, New York 13093

Mr. Paul Eddy
New York State Department of
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Plant Licensing Branch I-1
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Office of Nuclear Reactor Regulation
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17 Columbia Circle
Albany, New York 12203-6399

ATTACHMENT 1 to JAFP-08-0098

**Marked-up Technical Specification Page
Page 5.6-3**

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal/mechanical limits, core thermal/hydraulic limits, Emergency Core Cooling Systems (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
- d. The COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

5.6.6 PAM Report

When a report is required by Condition B or F of LCO 3.3.3.1, "Post Accident Monitoring (PAM) Instrumentation," a report shall be submitted within the following 14 days. The report shall outline the preplanned alternate method of monitoring, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channels of the Function to OPERABLE status.

5.6.7 Reactor Coolant System (RCS PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR))

- a. RCS pressure and temperature limits for heatup, cooldown, low temperature operation, criticality, and hydrostatic testing as well as heatup and cooldown rates shall be established and documented in the PTLR for the following:
 - i) Limiting Conditions for Operation Section 3.4.9 "RCS Pressure and Temperature (P/T) Limits"
 - ii) Surveillance Requirements Section 3.4.9 "RCS Pressure and Temperature (P/T) Limits"
 - b. The analytical methods used to determine the RCS pressure and temperature limits shall be those previously reviewed and approved by the NRC, specifically those described in the following document:
 - i) SIR-05-044-A, "Pressure-Temperature Limits Report Methodology for Boiling Water Reactors"
 - ii) SIA Calculation, 0800846.301, "2" Instrument Nozzle Stress Analysis"
 - c. The PTLR shall be provided to the NRC upon issuance for each reactor vessel fluence period and for any revision or supplement thereto.
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ATTACHMENT 2 to JAFP-08-0098

**Clean Typed Technical Specification Page
Page 5.6-3**

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal mechanical limits, core thermal hydraulic limits, Emergency Core Cooling Systems (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
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