



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 20, 2008

Mr. Dale E. Young, Vice President
Crystal River Nuclear Plant (NA1B)
ATTN: Supervisor, Licensing & Regulatory Programs
15760 W. Power Line Street
Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER UNIT 3 - REQUEST FOR ADDITIONAL INFORMATION,
REGARDING THE 2007 STEAM GENERATOR TUBE INSERVICE
INSPECTIONS DURING REFUELING OUTAGE 15 (TAC. NO. MD8918)

Dear Mr. Young:

By letter dated May 30, 2008, Florida Power Corporation (the licensee) submitted information summarizing the results of the fall 2007 steam generator tube inspections performed at Crystal River, Unit 3 during refueling outage 15. The Nuclear Regulatory Commission (NRC) staff has reviewed the information the licensee provided and determined that additional information is required in order to complete the evaluation. The NRC staff's request for additional information is enclosed.

Please respond to the enclosed questions within 30 days of the date of this letter. Please contact me at 301-415-1447, if you have any questions on this issue.

Sincerely,

Farideh E. Saba

Farideh E. Saba, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosure: As stated

cc w/enclosure: See next page

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*by Memorandum

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Crystal River Nuclear Plant, Unit 3

cc:

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Crystal River, FL 34428-6708

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REQUEST FOR ADDITIONAL INFORMATION
REGARDING CRYSTAL RIVER UNIT 3
STEAM GENERATOR TUBE INSERVICE INSPECTION REPORTS
DOCKET NO. 50-302, LICENSE NO. DPR-72

1. On page 7 and 8, you indicated that your normal operating differential pressure (NODP) is 1500 pounds per square inch (psi). However, you also indicated that 3 times your NODP is only 4300 psi. Please clarify. If you did not achieve your full 3 times NODP, please discuss the implications given that the flaws appeared to have been opening up (based on a comparison of the pre and post in situ eddy current inspection results).
2. Please confirm that all detected crack-like indications were plugged (except for those that were determined to be acceptable for service, based on an NRC-approved alternate tube repair criteria).
3. Please discuss whether any degradation of your tube supports was detected. If so, discuss the extent and the basis for dispositioning this degradation.
4. Please discuss the results of your visual and eddy current inspection of the plugs.
5. Other than postulated leakage from tube-end cracking flaws, please discuss whether there are any other sources that contributed to your estimates of accident-induced primary-to-secondary leakage.
6. Please discuss whether any indications were detected at dented locations. If so, please discuss the nature and size of the indication and the size of the dent. If any indications were found in dents whose voltage was near the threshold value for performing rotating probe examinations, please provide the basis for why no sample expansion was necessary.
7. Please discuss whether any indications were detected in the sleeves (including the parent tube at the joints) or in the lower tubesheet crevice. If any indications were detected, please discuss the nature of the indications.
8. Page 9 of 20 indicated that 293 re-roll repairs were performed in steam generator A and 703 re-roll repairs in steam generator B. These values do not appear to match the numbers on page 15 (263 tubes re-rolled in steam generator A and 566 tubes re-rolled in steam generator B). In addition, these numbers do not appear to match the number of indications re-rolled as identified in Appendix 4 (although these numbers do appear to match the number or re-rolled cited on page 5). Please clarify.
9. Please discuss the source and nature of the "general volumetric degradation" identified on page 5. Please discuss whether any volumetric indications other than wear and first span intergranular attack in steam generator B were left in service. If so, discuss the basis for leaving these flaws in service.
10. Please discuss whether the foreign object inside tube 73-99 in steam generator A was removed.

Enclosure