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Chapter 1 Introduction and General Description of the Plant

1.1 Introduction

This section of the ESBWR DCD, i.e., the referenced DCD, is incorporated by reference with the following departures and/or supplements.

1.1.1 Format and Content

VCS SUP 1.1-1

1.1.1.1 10 CFR 52 and Regulatory Guide 1.206

This FSAR was developed to comply with the content requirements of 10 CFR 52.79, and to the extent feasible, the content and format guidance contained in RG 1.206, "Combined License Applications for Nuclear Power Plants (LWR Edition)." See Table 1.9-203, Conformance With the FSAR Content Guidance In RG 1.206. If the information requested by RG 1.206 is not needed (e.g., because it is already provided in the DCD or is located elsewhere in the FSAR), the table specifies the location of the information.

Section C.III.6 of RG 1.206 addresses referencing a design certification (DC) application rather than a certified design. The existing DC rules (10 CFR 52 appendices) require that a COLA that references a certified design include a plant-specific DCD containing the same type of information and using the same organization and numbering as the generic DCD for the ESBWR design, as modified and supplemented by the applicant's exemptions and departures. Consistent with this guidance and the expected approval of the ESBWR DCD, the organization and numbering of this FSAR follows the organization and numbering of the generic DCD for the ESBWR design as modified and supplemented by exemptions and departures. Where necessary to present additional information, new sections were added following the logical structure of the ESBWR generic DCD.

1.1.1.2 Standard Review Plan

As required by 10 CFR 52.79(a)(41), an evaluation of the facility for conformance with the acceptance criteria contained in NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants LWR Edition," in effect 6 months before submittal of the COLA was performed. This evaluation determined that this FSAR contains no unacceptable deviations from the acceptance criteria given in

the applicable portions of the SRP. Where necessary, Table 1.9-201, Conformance with Standard Review Plan, provides a summary of any differences from the SRP acceptance criteria, along with a justification for an exception to a criterion or a branch technical position; or the table identifies the applicable FSAR section(s) that addresses a difference.

1.1.1.3 Tables and Figures

Tabulations of data are designated "tables." Each is identified by the section or subsection number followed by a number (for example, Table 1.9-204 would be an FSAR table in Section 1.9). The use of the "200" series for FSAR table numbers distinguishes FSAR tables from DCD tables. If a table from the DCD is referenced in the FSAR text, it is denoted as such, for example "DCD Table 4.1-1." If a table from the DCD was revised for use in the FSAR, the original DCD table number was appended with an "R;" for example, if "DCD Table 4.2-1" was revised, it would have become "Table 4.2-1R." Tables are located at the end of the section immediately following the text.

Drawings, pictures, sketches, curves, graphs, and engineering diagrams identified as "figures" are numbered using the section or subsection number followed by a number (for example, Figure 2.1-201 would be an FSAR figure in Section 2.1). The use of the "200" series for FSAR figure numbers distinguishes FSAR figures from DCD figures. If a figure from the DCD is referenced in the FSAR text, it is denoted as such; for example "DCD Figure 4.1-1." If a figure from the DCD was revised for use in the FSAR, the original DCD figure number was appended with an "R;" for example, if "DCD Figure 4.2-1" was revised, it would have become "Figure 4.2-1R." Figures are located at the end of the applicable section following the tables.

1.1.1.4 Numbering of Pages

Pages are numbered sequentially within each chapter, section, or subsection. For example, Page 3-2 is the second page in Chapter 3, Page 1.1-4 is the fourth page in Section 1.1, and Page 2.5.4-3 is the third page in the Subsection 2.5.4.

1.1.1.5 Proprietary and Security-Related Sensitive Unclassified Non-Safeguards Information (SUNSI)

Proprietary information and SUNSI[1] is withheld from public disclosure and therefore not included in the FSAR. This information is provided in COLA Part 9, SUNSI.

STD SUP 1.1-2

1.1.1.6 Acronyms

In addition to the summary list of acronyms in the FSAR front matter, acronyms are defined at their first occurrence in the FSAR text.

1.1.1.7 Incorporation by Reference

10 CFR 52.79 states in part that, "The final safety analysis report need not contain information or analyses submitted to the Commission in connection with the design certification, provided, however, that the final safety analysis report must either include or incorporate by reference the standard design certification final safety analysis report and must contain, in addition to the information and analyses otherwise required, information sufficient to demonstrate that the site characteristics fall within the site parameters specified in the design certification." Therefore, because this COLA references the ESBWR DC application, the FSAR incorporates by reference the ESBWR DCD with certain supplemental information (see Section 1.1.1.10). References in this FSAR to the DCD should be understood to mean the ESBWR DCD, Tier 2, submitted by GE-Hitachi Nuclear Energy Americas LLC (GEH), as Revision 4.

1.1.1.8 Departures from the Standard Design Certification (or Application)

A departure is a plant-specific "deviation" from design information in a standard DC rule or, consistent with Section C.III.6 of RG 1.206, from design information in a DC application.

10 CFR 52 clarifies that Tier 2 information in a standard DC rule does not include conceptual design information and per Section C.III.6 of RG 1.206, Tier 2 information in a standard DC application does not include

1. Any information that, if lost, misused, modified, or accessed without authorization, can reasonably be foreseen as causing harm to the public interest, the commercial or financial interest of the entity or individual to whom the information pertains, the conduct of NRC and Federal programs, or the personal privacy of individuals. SUNSI has been organized into the following seven groups: (1) Allegation information, (2) Investigation information, (3) Security-related information, (4) Proprietary information, (5) Privacy Act information, (6) Federal, state, foreign government, and international agency-controlled information, (7) Sensitive internal information. (Source: "Guidance for Electronic Submissions to the NRC," Rev. 3, 11/20/07)

conceptual design information. Therefore, replacement or revision of conceptual design information does not constitute a departure. Additionally, information addressing combined license (COL) information/holder items and supplemental information (see Section 1.1.1.10) that does not change the intent or meaning of the ESBWR DCD text is not considered a departure from the ESBWR DCD.

VCS SUP 1.1-2

1.1.1.9 Referencing of ESP Information

Not applicable, an ESP application was not submitted for the VCS site.

1.1.1.10 Supplements

Supplements fall into one of the following categories:

- COL Item
- Conceptual design information
- Supplemental information (see definition below)

Supplemental information is FSAR information that includes information not related to COL Items, departures, or conceptual design (see Table 1.1-201 for definition of terms), or is information to demonstrate that the site characteristics fall within the design parameters specified in the DCD.

STD SUP 1.1-3

1.1.1.11 Left Margin Annotations

FSAR sections are annotated in the left margin with information that identifies: 1) the reason the information is being provided and, as applicable, 2) whether the information is standard (identical) for any ESBWR application, or specific to the COLA for a particular plant.

The annotations used and their definitions are listed in Table 1.1-201.

1.1.1.12 Tense

Because this FSAR is a licensing basis document that will control plant design and operations after the COL is issued, the FSAR is generally written in the present tense. Thus, plant design and configuration are described in the present tense although the plant is not yet built. Similarly, programs, procedures, and organizational matters are generally described in the present tense although such descriptions may not yet be implemented. Accordingly, the use of the present tense in this FSAR should be understood as describing the plant, programs and procedures,

and organization as they will exist when in place, and not as a representation that they are already in place.

1.1.2 General Description

1.1.2.1 ESBWR Standard Plant Scope

Replace the last sentence with the following.

VCS CDI

The orientation of the principal plant structures for VCS Units 1 and 2 is shown in Figure 1.1-201.

Add the following at the end of this section.

VCS SUP 1.1-3

In addition to the buildings and structures within the scope of the ESBWR standard plant, the plant includes a normal power heat sink (cooling basin), plant service water cooling towers, a plant service water/cooling water intake structure, a sewage treatment plant, water treatment facilities, storage tanks for water and fuel oil, a switchyard and other site support systems and structures necessary to support the operation and maintenance of the facility.

1.1.2.2 Type of License Request

Add the following at the end of this section.

VCS SUP 1.1-4

This application by Exelon Nuclear Texas Holdings, LLC (Exelon) is for a combined construction permit and operating license, i.e., COL, under Section 103 of the Atomic Energy Act, for each of the two nuclear power plants to be located in Victoria County, Texas. This COLA references a DC application for an ESBWR (consistent with Section C.III.6 of RG 1.206). The two nuclear power plants are designated Victoria County Station, Units 1 and 2.

1.1.2.3 Number of Plant Units

Replace this subsection with the following.

VCS CDI

This FSAR is for two ESBWR units located at the VCS site. DCD changes and additional information needed to license the multiunit plant are supplied at appropriate locations in the FSAR.

1.1.2.4 Description of Location

Add the following at the end of this section.

VCS SUP 1.1-4

Subsection 2.1.1 provides the VCS site location and site description.

1.1.2.7 Rated Core Thermal Power

Replace the last four sentences of this subsection with the following.

VCS COL 1.1-1-A

Each VCS unit operates at an estimated gross electrical power output at rated power of approximately 1600 MWe. The estimated net electrical power output, which is dependent on site ambient conditions, the normal plant heat sink, and station electrical loads, is approximately 1535 MWe.

VCS SUP 1.1-5

1.1.2.8 Schedule

Key milestones associated with the estimated schedule for completion of construction and beginning of commercial operation for Unit 1 are as follows:

Unit 1 Milestone	Estimated Schedule Date
First Structural Concrete	December 2011
Preoperational Testing	December 2013
Fuel Load	June 2015
Commercial Operation	December 2015

Key milestones associated with the estimated schedule for completion of construction and beginning of commercial operation for Unit 2 are as follows:

Unit 2 Milestone	Estimated Schedule Date
First Structural Concrete	June 2013
Preoperational Testing	June 2015
Fuel Load	December 2016
Commercial Operation	June 2017

1.1.3 COL Unit-Specific Information

1.1-1-A Established Rated Electrical Output

VCS COL 1.1-1-A

This COL Item is addressed in Subsection 1.1.2.7.

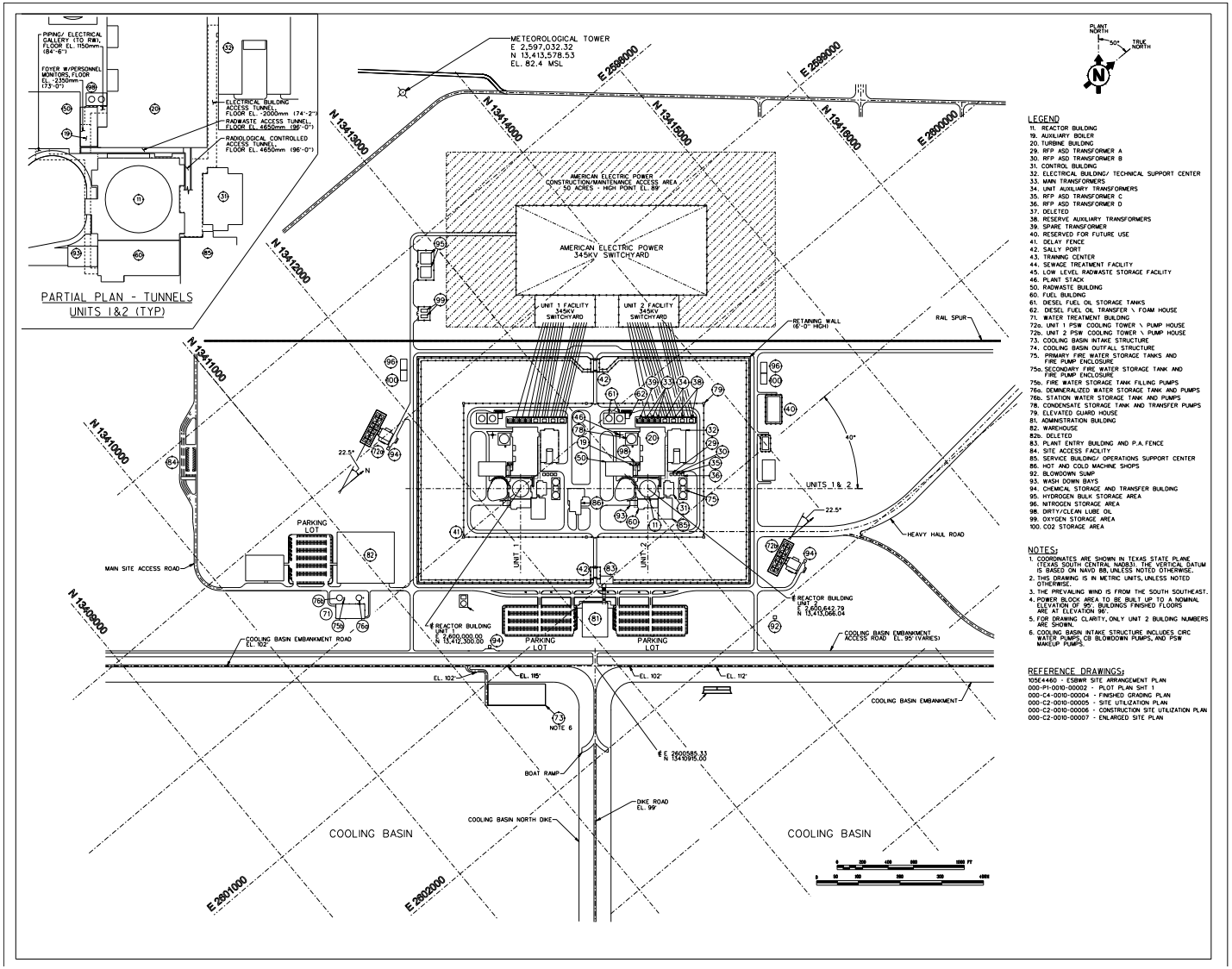
VCS SUP 1.1-2

Table 1.1-201 (Sheet 1 of 2)
Left Margin Annotations

FSAR Component	Margin Annotation	Definition and Use
Standard Departure	STD DEP X.Y.Z -#	FSAR information that departs from the generic DCD and is common for all parallel applicants; i.e., the departure and discussion of the departure are identical for all applicants of the ESBWR technology. Each Standard Departure is numbered based on the applicable section down to the X.Y.Z level, e.g., STD DEP 9.2-1, or STD DEP 9.2.1-1.
Plant-Specific Departure	VCS DEP X.Y.Z-#	FSAR information that departs from the generic DCD and is plant-specific; i.e., the departure and discussion of the departure are not identical for all applicants of the ESBWR technology. Each Plant-Specific Departure is numbered based on the applicable section down to the X.Y.Z level, e.g., VCS DEP 9.2-1, or VCS DEP 9.2.1-1.
Standard COL Item	STD COL X.Y-#-A or STD COL X.Y-#-H	FSAR information that addresses a DCD COL Item that is common for all parallel applicants; i.e., the response to and discussion of the DCD COL Item are identical for all applicants of the ESBWR technology. Each Standard COL Item is numbered as identified in ESBWR DCD Table 1.10-1. The -A refers to a COL Applicant item while the -H refers to a COL Holder item.
Plant-Specific COL Item	VCS COL X.Y-#-A or VCS COL X.Y-#-H	FSAR information that addresses a DCD COL Item that is plant-specific; i.e., the response to the COL Item is not a Standard COL Item for parallel applicants. Each Plant-Specific COL Item is numbered as identified in the ESBWR DCD (see STD COL above).
Standard Conceptual Design Information	STD CDI	A Conceptual Design Information designation is used to identify FSAR information that replaces Conceptual Design Information in the DCD, in whole or in part. Replacement and supplemental Conceptual Design Information is generally plant-specific; however, for conceptual design that is generic for all applications the annotation for standard (STD) is used, STD CDI.
Plant Specific Conceptual Design Information	VCS CDI	A Conceptual Design Information designation is used to identify FSAR information that replaces Conceptual Design Information in the DCD, in whole or in part. Plant specific replacement and supplemental Conceptual Design Information uses the annotation VCS CDI.
Standard Supplemental Information	STD SUP X.Y-#	Supplemental FSAR information that is identical for all parallel applicants; i.e., the supplemental information is identical for all applicants of the ESBWR technology. Each Standard Supplemental Information designation is numbered based on the applicable subsection down to the X.Y level, e.g., STD SUP 10.4-1.

Table 1.1-201 (Sheet 2 of 2)
Left Margin Annotations

FSAR Component	Margin Annotation	Definition and Use
Plant-Specific Supplemental Information	VCS SUP X.Y-#	Supplemental FSAR information that is plant-specific (not standard). Each Plant Specific Supplemental Information designation is numbered based on the applicable subsection down to the X.Y level, e.g., VCS SUP 10.4-1.



VCS CDI

Figure 1.1-201 VCS, Units 1 and 2, Plot Plan

1.2 General Plant Description

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

1.2.2.11.4 **Main Turbine**

Delete the second sentence of the first paragraph and replace the first sentence of the first paragraph with the following.

STD CDI

The main turbine has one high-pressure (HP) turbine and three low-pressure (LP) turbines.

1.2.2.11.6 **Turbine Bypass System**

Delete the last sentence of the third paragraph and replace the third sentence of the first paragraph with the following.

VCS CDI

The TBS has the design capability to shed 110% of rated steam flow, which facilitates shedding of 100% of the turbine generator rated load without reactor trip or operation of the SRVs.

1.2.2.11.7 **Main Condenser**

Delete the second sentence of the third paragraph and replace the first sentence of the third paragraph with the following.

VCS CDI

The main condenser is a two-pass, three-shell, single-pressure, deaerating unit.

1.2.2.12.13 **Hydrogen Water Chemistry System**

Replace the existing text with the following:

STD CDI

The HWC system consists of hydrogen and oxygen supply systems to inject hydrogen in the feedwater and oxygen in the offgas, plus monitoring systems to track the effectiveness of the system.

1.2.2.12.15 **Zinc Injection System**

Replace this section with the following:

STD CDI

The Zinc Injection System is not utilized.

1.2.2.12.16 **Freeze Protection**

Replace this section with the following.

STD CDI

Freeze protection is incorporated at the individual system level using insulation and heat tracing for all external tanks and piping that may freeze during winter weather.

1.2.2.16.10 **Other Building Structures**

Replace the third paragraph with the following.

VCS CDI

Other facilities include the Service Building, Water Treatment Building, Administration Building, Training Center, Sewage Treatment Plant, warehouse, and hot and cold machine shop. These are all of conventional size and design, and in some cases may be shared with other units at the same site.

1.2.2.19 **Modular Construction Techniques and Plans**

STD SUP 1.2-1

To the extent practical, modular construction techniques that have been applied during ABWR construction projects will be adapted and/or modified for use during ESBWR construction. Modularization reviews will be performed to develop a plan for bringing the ABWR experience into the ESBWR. Once completed, the results of the modularization reviews will be used as guidance to develop the detailed design of the areas affected by modularization.

1.3 Comparison Tables

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

Add the following at the end of this section.

VCS COL 1.3-1-A There are no updates to DCD Table 1.3-1 based on unit-specific information.

1.3.1 COL Unit-Specific Information

1.3-1-A Update Table 1.3-1

VCS COL 1.3-1-A This COL item is addressed in Section 1.3.

1.4 Identification of Agents and Contractors

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

1.4.1 Owner

VCS SUP 1.4-1

Units 1 and 2 at the VCS site are owned and operated by Exelon Nuclear Texas Holdings, LLC (Exelon). Exelon is the applicant for the COL and is responsible for making each of the key project decisions. Exelon is the lead for the financial, environmental, and technology evaluations necessary to support each of the project decisions leading to the COL and ultimate decision on whether to build Units 1 and 2. Exelon retains the responsibility for quality assurance compliance. Exelon assurance of quality compliance is confirmed by audits, documenting proper implementation of and compliance with the Exelon and contractor quality programs. Exelon, headquartered in Kennett Square, Pennsylvania, is one of the nation's largest producers of energy. Exelon currently owns and operates 10 nuclear stations with 17 nuclear units.

1.4.2 Nuclear Steam Supply System

General Electric-Hitachi Nuclear Energy (GEH) is responsible for the design, fabrication, and delivery of a boiling water nuclear steam supply system; for the fabrication of the first core of nuclear fuel; and for providing technical direction for the installation and startup of the equipment for the standard GEH ESBWR plant. Each unit at the VCS site is a standard ESBWR plant. GEH's experience and expertise relative to BWR design and construction is discussed in the ESBWR DCD Section 1.4.

1.4.3 Turbine Generator Vendor

GEH has the overall responsibility for the design, fabrication, and delivery of the entire turbine island, including the turbine generator system, for the standard GEH ESBWR dual unit plant. Any subcontractors used for the design, fabrication, and delivery of the turbine island equipment and turbine generator are the responsibility of GEH.

1.4.4 Architect-Engineer

The architect-engineer for the site-specific systems and structures outside the scope of the reactor vendor, for the construction phase of the project, has not been chosen at the time of COLA submittal; this

information will be supplied in an FSAR update as required by 10 CFR 50.71(e) following selection of the architect-engineer.

1.4.5 **Consultants**

Bechtel Power Corporation is responsible for the engineering and licensing efforts to coordinate and prepare the COLA content and support NRC review of the COLA. Founded in 1898, Bechtel is one of the world's premier engineering, construction, and project management companies. Bechtel has a history of supporting the nuclear power industry, beginning with the construction in 1950 of the EBR-1 reactor. Since then, Bechtel has constructed more than 60,000 MWe of nuclear power capacity worldwide.

Tetra Tech NUS, Inc. (TtNUS) performed field investigations to characterize the VCS site and prepared Subsection 2.1.3 and most of the Environmental Report for the COLA. TtNUS also provided general National Environmental Policy Act (NEPA) consultation. TtNUS is an environmental and engineering consulting company with a history of service to the nuclear power industry since the inception of its predecessor company, Nuclear Utility Services (NUS) Corporation in 1960. TtNUS currently has 20 offices and approximately 700 employees throughout the country. TtNUS is a wholly owned subsidiary of Tetra Tech, Inc., which has approximately 9000 employees worldwide.

MACTEC Engineering and Consulting, Inc. (MACTEC) performed geotechnical field investigations and laboratory testing in support of Chapter 2. The scope of services provided by MACTEC included performing soil borings with standard penetration tests, obtaining undisturbed soil samples, performing cone penetrometer tests, performing geophysical and sonic logging of selected borings, drilling, installation and development of observation wells, performing field permeability tests for observation wells, performing laboratory tests of samples, providing as-drilled location surveys, and preparing final boring logs and other data reports. MACTEC is a leading engineering and design, environmental, and construction services company headquartered in Alpharetta, Georgia. The services provided for the VCS site are managed by MACTEC's office in Raleigh, North Carolina.

William Lettis and Associates, Inc., (WLA) performed geotechnical field investigations, geologic mapping and characterization of seismic sources, and sensitivity analyses for the COLA. They also provided

support for the COLA preparation. WLA is a consulting firm based in Walnut Creek, California specializing in applied earth sciences. WLA provides a range of services to support clients in developing Early Site Permit applications and COLAs including detailed site characterization, assessment of capable tectonic features and seismic source zones, preparation of seismic source models for Probabilistic Seismic Hazard Assessment (PSHA) studies during project design, and preparation of Safety Analysis Reports. WLA developed new regulatory guidelines for the U.S. NRC (NUREG/CR 5562, *Dating and Earthquakes: Review of Quaternary Geochronology and its Application to Paleoseismology*, March 1998 and NUREG/CR 5503, *Techniques for Identifying Faults and Determining Their Origins*, July 1999) and has provided input and guidance for various international agencies to review or develop new regulatory criteria pertaining to seismic and geologic hazard studies.

Risk Engineering, Inc. performed PSHAs and related sensitivity analyses in support of Chapter 2. These assignments included sensitivity analyses of seismic source parameters and updated ground motion attenuation relationships, development of updated safe shutdown earthquake ground motion values, and preparation of the related sections.

Additional consultants may be used during the engineering, construction, startup, and operational phases of Units 1 and 2 for activities that are not within the scope of the reactor vendor. Information on these consultants, who have not been chosen at the time of COLA submittal, will be supplied in an FSAR update as required by 10 CFR 50.71(e).

1.5 Requirements for Further Technical Information

This section of the referenced DCD is incorporated by reference with no departures or supplements.

1.6 Material Incorporated by Reference

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

Add the following paragraph at the end of this section.

VCS SUP 1.6-1

Table 1.6-201 lists topical reports not included in DCD Section 1.6 that are incorporated in whole or in part by reference in the FSAR.

VCS SUP 1.6-1

Table 1.6-201
Referenced Topical Reports

Report No.	Title	Section No.
NEI 06-13A	Nuclear Energy Institute, “Technical Report on Template for an Industry Training Program Description,” NEI 06-13A, Revision 1, March 2008	13BB
NEI 06-14A	Nucler Energy Institute, "Quality Assurance Program Description," NEI 06-14A, Revision 4, July 2007	17BB
NEI 07-02A	Nuclear Energy Institute, “Generic FSAR Template Guidance for Maintenance Rule Program Description for Plants Licensed under 10 CFR Part 52,” NEI 07-02A, Revision 0, March 2008	17.6
NEI 07-03	Nuclear Energy Institute, “Generic FSAR Template Guidance for Radiation Protection Program Description,” NEI 07-03, Revision 3, October 2007	12BB
NEI 07-08	Nuclear Energy Institute, “Generic FSAR Template Guidance for Ensuring That Occupational Radiation Exposures Are As Low As Is Reasonably Achievable (ALARA),” NEI 07-08, Revision 0, September 2007	12AA
NEI 07-09	Nuclear Energy Institute, “Generic FSAR Template Guidance for Offsite Dose Calculation Manual (ODCM) Program Description,” NEI 07-09, Revision 0, September 2007	11.5
NEI 07-10	Nuclear Energy Institute, “Generic FSAR Template Guidance for Process Control Program (PCP) Description,” NEI 07-10, Revision 1, October 2007	11.4

1.7 Drawings and Other Detailed Information

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

Add the following paragraph at the end of this section:

STD COL 1.7-1-H

The final P&IDs used for construction will be available upon completion of the final design configuration. Design changes that result in revisions to the simplified diagrams will be incorporated in subsequent updates to the FSAR.

1.7.1 Electrical, Instrumentation and Control Drawings

Add the following at the end of this section:

VCS SUP 1.7-1

Table 1.7-201 supplements DCD Table 1.7-2 for those portions of the electrical system configuration drawings outside the scope of the DCD.

1.7.2 Piping and Instrumentation Diagrams

Add the following at the end of the first paragraph:

VCS SUP 1.7-2

Table 1.7-202 supplements DCD Table 1.7-3 for those portions of the mechanical system configuration drawings outside the scope of the DCD.

1.7.4 COL Information

1.7-1-H **Final Design Configuration Confirmation**

STD COL 1.7-1-H

This COL item is addressed in Section 1.7.

VCS SUP 1.7-1

Table 1.7-201
Summary of Electrical System Configuration Drawings

Figure 8.2-201 Site Switchyard-Substation One-Line Diagram
Figure 8.2-202 VCS Site and Why Substation Location Plan
Figure 8.2-203 Overall 345 kV Transmission Interconnection Configuration
Figure 8.2-204 Transmission Corridors Conceptual Location Plan

VCS SUP 1.7-2

Table 1.7-202
Summary of Mechanical System Configuration Drawings

Figure 9.2.1-201 Plant Service Water System Simplified Diagram
Figure 9.2.3-201 Makeup Water System
Figure 9.2.4-201 Potable Water and Sanitary Waste System Simplified Diagram
Figure 9.2.4-202 Sanitary Waste Discharge System Simplified Diagram
Figure 9.2.10-201 Station Water System Simplified Diagram
Figure 9.3-3R Service Air and Instrument Air System Simplified Diagram
Figure 9.5-201 Fire Protection System
Figure 9.5-202 Fire Protection System-Yard

1.8 Interfaces with Standard Design

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

1.8.2 Identification of BOP Interfaces

Add the following paragraph after the first paragraph of this section.

STD CDI

The significant interface requirements for those systems that are beyond the scope of the DCD are identified in DCD Tier 1.

Delete the second sentence of the second paragraph of this section.

VCS SUP 1.8-1

1.8.3 Verification of Site Parameters

Chapter 2 provides information demonstrating that the site characteristics fall within the ESBWR site parameters specified in the referenced DCD.

VCS SUP 1.8-2

1.8.4 COL Information Items and Permit Conditions

Section 1.10 identifies specific FSAR sections that address the COL information items from the referenced DCD.

VCS SUP 1.8-3

1.8.5 Generic Changes and Departures from the Referenced Certified Design

Departures from the referenced DCD are listed in Table 1.8-201. These departures are described and evaluated in Part 7 of the COLA. There are no generic changes from the referenced certified design.

VCS SUP 1.8-4

1.8.6 Variances from the ESP and ESPA SSAR

Not applicable because an ESP application was not submitted.

VCS SUP 1.8-5

1.8.7 Conceptual Design Information

The referenced DCD includes conceptual design information (CDI) for certain systems, or portions of systems, that are outside the scope of the standard plant design. Table 1.8-203 identifies systems for which either the CDI in the DCD is adopted as the actual system design information, or the CDI in the DCD is replaced with site-specific design information, along with cross references to FSAR sections where the CDI is treated. Where there are differences between the conceptual design and the actual design, these differences have been evaluated. The evaluations have concluded that there are no impacts on the safety evaluations provided in the referenced DCD.

VCS SUP 1.8-6

1.8.8 Probabilistic Risk Assessment (PRA)

Site- and plant-specific information, including site meteorological data and site-specific population distribution, plant-specific design information that replaced conceptual design information described in the DCD, and the departures listed in Subsection 1.8.5, were reviewed with respect to the design certification PRA. The conclusion, which is documented in Section 19.5, is that there is no significant change from the certified design PRA.

VCS SUP 1.8-3

Table 1.8-201
Departures from the Referenced Certified Design

Number	Subject	FSAR Section
2.0-1	Toxic Gas Concentrations at the Main Control Room HVAC Intakes	2.0
2.5.4-1	Minimum Shear Wave Velocity	2.0, 2.5.4.4.5, 3.7.2.4.1
9.2-1	Makeup Water System	9.2.3.1

VCS SUP 1.8-4

Table 1.8-202
Deleted

VCS SUP 1.8-5

Table 1.8-203 (Sheet 1 of 3)
Conceptual Design Information

Item in DCD	CDI in DCD adopted as actual design	CDI in DCD replaced with actual design	Evaluation	FSAR Section
1.1.2.1 ESBWR Standard Plant Scope Figure 1.1-1 ESBWR Standard Plant General Site Plan		X	Site-specific general site plan provided	1.1.2.1 Figure 1.1-201
1.2.2.11.4 Main Turbine	X		Conceptual turbine type selected as site-specific design	1.2.2.11.4
1.2.2.11.6 Turbine Bypass System	X		Conceptual turbine bypass system selected as site-specific design	1.2.2.11.6
1.2.2.11.7 Main Condenser 10.4.1 Main Condenser		X	Main condenser is a two-pass, three-shell, single pressure design	1.2.2.11.7 10.4.1.2.1 10.4.1.2.2 Table 10.4-1R
1.2.2.12.13 Hydrogen Water Chemistry Table 3.2-1 P73 Note 9.3.9 Hydrogen Water Chemistry		X	Hydrogen water chemistry option utilized	1.2.2.12.13 3.2 9.3.9
1.2.2.12.15 Zinc Injection System Table 3.2-1 P74 Note 9.3.11 Zinc Injection System		X	Zinc Injection system not utilized	1.2.2.12.15 3.2 9.3.11
1.2.2.12.16 Freeze Protection		X	Freeze protection incorporated for external tanks and piping that may freeze during winter weather	1.2.2.12.16 9.5.1.4
1.2.2.16.10 Other Building Structures		X	Site-specific buildings specified	1.2.2.16.10
1.8.2 Identification of BOP Interfaces	X		Not applicable	1.8.2
Appendix 3A Seismic Soil-Structure Interaction Analysis		X	Site-specific geotechnical data described in Chapter 2	Appendix 3A.1
Appendix 3A.2 ESBWR Standard Site Plan		X	Site-specific general site plan provided	Section 3A.2 Figure 1.1-201

Table 1.8-203 (Sheet 2 of 3)
Conceptual Design Information

Item in DCD	CDI in DCD adopted as actual design	CDI in DCD replaced with actual design	Evaluation	FSAR Section
9.2.1 Plant Service Water Table 9.2-2 Figure 9.2-1		X	Site-specific system description and design characteristics described	9.2.1.2 Table 9.2.1-201 Figure 9.2.1-201
9.2.3 Makeup Water System Table 9.2-9		X	Site-specific system description and design characteristics described	9.2.3.2 Table 9.2.3-201
9.2.4 Potable and Sanitary Water Systems		X	Site-specific system description and design characteristics described	9.2.4 Figure 9.2.4-201 Figure 9.2.4-202
9.2.10 Station Water System		X	Site-specific system description and design characteristics described	9.2.10.1 9.2.10.2 Table 9.2.10-201 Figure 9.2.10-201
9.3.7 Service Air System		X	Site-specific system description and design characteristics described	9.3.6.2 9.3.7.1 9.3.7.2 Figure 9.3-3R
9A Appendix 9A Fire Hazards Analysis		X	Site-specific buildings specified. Site-specific Fire Zone drawing supplied.	9A Contents 9A.1 9A.3.1 9A.4.9 9A.4.10 Figure 9A.2-33R Figures 9A.2-201 through 9.1.2-203

Table 1.8-203 (Sheet 3 of 3)
Conceptual Design Information

Item in DCD	CDI in DCD adopted as actual design	CDI in DCD replaced with actual design	Evaluation	FSAR Section
10.4.5 Circulating Water System Table 10.4-3 Figure 10.4-1		X	Site-specific system description and design characteristics described	10.1 10.4.5.2.1 10.4.5.2.2 10.4.5.5 10.4.5.8 Table 10.4-3R Figure 10.1-1R Figure 10.4-201
11.2 Liquid Waste Management System	X		Conceptual design for liquid waste management selected as site-specific design	11.2
11.4 Solid Waste Management System	X		Conceptual design for solid waste management selected as site-specific design	11.4

1.9 Conformance with Standard Review Plan and Applicability of Codes and Standards

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

1.9.1 Conformance with Standard Review Plan

Add the following paragraph at the end of this section.

VCS COL 1.9-3-A

Table 1.9-201 evaluates conformance with the standard review plan (SRP) sections and Branch Technical Positions in effect 6 months before the submittal of the COLA. Table 1.9-201 does not readdress conformance with the SRP for those portions of the facility design included in the referenced DCD.

In the table, the term “conforms” means that no exception is being taken to the guidance in the SRP section/acceptance criteria as it applies to site-specific design information, operational aspects of the facility, or siting information. The term “not applicable” means that the SRP section/acceptance criteria do not apply to the ESBWR or VCS. Any differences with the SRP acceptance criteria are identified and justified, with references to the applicable FSAR or DCD section(s) that address the difference, as necessary.

1.9.2 Applicability to Regulatory Criteria

Add the following paragraphs at the end of this section.

VCS COL 1.9-3-A

Division 1, 4, 5, and 8 Regulatory Guides

Table 1.9-202 evaluates conformance with Division 1, 4, 5, and 8 RGs in effect 6 months before the submittal of the COLA. Each issued Division 1 RG is evaluated. Issued Division 4, 5, and 8 RGs identified in the SRP, RG 1.206, or DCD Table 1.9-21 as COL responsibility are also evaluated. Table 1.9-202 does not readdress conformance with RGs for those portions of the facility design included in the referenced DCD.

In the table, the term “conforms” means that no exception is being taken to the guidance in the regulatory positions as it applies to site-specific design information, operational aspects of the facility, or siting information. The term “not applicable” means that the regulatory positions do not apply to the ESBWR or VCS.

Regulatory Guide 1.206

Table 1.9-203 evaluates conformance with the FSAR content guidance in RG 1.206. Where necessary, the table identifies the FSAR section where the required information is provided. In the table, the term “conforms” means that the information called for in RG 1.206 is either: (1) already addressed in the DCD, or (2) addressed by adding new information beyond that contained in the DCD. The term “not applicable” means that the information called for in RG 1.206 does not apply to the ESBWR or VCS.

Table 1.9-203 evaluates conformance with RG 1.206, Section C.III.1, “Information Needed for a Combined License Application Referencing a Certified Design.” Section C, Part I, “Standard Format and Content of Combined License Applications for Nuclear Power Plants - LWR Edition,” was also evaluated, as applicable, if portions of this section were referenced or identified in RG 1.206, Section C.III.1.

VCS SUP 1.9-1

Industrial Codes and Standards

Table 1.9-204 identifies the industrial codes and standards that are applicable to those portions of the VCS design that are beyond the scope of the DCD and to the operational aspects of the facility.

1.9.3 Applicability of Experience Information

Add the following after the first sentence of the section.

VCS SUP 1.9-2

Table 1.9-205 lists NUREG and NUREG/CR reports cited in the FSAR.

Add the following paragraph at the end of this section.

Table 1.9-205 addresses operational experience information, as described in applicable NUREG reports, for those portions of the design and operation that are beyond the scope of the DCD. The comment column of Table 1.9-205 includes a reference to the applicable FSAR section that provides further discussion of the operational experience.

1.9.4 COL Information

VCS COL 1.9-3-A

1.9-3-A RP and Regulatory Guide Applicability

This COL Item is addressed in Sections 1.9.1 and 1.9.2.

VCS COL 1.9-3-A

Table 1.9-201 (Sheet 1 of 43)
Conformance with Standard Review Plan

SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
1	Introduction and Interfaces	Rev. 1	11/2007	No Specific Acceptance Criteria	Conforms
2.0	Site Characteristics and Site Parameters	Initial Issuance	03/2007	II.1, II.2, II.3, II.5	Not applicable
				II.4	Conforms
2.1.1	Site Location and Description	Rev. 3	03/2007	II.1, II.2	Conforms
2.1.2	Exclusion Area Authority and Control	Rev. 3	03/2007	II.1, II.2, II.3	Conforms
2.1.3	Population Distribution	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5	Conforms
2.2.1–2.2.2	Identification of Potential Hazards in Site Vicinity	Rev. 3	03/2007	II.1, II.2, II.3	Conforms
2.2.3	Evaluation of Potential Accidents	Rev. 3	03/2007	II.1, II.2	Conforms
2.3.1	Regional Climatology	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9	Conforms
2.3.2	Local Meteorology	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms
2.3.3	Onsite Meteorological Measurements Programs	Rev. 3	03/2007	II.1, II.2, II.3	Conforms
2.3.4	Short Term Atmospheric Dispersion Estimates for Accident Releases	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6	Conforms
2.3.5	Long-Term Atmospheric Dispersion Estimates for Routine Releases	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6	Conforms
2.4.1	Hydrologic Description	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6	Conforms
2.4.2	Floods	Rev. 4	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10	Conforms
2.4.3	Probable Maximum Flood (PMF) on Streams and Rivers	Rev. 4	03/2007	II.1, II.2, II.3	Conforms

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Conformance with Standard Review Plan

SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
2.4.4	Potential Dam Failures	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7	Conforms
2.4.5	Probable Maximum Surge and Seiche Flooding	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6	Conforms
2.4.6	Probable Maximum Tsunami Hazards	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8	Conforms
2.4.7	Ice Effects	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5	Conforms
2.4.8	Cooling Water Canals and Reservoirs	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms
2.4.9	Channel Diversions	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7	Conforms
2.4.10	Flooding Protection Requirements	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms
2.4.11	Low Water Considerations	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5	Conforms
2.4.12	Groundwater	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5	Conforms
2.4.13	Accidental Releases of Radioactive Liquid Effluents in Ground and Surface Waters	Rev. 3	03/2007	II.1	Conforms. Alternate conceptual models are discussed in Appendix 2.4.12-C.
				II.2, II.3, II.5	Conforms
				II.4	Conforms. There are no site-proximity hazards, seismic, or non-seismic events that would increase the radionuclide concentrations above the values reported in Section 2.4.13.
2.4.14	Technical Specifications and Emergency Operation Requirements	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5	Conforms
2.5.1	Basic Geologic and Seismic Information	Rev. 4	03/2007	II.1, II.2	Conforms
2.5.2	Vibratory Ground Motion	Rev. 4	03/2007	II.1, II.2, II.3, II.4, II.5, II.6	Conforms
2.5.3	Surface Faulting	Rev. 4	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8	Conforms

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Conformance with Standard Review Plan

SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
2.5.4	Stability of Subsurface Materials and Foundations	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12	Conforms
2.5.5	Stability of Slopes	Rev. 3	03/2007	II.1, II.2, II.3	Conforms
3.2.1	Seismic Classification	Rev. 2	03/2007	II.1	Conforms
3.2.2	System Quality Group Classification	Rev. 2	03/2007	II.1	Conforms
3.3.1	Wind Loadings	Rev. 3	03/2007	II.1, II.2, II.3	Conforms
3.3.2	Tornado Loadings	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms
3.4.1	Internal Flood Protection for Onsite Equipment Failures	Rev. 3	03/2007	II.1, II.2	Conforms
3.4.2	Analysis Procedures	Rev. 3	03/2007	II.1, II.2, II.3	Conforms
3.5.1.1	Internally Generated Missiles (Outside Containment)	Rev. 3	03/2007	II.1, II.2	Conforms
3.5.1.2	Internally-Generated Missiles (Inside Containment)	Rev. 3	03/2007	II.1, II.2	Conforms
3.5.1.3	Turbine Missiles	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6	Conforms
3.5.1.4	Missiles Generated by Tornadoes and Extreme Winds	Rev. 3	03/2007	II.1, II.2	Conforms
3.5.1.5	Site Proximity Missiles (Except Aircraft)	Rev. 4	03/2007	II.1, II.2	Conforms
3.5.1.6	Aircraft Hazards	Rev. 3	03/2007	II.1, II.2	Conforms
3.5.2	Structures, Systems, and Components to be Protected from Externally-Generated Missiles	Rev. 3	03/2007	RG 1.13, RG 1.127, RG 1.115, RG 1.117	Conforms

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Conformance with Standard Review Plan

SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
3.5.3	Barrier Design Procedures	Rev. 3	03/2007	II.1, II.2	Conforms
3.6.1	Plant Design for Protection Against Postulated Piping Failures in Fluid Systems Outside Containment	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5	Conforms
3.6.2	Determination of Rupture Locations and Dynamic Effects Associated with the Postulated Rupture of Piping	Rev. 2	03/2007	II.1, II.2, II.3	Conforms
3.6.3	Leak-Before-Break Evaluation Procedures	Rev. 1	03/2007	II.1, II.2	Not applicable. ESBWR design does not rely on a Leak-Before-Break Evaluation.
3.7.1	Seismic Design Parameters	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms
3.7.2	Seismic System Analysis	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12, II.13, II.14	Conforms
3.7.3	Seismic Subsystem Analysis	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12, II.13, II.14	Conforms
3.7.4	Seismic Instrumentation	Rev. 2	03/2007	II.1, II.2	Conforms
3.8.1	Concrete Containment	Rev. 2	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7	Conforms
3.8.2	Steel Containment	Rev. 2	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7	Conforms
3.8.3	Concrete and Steel Internal Structures of Steel or Concrete Containments	Rev. 2	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7	Conforms
3.8.4	Other Seismic Category I Structures	Rev. 2	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8	Conforms
3.8.5	Foundations	Rev. 2	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7	Conforms
3.9.1	Special Topics for Mechanical Components	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms

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Conformance with Standard Review Plan

SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
3.9.2	Dynamic Testing and Analysis of Systems, Structures, and Components	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7	Conforms
3.9.3	ASME Code Class 1, 2, and 3 Components, and Component Supports, and Core Support Structures	Rev. 2	03/2007	II.1, II.2, II.3	Conforms
3.9.4	Control Rod Drive Systems	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms
3.9.5	Reactor Pressure Vessel Internals	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6	Conforms
3.9.6	Functional Design, Qualification, and Inservice Testing Programs for Pumps, Valves, and Dynamic Restraints	Rev. 3	03/2007	II.1, II.3, II.4, II.5, II.6	Conforms
				II.2	Not applicable. There are no safety related pumps.
3.9.7	Risk-Informed Inservice Testing	Rev. 0	08/1998	II.A, II.B	Not applicable. Risk-Informed Inservice Testing is not being used.
3.9.8	Risk-Informed Inservice Inspection of Piping	Rev. 0	9/2003	II.1, II.2, II.3	Not applicable. Risk-Informed Inservice Inspection of piping is not being used.
3.10	Seismic and Dynamic Qualification of Mechanical and Electrical Equipment	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6	Conforms
3.11	Environmental Qualification of Mechanical and Electrical Equipment	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12, II.13, II.14, II.15, II.16	Conforms
3.12	ASME Code Class 1, 2, and 3 Piping Systems, Piping Components and their Associated Supports	Initial Issuance	03/2007	II.A, II.B, II.C, II.D	Conforms

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Conformance with Standard Review Plan

SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
3.13	Threaded Fasteners - ASME Code Class 1, 2, and 3	Initial Issuance	03/2007	II.1, II.2	Conforms
BTP 3-1	Classification of Main Steam Components Other than the Reactor Coolant Pressure Boundary for BWR Plants	Rev. 2	03/2007		Conforms
BTP 3-2	Classification of BWR/6 Main Steam and Feedwater Components Other than the Reactor Coolant Pressure Boundary	Rev. 2	03/2007		Conforms
BTP 3-3	Protection Against Postulated Piping Failures in Fluid Systems Outside Containment	Rev. 3	03/2007		Conforms
BTP 3-4	Postulated Rupture Locations in Fluid System Piping Inside and Outside Containment	Rev. 2	03/2007		Conforms
4.2	Fuel System Design	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms
4.3	Nuclear Design	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms
4.4	Thermal and Hydraulic Design	Rev. 2	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.8, II.9, II.10	Conforms
				II.7	Not applicable. RG 1.133 is not applicable to the ESBWR.
4.5.1	Control Rod Drive Structural Materials	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms
4.5.2	Reactor Internal and Core Support Structure Materials	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5	Conforms

Table 1.9-201 (Sheet 7 of 43)
Conformance with Standard Review Plan

SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
4.6	Functional Design of Control Rod Drive System	Rev. 2	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8	Conforms
BTP 4-1	Westinghouse Constant Axial Offset Control (CAOC)	Rev. 3	03/2007		Not applicable to the ESBWR
5.2.1.1	Compliance with the Codes and Standards Rule, 10 CFR 50.55a	Rev. 3	03/2007	RG 1.26	Conforms
5.2.1.2	Applicable Code Cases	Rev. 3	03/2007	RG 1.84, RG 1.147, RG 1.192	Conforms
5.2.2	Overpressure Protection	Rev. 3	03/2007	II.1, II.2, II.5, II.6, II.7	Conforms
				II.3, II.4	Not applicable to the ESBWR
5.2.3	Reactor Coolant Pressure Boundary Materials	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms. Acceptance Criterion II.3 is addressed in DCD Section 3.9.3.9.
5.2.4	Reactor Coolant Pressure Boundary Inservice Inspection and Testing	Rev. 2	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11	Conforms
5.2.5	Reactor Coolant Pressure Boundary Leakage Detection	Rev. 2	03/2007	II.1, II.2	Conforms
5.3.1	Reactor Vessel Materials	Rev. 2	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7	Conforms
5.3.2	Pressure-Temperature Limits, Upper-Shelf Energy, and Pressurized Thermal Shock	Rev. 2	03/2007	II.1, II.2, II.3	Conforms
5.3.3	Reactor Vessel Integrity	Rev. 2	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8	Conforms
5.4	Reactor Coolant System Component and Subsystem Design	Rev. 2	03/2007		Refers to other SRP sections
5.4.1.1	Pump Flywheel Integrity (PWR)	Rev. 2	03/2007		Not applicable to the ESBWR

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Conformance with Standard Review Plan

SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
5.4.2.1	Steam Generator Materials	Rev. 3	03/2007		Not applicable to the ESBWR
5.4.2.2	Steam Generator Program	Rev. 2	03/2007		Not applicable to the ESBWR
5.4.6	Reactor Core Isolation Cooling System (BWR)	Rev. 4	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10	Conforms
5.4.7	Residual Heat Removal (RHR) System	Rev. 4	03/2007	II.1, II.2, II.3, II.4	Conforms
5.4.8	Reactor Water Cleanup System (BWR)	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms
5.4.11	Pressurizer Relief Tank	Rev. 3	03/2007		Not applicable to the ESBWR
5.4.12	Reactor Coolant System High Point Vents	Rev. 1	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12, II.13, II.14	Conforms
5.4.13	Isolation Condenser System (BWR)	Initial Issuance	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12	Conforms
BTP 5-1	Monitoring of Secondary Side Water Chemistry in PWR Steam Generators	Rev. 3	03/2007		Not applicable to the ESBWR
BTP 5-2	Overpressurization Protection of Pressurized-Water Reactors While Operating at Low Temperatures	Rev. 3	03/2007		Not applicable to the ESBWR
BTP 5-3	Fracture Toughness Requirements	Rev. 3	03/2007		Conforms
BTP 5-4	Design Requirements of the Residual Heat Removal System	Rev. 3	03/2007		Not applicable to ESBWR
6.1.1	Engineered Safety Features Materials	Rev. 2	03/2007	II.1, II.2, II.3, II.4	Conforms

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Conformance with Standard Review Plan

SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
6.1.2	Protective Coating Systems (Paints) - Organic Materials	Rev. 3	03/2007	II.1	Conforms
6.2.1	Containment Functional Design	Rev. 3	03/2007		Refers to other SRP sections
6.2.1.1.A	PWR Dry Containments, Including Subatmospheric Containments	Rev. 3	03/2007		Not applicable to the ESBWR
6.2.1.1.B	Ice Condenser Containments	Draft Rev. 3	06/1996		Not applicable to the ESBWR
6.2.1.1.C	Pressure-Suppression Type BWR Containments	Rev. 7	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11	Conforms
6.2.1.2	Subcompartment Analysis	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms
6.2.1.3	Mass and Energy Release Analysis for Postulated Loss-of-Coolant Accidents (LOCAs)	Rev. 3	03/2007	II.1, II.2, II.3	Conforms
6.2.1.4	Mass and Energy Release Analysis for Postulated Secondary System Pipe Ruptures	Rev. 2	03/2007		Not applicable to the ESBWR
6.2.1.5	Minimum Containment Pressure Analysis for Emergency Core Cooling System Performance Capability Studies	Rev. 3	03/2007		Not applicable to the ESBWR
6.2.2	Containment Heat Removal Systems	Rev. 5	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8	Conforms
6.2.3	Secondary Containment Functional Design	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms. See DCD Table 1.9-20.

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Conformance with Standard Review Plan

SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
6.2.4	Containment Isolation System	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12, II.13, II.14, II.15, II.16, II.17, II.18, II.19, II.20, II.21, II.22	Conforms
6.2.5	Combustible Gas Control in Containment	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9	Conforms
6.2.6	Containment Leakage Testing	Rev. 3	03/2007	No enumerated acceptance criteria	Conforms
6.2.7	Fracture Prevention of Containment Pressure Boundary	Rev. 1	03/2007	II.1, II.2	Conforms
6.3	Emergency Core Cooling System	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.6, II.7, II.8, II.10	Conforms
				II.5, II.9	Not applicable to the ESBWR
6.4	Control Room Habitability System	Rev. 3	03/2007	II.1, II.2, II.4, II.5, II.6, II.7	Conforms
				II.3	Exception: For differential pressure testing of the control room, the periodic verification interval of every 18 months in Acceptance Criteria II.3.a through II.3.c is increased to every 24 months to accommodate the ESBWR's 2-year operating cycle. The frequencies for testing the CR HVAC system are defined by Technical Specifications 3.7.2 and 5.5.12 of the referenced certified design.
6.5.1	ESF Atmosphere Cleanup Systems	Rev. 3	03/2007		Conforms. Surveillances, testing, and maintenance guidelines for CRHAVS are addressed in Technical Specifications 3.7.2, 5.5.12, and 5.5.13, maintenance rule requirements in Section 17.6, and procedure requirements in Section 13.5.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
6.5.2	Containment Spray as a Fission Product Cleanup System	Rev. 4	03/2007		Not applicable. See DCD Table 1.9-20.
6.5.3	Fission Product Control Systems and Structures	Rev. 3	03/2007	II.1, II.2, (there is no II.3)	Conforms
				II.4	Not applicable. Drywell spray function is not credited in DCD Chapter 15 dose analysis.
6.5.4	Ice Condenser as a Fission Product Cleanup System	Draft Rev. 4	06/1996		Not applicable to the ESBWR
6.5.5	Pressure Suppression Pool as a Fission Product Cleanup System	Rev. 1	03/2007	II.1, II.2	Conforms. Refer to DCD Table 1.9-20.
				II.3	Not applicable to the ESBWR
6.6	Inservice Inspection and Testing of Class 2 and 3 Components	Rev. 2	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11	Conforms
6.7	Main Steam Isolation Valve Leakage Control System (BWR)	Draft Rev. 3	06/1996		Not applicable to the ESBWR
BTP 6-1	pH For Emergency Coolant Water for Pressurized Water Reactors	Initial Issuance	03/2007		Not applicable to the ESBWR
BTP 6-2	Minimum Containment Pressure Model for PWR ECCS Performance Evaluation	Rev. 3	03/2007		Not applicable to the ESBWR
BTP 6-3	Determination of Bypass Leakage Paths in Dual Containment Plants	Rev. 3	03/2007		Conforms. Refer to DCD Table 1.9-20.
BTP 6-4	Containment Purging During Normal Plant Operations	Rev. 3	03/2007		Conforms. Refer to TS SR 3.6.1.3.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
BTP 6-5	Currently the Responsibility of Reactor Systems Piping From the RWST (or BWST) and Containment Sump(s) to the Safety Injection Pumps	Rev. 3	03/2007		Not applicable to the ESBWR
7.0	Instrumentation and Controls - Overview of Review Process	Rev. 5	03/2007		No acceptance criteria
Appendix 7.0-A	Review Process for Digital Instrumentation and Control Systems	Rev. 5	03/2007		Conforms
7.1	Instrumentation and Controls - Introduction	Rev. 5	03/2007	II.1, II.2, II.3	Conforms. Procedures addressed in Section 13.5. ITAAC addressed in COLA Part 10.
7.1-T	Table 7-1 Regulatory Requirements, Acceptance Criteria, and Guidelines for Instrumentation and Control Systems Important to Safety	Rev. 5	03/2007		Conforms
Appendix 7.1-A	Acceptance Criteria and Guidelines for Instrumentation and Controls Systems Important to Safety	Rev. 5	03/2007	1, 2, 3, 4, 5	Conforms
Appendix 7.1-B	Guidance for Evaluation of Conformance to IEEE Std 279	Rev. 5	03/2007		Conforms

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
Appendix 7.1-C	Guidance for Evaluation of Conformance to IEEE Std 603	Rev. 5	03/2007		Conforms
Appendix 7.1-D	Guidance for Evaluation of the Application of IEEE Std 7-4.3.2	Initial Issuance	03/2007	SRM to SECY 93-087 II.Q	Conforms
7.2	Reactor Trip System	Rev. 5	03/2007	II.1, II.2, II.3, II.4, SRM to SECY 93-087 II.Q	Conforms. Procedures addressed in Section 13.5. Technical Specifications addressed in Chapter 16. ITAAC addressed in COLA Part 10.
7.3	Engineered Safety Features Systems	Rev. 5	03/2007	II.1, II.2, II.3, II.4, SRM to SECY 93-087 II.Q	Conforms. Procedures addressed in Section 13.5. Technical Specifications addressed in Chapter 16. ITAAC addressed in COLA Part 10.
7.4	Safe Shutdown Systems	Rev. 5	03/2007	II.1, II.2, II.3	Conforms. Procedures addressed in Section 13.5. Technical Specifications addressed in Chapter 16. ITAAC addressed in COLA Part 10.
7.5	Information Systems Important to Safety	Rev. 5	03/2007	II.1, II.2, II.3, II.4, II.5, SRM to SECY 93-087 II.Q	Conforms. Procedures addressed in Section 13.5. Technical Specifications addressed in Chapter 16. ITAAC addressed in COLA Part 10.
7.6	Interlock Systems Important to Safety	Rev. 5	03/2007	II.1, II.2, II.3	Conforms. Procedures addressed in Section 13.5. Technical Specifications addressed in Chapter 16. ITAAC addressed in COLA Part 10.
7.7	Control Systems	Rev. 5	03/2007	II.1, II.2, II.3, II.4, SRM to SECY 93-087 II.Q	Conforms. Procedures addressed in Section 13.5. Technical Specifications addressed in Chapter 16. ITAAC addressed in COLA Part 10.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
7.8	Diverse Instrumentation and Control Systems	Rev. 5	03/2007	II.1, II.2, II.3, II.4, SRM to SECY 93-087 II.Q	Conforms. Procedures addressed in Section 13.5. Technical Specifications addressed in Chapter 16. ITAAC addressed in COLA Part 10.
7.9	Data Communication Systems	Rev. 5	03/2007	II.1, II.2, II.3	Conforms. Addressed in DCD Section 7.1. Procedures addressed in Section 13.5. Technical Specifications addressed in Chapter 16. ITAAC addressed in COLA Part 10.
Appendix 7-A	General Agenda, Station Site Visits (formerly Appendix 7-B)	Rev. 5	03/2007		Not applicable. Provides guidance to the NRC to conduct site visits.
Appendix 7-B	Acronyms, Abbreviations, and Glossary (formerly Appendix 7-C)	Rev. 5	03/2007		Conforms
BTP 7-1	Guidance on Isolation of Low-Pressure Systems from the High-Pressure Reactor Coolant System	Rev. 5	03/2007		Conforms
BTP 7-2	Guidance on Requirements of Motor-Operated Valves in the Emergency Core Cooling System Accumulator Lines	Rev. 5	03/2007		Not applicable to the ESBWR
BTP 7-3	Guidance on Protection System Trip Point Changes for Operation with Reactor Coolant Pumps Out of Service	Rev. 5	03/2007		Not applicable to the ESBWR

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
BTP 7-4	Guidance on Design Criteria for Auxiliary Feedwater Systems	Rev. 5	03/2007		Not applicable to the ESBWR
BTP 7-5	Guidance on Spurious Withdrawals of Single Control Rods in Pressurized Water Reactors	Rev. 5	03/2007		Not applicable to the ESBWR
BTP 7-6	Guidance on Design of Instrumentation and Controls Provided to Accomplish Changeover from Injection to Recirculation Mode	Rev. 5	03/2007		Not applicable. ESBWR does not use recirculation pumps or active ECCS pumps.
HICB-7	Not Used		03/2007		Not used
BTP 7-8	Guidance for Application of Regulatory Guide 1.22	Rev. 5	03/2007		Conforms. Chapter 16 addresses technical specifications.
BTP 7-9	Guidance on Requirements for Reactor Protection System Anticipatory Trips	Rev. 5	03/2007		Conforms
BTP 7-10	Guidance on Application of Regulatory Guide 1.97	Rev. 5	03/2007		Conforms. Section 13.5 addresses procedures.
BTP 7-11	Guidance on Application and Qualification of Isolation Devices	Rev. 5	03/2007		Conforms
BTP 7-12	Guidance on Establishing and Maintaining Instrument Setpoints	Rev. 5	03/2007		Conforms. Section 13.5 addresses procedures.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
BTP 7-13	Guidance on Cross-Calibration of Protection System Resistance Temperature Detectors	Rev. 5	03/2007		Not applicable. RTDs are not used in the ESBWR protection systems.
BTP 7-14	Guidance on Software Reviews for Digital Computer-Based Instrumentation and Control Systems	Rev. 5	03/2007		Conforms
HCIB-15	Not Used		03/2007		Not used
HCIB-16	Withdrawn		03/2007		Withdrawn
BTP 7-17	Guidance on Self-Test and Surveillance Test Provisions	Rev 5	03/2007		Conforms. Section 13.5 addresses procedures. Chapter 16 addresses technical specifications.
BTP 7-18	Guidance on the Use of Programmable Logic Controllers in Digital Computer-Based Instrumentation and Control Systems	Rev. 5	03/2007		Conforms. Section 13.5 addresses procedures.
BTP 7-19	Guidance for Evaluation of Diversity and Defense-in-Depth in Digital Computer-Based Instrumentation and Control Systems	Rev. 5	03/2007		Conforms
HCIB-20	Not Used				Not used
BTP 7-21	Guidance on Digital Computer Real-Time Performance	Rev. 5	03/2007		Conforms

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
8.1	Electric Power - Introduction	Rev. 3	03/2007		Conforms
8.2	Offsite Power System	Rev. 4	03/2007	II.1, II.2, II.3, II.7	Not applicable. ESBWR is a passive design and does not rely on offsite power.
				II.4, II.5, II.6, II.8	Conforms
8.3.1	A-C Power Systems (Onsite)	Rev. 3	03/2007	II.1, II.2, II.3, II.4.A, II.4.C, II.4.D, II.4.E, II.4.F, II.4.H, II.4.J, II.5, II.6, II.7, II.10	Conforms
				II.4.B, II.4.I	Not applicable. The ESBWR diesel generators are not safety-related.
				II.4.G, II.8	Not applicable. The ESBWR diesel generators are not safety-related, nor is AC power needed to achieve safe shutdown.
				II.9	Conforms. Addressed in DCD Section 17.4 and in Section 17.6.
8.3.2	D-C Power Systems (Onsite)	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.7, II.8, II.9, II.10	Conforms
				II.5, II.6	Not applicable. Addressed in DCD Sections 8.3.2.1.1 and 8.3.2.2.2.
				II.11	Not applicable. The ESBWR is designed to shutdown safely without reliance on offsite or diesel-generator-derived AC power for 72 hours, which exceeds station blackout requirements.
				II.12, II.13	Conforms. Addressed in Section 17.6.
8.4	Station Blackout	Initial Issuance	03/2007	II.1, II.2	Conforms. Addressed in DCD Section 15.5.5.
				II.3	Not applicable. Onsite Class 1E Emergency AC power sources are not required for ESBWR safe shutdown.
				II.4, II.5	Conforms. Addressed in Section 17.6.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
Appendix 8-A	General Agenda, Station Site Visits	Rev. 1	03/2007		Not applicable. Provides guidance to NRC to conduct site visits.
BTP 8-1	Requirements on Motor-Operated Valves in the ECCS Accumulator Lines	Rev. 3	03/2007		Not applicable. The ESBWR does not have any safety-related motor-operated valves.
BTP 8-2	Use of Diesel-Generator Sets for Peaking	Rev. 3	03/2007		Not applicable. The ESBWR will not use the non-safety related diesel generators as peaking units.
BTP 8-3	Stability of Offsite Power Systems	Rev. 3	03/2007		Conforms. Stability studies were performed to investigate the loss of off-site generation.
BTP 8-4	Application of the Single Failure Criterion to Manually Controlled Electrically Operated Valves	Rev. 3	03/2007		Not applicable. The ESBWR does not use any manually-operated valves to mitigate an accident.
BTP 8-5	Supplemental Guidance for Bypass and Inoperable Status Indication for Engineered Safety Features Systems	Rev. 3	03/2007		Not applicable. The ESBWR is designed in accordance with ICSB 21, the predecessor to BTP 8-5, as stated in DCD Table 8.1-1 and DCD Section 8.3.2.2.2. Also refer to DCD Table 7.1-1 for conformance to RG 1.47 and BISI for all safety-related systems.
BTP 8-6	Adequacy of Station Electric Distribution System Voltages	Rev. 3	03/2007		Not applicable. The ESBWR is designed in accordance with PSB 1, the predecessor to BTP 8-6, as stated in DCD Table 8.1-1 and DCD Section 8.3.1.1.2.
BTP 8-7	Criteria for Alarms and Indications Associated with Diesel-Generator Unit Bypassed and Inoperable Status	Rev. 3	03/2007		Not applicable. The ESBWR does not use safety-related diesel generators.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
9.1.1	Criticality Safety of Fresh and Spent Fuel Storage and Handling	Rev. 3	03/2007	II.1	Conforms
9.1.2	New and Spent Fuel Storage	Rev. 4	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9	Conforms
9.1.3	Spent Fuel Pool Cooling and Cleanup System	Rev. 2	03/2007	II.1, II.2, II.4, II.5, II.6, II.7	Conforms
				II.3	Conforms. Fuel and auxiliary pool cooling system is not a shared system.
				II.8	Conforms. EP-ITAAC are addressed in COLA Part 10.
9.1.4	Light Load Handling System (Related to Refueling)	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms
9.1.5	Overhead Heavy Load Handling Systems	Rev. 1	03/2007	II.1, II.2, II.3, II.4	Conforms
9.2.1	Station Service Water System	Rev. 5	03/2007	II.1, II.2, II.3, II.4, II.5, II.6	Conforms
9.2.2	Reactor Auxiliary Cooling Water Systems	Rev. 4	03/2007	II.1, II.2, II.3, II.4, II.5, II.6	Conforms
9.2.3	Demineralized Water Makeup System				SRP withdrawn
9.2.4	Potable and Sanitary Water Systems	Rev. 3	03/2007	II.1.A, II.1.B, II.1.C	Conforms
9.2.5	Ultimate Heat Sink	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5	Conforms
9.2.6	Condensate Storage Facilities	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7	Conforms

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
9.3.1	Compressed Air System	Rev. 2	03/2007	II.1, II.2, II.3, II.4	Conforms. Instrument Air is addressed in DCD Section 9.3.6, Service Air is addressed in DCD Section 9.3.7, and High Pressure Nitrogen Supply System is addressed in DCD Section 9.3.8.
9.3.2	Process and Post-Accident Sampling Systems	Rev. 3	03/2007	II.1, II.3, II.4	Conforms
				II.2	Exception. Technical specifications do not require analyses. Section 9.3.2 addresses actions required to qualify process sampling for taking radioactive samples without having a specific post-accident sampling system. Analyses and frequencies of process systems are addressed in plant operating procedures.
9.3.3	Equipment and Floor Drainage System	Rev. 3	03/2007	II.1, II.2, II.3	Conforms
9.3.4	Chemical and Volume Control System (PWR) (Including Boron Recovery System)	Rev. 3	03/2007		Not applicable to the ESBWR
9.3.5	Standby Liquid Control System (BWR)	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5	Conforms
9.4.1	Control Room Area Ventilation System	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6	Conforms. Section 9.4 was evaluated against these criteria.
9.4.2	Spent Fuel Pool Area Ventilation System	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms
9.4.3	Auxiliary and Radwaste Area Ventilation System	Rev. 3	03/2007	II.1, II.2, II.3	Conforms. Section 9.4 was evaluated against these criteria.
9.4.4	Turbine Area Ventilation System	Rev. 3	03/2007	II.1, II.2, II.3	Conforms

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
9.4.5	Engineered Safety Feature Ventilation System	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6	Conforms
9.5.1	Fire Protection Program	Rev. 5	03/2007	II.1, II.2, II.4	Not applicable. See DCD Table 1.9-21.
				II.3, II.5, II.6	Conforms
				II.7	Exception: The elements of the Fire Protection Program required to be operational prior to receipt of new fuel are those elements necessary to protect buildings storing new fuel and adjacent fire areas that could affect the fuel storage area. Other required elements of the Fire Protection Program will be fully operational prior to initial fuel loading. Refer to Section 13.4.
9.5.2	Communications Systems	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12, II.13, II.14	Conforms
9.5.3	Lighting Systems	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms
9.5.4	Emergency Diesel Engine Fuel Oil Storage and Transfer System	Rev. 3	03/2007		Not applicable to the ESBWR
9.5.5	Emergency Diesel Engine Cooling Water System	Rev. 3	03/2007		Not applicable to the ESBWR
9.5.6	Emergency Diesel Engine Starting System	Rev. 3	03/2007		Not applicable to the ESBWR
9.5.7	Emergency Diesel Engine Lubrication System	Rev. 3	03/2007		Not applicable to the ESBWR
9.5.8	Emergency Diesel Engine Combustion Air Intake and Exhaust System	Rev. 3	03/2007		Not applicable to the ESBWR

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
10.2	Turbine Generator	Rev. 3	03/2007	II.1.A, II.1.B, II.2.B, II.3	Conforms
				II.1.C	Exception: The TGS has the capability to permit periodic testing of all components important to safety while the unit is at or above rated speed. In DCD Section 10.2.2.7, a list of components that may be tested with the unit at load is provided. However, some load reduction may be necessary before testing main stop and control valves, and intermediate stop and intercept valves (see DCD Section 10.2.3.7). Overspeed trip testing is performed at speed levels greater than or equal to rated speed with no electrical load. Thus, not all components are capable of being tested at rated load as required in the corresponding Acceptance Criterion. Load reduction for turbine valve testing is common in the existing fleet of power reactors and is considered acceptable. Testing at turbine loads below the rated load condition is considered an acceptable means of confirming that equipment relied on to prevent turbine overspeed related failures is available and capable of providing required functions. Further, component redundancies, as described in DCD Section 10.2.2.4, ensure that a single failure of any of the above valves important to safety will not disable the function of the overspeed protection system.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
10.2	Turbine Generator (continued)			II.2.A	Exception: Inservice inspection of main steam and reheat valves is discussed in DCD Sections 10.2.2.7 and 10.2.3.7. The first disassembly and visual inspection of all main stop valves, main control valves, intermediate stop, and intercept valves are performed within the first three refueling shutdowns. However, the interval for subsequent inspections may be extended beyond the SRP interval of 3-1/3 years to an interval consistent with applicable industry guidance, subject to the requirements of the turbine missile probability analysis. The inspection interval may not exceed the requirements or assumptions in the turbine missile probability analysis. Further, inspection intervals are only extended if there are no significant findings in the initial (baseline) inspections. Thus, with the above provisions, extending the inspection interval beyond the SRP interval is considered acceptable.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
10.2.3	Turbine Rotor Integrity	Rev. 2	03/2007	II.1, II.2 II.3.B, II.3.C, II.3.D, II.4, II.5	Conforms
				II.3.A	Exception: DCD Section 10.2.3.5 states that, "Forgings are rough-machined with minimum stock allowance prior to heat treatment." This statement meets the intent of the corresponding SRP Acceptance Criterion. The exception to the Acceptance Criterion is introduced with the reference to welded rotors. The GE N3R steam turbine selected for this site uses integral forgings in the rotor design and fabrication. Although other manufacturers produce welded rotors, the GE N3R rotor is not a welded rotor design and does not use welding to construct the base rotor. Flaws in the forging may be repaired by welding and other means, but only after heat treatment. Thus, the intent of this Acceptance Criterion is met.
10.3	Main Steam Supply System	Rev. 4	03/2007	II.1, II.2, II.5, II.6, II.7, II.8	Conforms
				II.3	Conforms. The main steam supply system is not a shared system.
				II.4	Not applicable to the ESBWR
10.3.6	Steam and Feedwater System Materials	Rev. 3	03/2007	II.1, II.2	Conforms
10.4.1	Main Condensers	Rev. 3	03/2007	II.1	Conforms
10.4.2	Main Condenser Evacuation System	Rev. 3	03/2007	II.1	Conforms
10.4.3	Turbine Gland Sealing System	Rev. 3	03/2007	No specific acceptance criteria	Conforms
10.4.4	Turbine Bypass System	Rev. 3	03/2007	II.1, II.2, II.3	Conforms

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
10.4.5	Circulating Water System	Rev. 3	03/2007	II.1	Conforms
10.4.6	Condensate Cleanup System	Rev. 3	03/2007	II.1	Conforms
				II.2	Not applicable to the ESBWR
10.4.7	Condensate and Feedwater System	Rev. 4	03/2007	II.1, II.2.B, II.4, II.5, II.6	Conforms
				II.2.A	Not applicable to the ESBWR
				II.3	Conforms. The condensate and feedwater system is not a shared system.
				II.7	Exception: This SRP Acceptance Criterion states that guidance for acceptable FAC inspection programs "is found in (NRC) Generic Letter 89-08 and in EPRI NP-3944." EPRI document NSAC-202L, Rev. 2, supersedes EPRI NP-3944 and is therefore referenced in place of EPRI NP-3944 in DCD Section 6.6.7, for guidance regarding FAC (erosion corrosion) monitoring and related inspection programs. The more recent document, EPRI NSAC-202L, uses more extensive industry experience and improved inspection methods and modeling. The substitution of EPRI NSAC-202L, Rev. 2, in place of EPRI NP-3944 is therefore acceptable.
				II.8	Conforms. Addressed in DCD Sections 3.9.3, 5.2.4, and 10.4.7, and DCD Tables 1.9-22 and 1.11-1.
10.4.8	Steam Generator Blowdown System (PWR)	Rev. 3	03/2007		Not applicable to the ESBWR
10.4.9	Auxiliary Feedwater System (PWR)	Rev. 3	03/2007		Not applicable to the ESBWR

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
BTP 10-1	Design Guidelines for Auxiliary Feedwater System Pump Drive and Power Supply Diversity for Pressurized Water Reactor Plants	Rev. 3	03/2007		Not applicable to the ESBWR
BTP 10-2	Design Guidelines for Avoiding Water Hammers in Steam Generators	Rev. 4	03/2007		Not applicable to the ESBWR
11.1	Source Terms	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.6, II.7, II.8, II.9	Conforms. Addressed in DCD Section 12.2 and in Section 12.2.
				II.5	Conforms. Addressed in Section 11.2 and 11.3.
11.2	Liquid Waste Management System	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5	Conforms. Addressed in DCD Sections 11.2 and 12.2, and in Section 11.2 and 12.2.
				II.6	Not applicable. Applies to ESP applications.
11.3	Gaseous Waste Management System	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7	Conforms. Addressed in DCD Sections 11.3 and 12.2, and in Section 11.3 and 12.2.
				II.8	Not applicable. Applies to ESP applications.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
11.4	Solid Waste Management System	Rev. 3	03/2007	II.1, II.2, II.5, II.7, II.8, II.9, II.14	Conforms
				II.3, II.4, II.6, II.11. II.12, II.13	Conforms (addressed in DCD Section 11.4 and in Section 11.4; for Acceptance Criterion II.13, this is also addressed in Section 11.5) with the following exception: RG 1.206, Section 13.4 includes the PCP as an operational program, and only requires a program description in the COLA and a milestone for full program implementation. The FSAR provides a description of the PCP, along with the implementation milestone. Procedures for handling waste will be developed once the PCP is implemented.
				II.10	Not applicable. There is no temporary onsite storage facility.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
11.5	Process and Effluent Radiological Monitoring Instrumentation and Sampling Systems	Rev. 4	03/2007	II.1, II.2	Conforms (addressed in DCD Section 11.5.2) with the following exception: Procedural controls are based on NQA-1, rather than RG 1.33, as described in Section 13.5. Quality Assurance Program requirements are addressed in Section 17.5.
				II.3, II.4, II.5	Conforms (addressed in DCD Sections 11.5.2 and 11.5.3, and in Section 11.5) with the following exceptions: 1) RG 1.206, Section 13.4 includes the ODCM (including the SREC) and PCP as operational programs, and only requires program descriptions in the COLA and milestones for full program implementation. The FSAR provides descriptions of the PCP and ODCM along with implementation milestones. 2) Procedural controls are based on NQA-1, rather than RG 1.33, as described in Section 13.5. Quality Assurance Program requirements are addressed in Section 17.5. Conformance with NUREG-0718 is addressed in DCD Table 1.9-8.
				II.6	Conforms

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
BTP 11-3	Design Guidance for Solid Radioactive Waste Management Systems Installed in Light-Water-Cooled Nuclear Power Reactor Plants	Rev. 3	03/2007	B.1, B.3, B.5	Conforms
				B.2, B.4	Conforms (addressed in DCD Section 11.4 and in Section 11.4; for Acceptance Criterion II.13, this is also addressed in Section 11.5) with the following exception: RG 1.206, Section 13.4 includes the PCP as an operational program, and only requires a program description in the COLA and a milestone for full program implementation. The FSAR provides a description of the PCP, along with the implementation milestone. Procedures for handling waste will be developed once the PCP is implemented.
BTP 11-5	Postulated Radioactive Releases Due to a Waste Gas System Leak or Failure	Rev. 3	03/2007		Conforms. Addressed in DCD Section 11.3.
BTP 11-6	Postulated Radioactive Releases Due to Liquid-Containing Tank Failures	Rev. 3	03/2007		Conforms. Addressed in DCD Section 15.3.16 and in Section 2.4.1.3.
12.1	Assuring that Occupational Radiation Exposures Are As Low As Is Reasonably Achievable	Rev. 3	03/2007	II.1, II.2, II.3, II.4	Conforms. Addressed in Section 13.2, and Appendix 12AA and 12BB.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
12.2	Radiation Sources	Rev. 3	03/2007	II.1	Not applicable. Acceptance Criterion cites RG 1.3. SRP states RG 1.3 is applicable to license holders issued before January 10, 1997. COL applicant is not a license holder.
				II.2	Not applicable to the ESBWR
				II.3	Conforms. Addressed in DCD Sections 12.3 and 15.4 and in Section 6.4.
				II.4	Conforms. Addressed in DCD Section 12.3.
				II.5	Conforms
				II.6	Conforms. Addresses in DCD Sections 1A and 12.2.
				II.7	Conforms. Addressed in DCD Section 12.2.
12.3–12.4	Radiation Protection Design Features	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5	Conforms

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
12.5	Operational Radiation Protection Program	Rev. 3	03/2007	II.1	Conforms with the following exceptions: 1) NUREG-0731 is not active and is not used; 2) RG 8.8 specifies the use of RG 1.16. Reporting per C.1.b(2) and C.1.b(3) of RG 1.16 is no longer required.
				II.2.A, II.2.B, II.2.C, II.2.D, II.2.E.i, II.2.E.ii, II.2.E.iii, II.2.E.iv, II.2.F, II.2.G, II.2.H, II.4	Conforms
				II.2.E.v	Conforms with the following exception: NUREG-1736 states that RGs 8.20, 8.26, and 8.32 are outdated and recommends use of the methods in RG 8.9, Rev. 1. Therefore, the methods identified in RG 8.9, Rev. 1 will be used in place of those in RGs 8.20, 8.26, and 8.32.
				II.3	Conforms with the following exceptions: 1) RG 8.25 is not applicable to power stations; 2) NUREG-1736 states that RGs 8.20, 8.26, and 8.32 are outdated and recommends use of the methods in RG 8.9, Rev. 1; and 3) RP program and procedures are established, implemented, maintained, and reviewed under the QA Program described in Section 17.5.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
13.1.1	Management and Technical Support Organization	Rev. 5	03/2007	II.1.A, II.1.B, II.1.D, II.2.A.i through II.2.A.v	Conforms
				II.1.C	Exception: The experience requirements of corporate staff are set by corporate policy and not provided in detail; however, the experience level of Exelon, as discussed in Section 13.1 and Appendix 13AA, in the area of nuclear plant development, construction, and management establishes that Exelon has the necessary capability and staff to ensure that design and construction of the facility will be performed in an acceptable manner.
				II.2.A.vi, II.2.A.vii	Conforms. Addressed in Sections 13.1 and 14.2.
				II.2.A.viii	Not applicable. Only applies to applicants whose applications were pending as of February 16, 1982.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
13.1.2–13.1.3	Operating Organization	Rev. 6	03/2007	General 1, General 2, General 3	Conforms
				General 4	Not applicable. There are no requests for exemptions from the requirements of 10 CFR 50.54(m).
				II.1.A, II.1.B	Conforms with the following exception: Quality assurance is in accordance with the QAPD. QA requirements as they apply to the operating organization and onsite review are described in the QAPD. Responsibilities and authorities of operating personnel conform to the guidance of ANSI/ANS-3.2-1994 (R1999). Rules of practice, fire protection, RG 1.8 and TMI item I.C.3 are addressed in Section 13.1.
				II.1.A.i through II.1.A.v, II.1.C, II.1.E, II.1.F, II.1.G	Conforms
				II.1.D	Not applicable
				II.1.H	Conforms. Addressed in Section 13.2.
13.2.1	Reactor Operator Requalification Program: Reactor Operator Training	Rev. 3	03/2007	II.1.A.i	Conforms. Addressed in Section 13.1.
				II.1.A.ii, II.1.A.iii, II.1.A.v, II.1.B, II.1.C, II.1.D, II.1.E	Conforms
				II.1.A.iv	Conforms. Addressed in Sections 13.1, 13.2, and 17.5.
				II.1.A.vi	Conforms. Addressed in DCD Chapter 18.
				II.1.A.vii	Exception: The COLA incorporates by reference approved industry template NEI 06-13, which does not address compliance with NUREG-1021.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
13.2.2	Non-Licensed Plant Staff Training	Rev. 3	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9	Conforms
				II.10	Conforms. Addressed in DCD Section 9.5.1.
				II.11	Conforms. Addressed in Sections 13.2 and 13.4.
13.3	Emergency Planning	Rev. 3	03/2007	II.1, II.2	Conforms. Addressed in Section 13.4, COLA Part 5, and COLA Part 10.
				II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12, II.13, II.17, II.18, II.27, II.28, II.29, II.30	Conforms. Addressed in COLA Part 5.
				II.14	Not applicable. Allows NRC to issue a license when applicant asserts that noncompliance with offsite EP requirements is because state or local government has declined to participate in emergency planning.
				II.15, II.16, II.19, II.20, II.21	Not applicable. Only applies to ESP applications.
				II.22	Not applicable. Only applies to design certification applications.
				II.23, II.24	Conforms. Addressed in COLA Part 10.
				II.25	Conforms. Addressed in DCD Section 13.3 and COLA Part 5.
				II.26	Conforms. Reviewed under SRPs 7.5 and 18.2.
				II.31	Conforms. Addressed in Section 13.4.
13.4	Operational Programs	Rev. 3	03/2007		Refers to other SRP sections

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
13.5.1.1	Administrative Procedures - General	Initial Issuance	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7 II.9, II.10, II.11, II.12, II.13, II.14, II.15, II.16, II.17, II.18, II.19, II.20	Conforms
				II.8	Conforms. Section 13.5 and DCD Section 18.9 discuss conformance with NUREG-0711.
13.5.2.1	Operating and Emergency Operating Procedures	Rev. 2	03/2007	II.1 II.2.A, II.2.B II.2.D, II.2.E, II.2.F, II.2.G, II.2.H, II.2.I	Conforms
				II.2.C	Conforms. Section 13.5 and DCD Section 18.9 discuss conformance with NUREG-0711
13.6	Physical Security	Rev. 3	03/2007		Not applicable. Refers to SRP Section 13.6.1.
13.6.1	Physical Security - Combined License Review Responsibilities	Initial Issuance	03/2007	II.1, II.2, II.3, II.4	Addressed in COLA Part 8.
13.6.2	Physical Security - Design Certification	Initial Issuance	03/2007		Not applicable. Applies to design certification applications.
13.6.3	Physical Security - Early Site Permit	Initial Issuance	03/2007		Not applicable. Applies to ESP applications.
14.2	Initial Plant Test Program - Design Certification and New License Applicants	Rev. 3	03/2007	II.1.A, II.1.B, II.1.C, II.2.A, COL/OL Applicants: II.3.A, II.3.B, II.3.C, II.3.D, II.3.E, II.3.F, II.3.G, II.3.H, II.4.A, II.4.B, II.5.A, II.5.B, II.5.C, II.5.D, II.6.A, II.6.B, II.6.C	Conforms
				DC Applicants: II.3.A, II.3.B, II.3.C, II.3.D, II.4.A, II.6.A, II.6.B, II.6.C	Not applicable. Applies to DC applicants.
14.2.1	Generic Guidelines for Extended Power Uprate Testing Programs	Initial Issuance	08/2006		Not applicable. Applies to power uprates.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
14.3	Inspections, Tests, Analyses, and Acceptance Criteria	Initial Issuance	03/2007	II.1, II.2	Conforms
14.3.1	[Reserved]	[Reserved]	03/2007		Not used
14.3.2	Structural and Systems Engineering - Inspections, Tests, Analyses, and Acceptance Criteria	Initial Issuance	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11	Conforms
14.3.3	Piping Systems and Components - Inspections, Tests, Analyses, and Acceptance Criteria	Initial Issuance	03/2007	II.1, II.2.A, II.2.B, II.2.C, II.2.D, II.2.E	Conforms
14.3.4	Reactor Systems - Inspections, Tests, Analyses, and Acceptance Criteria	Initial Issuance	03/2007	II.1, II.2, II.3, II.4, II.5	Conforms
14.3.5	Instrumentation and Controls - Inspections, Tests, Analyses, and Acceptance Criteria	Initial Issuance	03/2007	II.1, II.2, II.3, II.4, II.5	Conforms
14.3.6	Electrical Systems - Inspections, Tests, Analyses, and Acceptance Criteria	Initial Issuance	03/2007	Class 1E Equipment: II.1, II.2, II.3, II.4, II.5 Other Electrical Equipment Important to Safety: II.1, II.2, II.3, II.4, II.5	Conforms
14.3.7	Plant Systems - Inspections, Tests, Analyses, and Acceptance Criteria	Initial Issuance	03/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9	Conforms

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
14.3.8	Radiation Protection - Inspections, Tests, Analyses, and Acceptance Criteria	Initial Issuance	03/2007	II.1, II.2, II.3	Conforms
14.3.9	Human Factors Engineering - Inspections, Tests, Analyses, and Acceptance Criteria	Initial Issuance	03/2007	II.1, II.2, II.3, II.4, II.5, II.6	Conforms
14.3.10	Emergency Planning - Inspections, Tests, Analyses, and Acceptance Criteria	Initial Issuance	03/2007	II.1, II.2	Conforms
14.3.11	Containment Systems - Inspections, Tests, Analyses, and Acceptance Criteria	Initial Issuance	03/2007	II.1, II.2, II.3, II.4, II.5	Conforms
14.3.12	Physical Security Hardware - Inspections, Tests, Analyses, and Acceptance Criteria	Initial Issuance	03/2007	II.1	Conforms
15	Introduction - Transient and Accident Analyses	Rev. 3	03/2007	I.1, I.2, 1.3, I.4, I.5, I.6	Conforms
15.0.1	Radiological Consequence Analyses Using Alternative Source Terms	Rev. 0	07/2000	No enumerated acceptance criteria	Conforms
15.0.2	Review of Transient and Accident Analysis Method	Rev. 0	12/2005	II.1, II.2, II.3, II.4, II.5, II.6	Conforms
15.0.3	Design Basis Accident Radiological Consequences of Analyses for Advanced Light Water Reactors	Initial Issuance	03/2007		Not applicable to the ESBWR

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
15.1.1– 15.1.4	Decrease in Feedwater Temperature, Increase in Feedwater Flow, Increase in Steam Flow, and Inadvertent Opening of a Steam Generator Relief or Safety Valve	Rev. 2	03/2007	II.1, II.2, II.3, II.4, II.5, 1, 2, 3, 4	Conforms
15.1.5	Steam System Piping Failures Inside and Outside of Containment (PWR)	Rev. 3	03/2007		Not applicable to the ESBWR
15.1.5.A	Radiological Consequences of Main Steam Line Failures Outside Containment of a PWR	Rev. 2	07/1981		Not applicable to the ESBWR
15.2.1– 15.2.5	Loss of External Load; Turbine Trip; Loss of Condenser Vacuum; Closure of Main Steam Isolation Valve (BWR); and Steam Pressure Regulator Failure (Closed)	Rev. 2	03/2007	II.1.A, II.1.B, II.1.C, II.1.D, II.2.A, II.2.B, II.2.D, II.2.E, II.2.F, II.3.A, II.3.B, II.3.C, II.3.D	Conforms
				II.2.C	Not applicable. This is not an event of moderate frequency.
15.2.6	Loss of Nonemergency AC Power to the Station Auxiliaries	Rev. 2	03/2007	II.1, II.2, II.4, II.5, II.5.B, II.5.C, II.5.D	Conforms
				II.3	Not applicable. This is not an event of moderate frequency.
				II.5.A	Not applicable. There are no RCS loops in the ESBWR.
15.2.7	Loss of Normal Feedwater Flow	Rev. 2	03/2007	II.1.A, II.1.B, II.1.C, II.1.D, II.2.A, II.2.B, II.2.D, II.2.E, II.2.F, II.3.A, II.3.B, II.3.C, II.3.D	Conforms
				II.2.C	Not applicable. This is not an event of moderate frequency.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
15.2.8	Feedwater System Pipe Breaks Inside and Outside Containment (PWR)	Rev. 2	03/2007		Not applicable to the ESBWR
15.3.1–15.3.2	Loss of Forced Reactor Coolant Flow Including Trip of Pump Motor and Flow Controller Malfunctions	Rev. 2	03/2007		Not applicable to the ESBWR
15.3.3–15.3.4	Reactor Coolant Pump Rotor Seizure and Reactor Coolant Pump Shaft Break	Rev. 3	03/2007		Not applicable to the ESBWR
15.4.1	Uncontrolled Control Rod Assembly Withdrawal from a Subcritical or Low Power Startup Condition	Rev. 3	03/2007	II.1.A, II.1.C	Conforms
				II.1.B	Not applicable to the ESBWR
15.4.2	Uncontrolled Control Rod Assembly Withdrawal at Power	Rev. 3	03/2007	II.1.A, II.1.C	Conforms
				II.1.B	Not applicable to the ESBWR
15.4.3	Control Rod Misoperation (System Malfunction or Operator Error)	Rev. 3	03/2007	II.1, II.2, II.3	Conforms
15.4.4 - 15.4.5	Startup of an Inactive Loop or Recirculation Loop at an Incorrect Temperature, and Flow Controller Malfunction Causing an Increase in BWR Core Flow Rate	Rev. 2	03/2007	II.A, II.B, II.D, II.E, II.F, II.1, II.2, II.3, II.4	Conforms
				II.C	Not applicable. This is not an event of moderate frequency.
15.4.6	Inadvertent Decrease in Boron Concentration in the Reactor Coolant System (PWR)	Rev. 2	03/2007		Not applicable to the ESBWR

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
15.4.7	Inadvertent Loading and Operation of a Fuel Assembly in an Improper Position	Rev. 2	03/2007	II.1, II.2	Conforms
15.4.8	Spectrum of Rod Ejection Accidents (PWR)	Rev. 3	03/2007		Not applicable to the ESBWR
15.4.8.A	Radiological Consequences of a Control Rod Ejection Accident (PWR)	Rev. 1	07/1981		Not applicable to the ESBWR
15.4.9	Spectrum of Rod Drop Accidents (BWR)	Rev. 3	03/2007	II.1, II.2, II.3	Conforms. Postulated events are not applicable to the ESBWR.
15.4.9.A	Radiological Consequences of Control Rod Drop Accident (BWR)	Rev 2	07/1981		Conforms. Postulated Control Rod Drop events are not applicable to the ESBWR.
15.5.1–15.5.2	Inadvertent Operation of ECCS and Chemical and Volume Control System Malfunction that Increases Reactor Coolant Inventory	Rev. 2	03/2007	II.1, II.2, II.3	Conforms
15.6.1	Inadvertent Opening of a PWR Pressurizer Pressure Relief Valve or a BWR Pressure Relief Valve	Rev. 2	03/2007	II.1, II.2, II.3, II.A, II.B, II.C, II.D	Conforms
15.6.2	Radiological Consequences of the Failure of Small Lines Carrying Primary Coolant Outside Containment	Rev. 2	07/1981	II.1, II.2	Conforms
15.6.3	Radiological Consequences of Steam Generator Tube Failure	Rev. 2	07/1981		Not applicable to the ESBWR

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
15.6.4	Radiological Consequences of Main Steam Line Failure Outside Containment (BWR)	Rev. 2	07/1981	II.1, II.2, II.3	Conforms
				II.4	Conforms. Addressed in TS 3.4.3.
15.6.5	Loss-of-Coolant Accidents Resulting From Spectrum of Postulated Piping Breaks Within the Reactor Coolant Pressure Boundary	Rev. 3	03/2007	II.1.A, II.1.B, II.1.C, II.1.D, II.1.E, II.2, II.3	Conforms
15.6.5.A	Radiological Consequences of a Design Basis Loss-of-Coolant Accident Including Containment Leakage Contribution	Rev 1	07/1981		Not Applicable. Reference DCD Table 1.9-20.
15.6.5.B	Radiological Consequences of a Design Basis Loss-of-Coolant Accident: Leakage From Engineered Safety Feature Components Outside Containment	Rev 1	07/1981		Not Applicable. Reference DCD Table 1.9-20.
15.6.5.D	Radiological Consequences of a Design Basis Loss-of-Coolant Accident: Leakage From Main Steam Isolation Valve Leakage Control System (BWR)	Rev 1	07/1981		Not Applicable. Reference DCD Table 1.9-20.
15.7.3	Postulated Radioactive Releases Due to Liquid-Containing Tank Failures	Rev. 2	07/1981	II.1, II.2	Conforms

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
15.7.4	Radiological Consequences of Fuel Handling Accidents	Rev. 1	07/1981	II.1, II.2, II.3, II.4, II.5	Conforms. Radiological assumptions superseded by SRP 15.0.1.
15.7.5	Spent Fuel Cask Drop Accidents	Rev. 2	07/1981	II.1, II.2, II.3, II.4, II.5	Conforms. Because a spent fuel cask drop exceeding 9.2 meters (30 feet) is not postulated (DCD Section 15.4.10.1), per SRP 15.7.5 a design basis radiological analysis is not required. Therefore, the acceptance criteria do not apply even though the SRP does.
15.8	Anticipated Transients Without Scram	Rev. 2	03/2007	1A	Not applicable. ESBWR does not have recirculation pumps.
				1B, 1C, 1D, 1E, 1F	Conforms
15.9	Boiling Water Reactor Stability	Initial Issuance	03/2007	II.1, II.2, II.3, II.4.A, II.4.B, II.5, II.6, II.7, II.8, II.9.A, II.9.B, II.9.C, II.9.D, II.10, II.11	Conforms
16	Technical Specifications	Rev. 2	03/2007		Conforms
16.1	Risk-informed Decision Making: Technical Specifications	Rev. 1	03/2007		Not applicable
17.1	Quality Assurance During the Design and Construction Phases	Rev. 2	07/1981		Not applicable. RG 1.206 refers the COL applicant to Section 17.5 for the format and content of a QA Program for design and construction of new plants.
17.2	Quality Assurance During the Operations Phase	Rev. 2	07/1981		Not applicable. RG 1.206 refers the COL applicant to Section 17.5 for the format and content of a QA Program for design and construction of new plants.
17.3	Quality Assurance Program Description	Rev. 0	08/1990		Not applicable. RG 1.206 refers the COL applicant to Section 17.5 for the format and content of a QA Program for design and construction of new plants.

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SRP Section	Title	Rev	Date	Specific Acceptance Criteria	Evaluation
17.4	Reliability Assurance Program (RAP)	Initial Issuance	03/2007	II.B.1, II.B.2, II.B.3, II.B.4, II.B.5, II.B.6, II.B.7, II.B.8, II.B.9	Conforms. Addressed in DCD Sections 17.4 and 17.6.
17.5	Quality Assurance Program Description - Design Certification, Early Site Permit and New License Applicants	Initial Issuance	03/2007	II.A, II.B, II.C, II.D., II.E, II.F, II.G, II.H, II.I, II.J, II.K, II.L, II.M, II.N, II.O, II.P, II.Q, II.R, II.S, II.T, II.U, II.V, II.W Option I	Conforms
				II.W Option II	Not applicable for VCS. Option I chosen.
17.6	Maintenance Rule	Rev. 1	08/2007	II.1, II.2	Conforms
18	Human Factors Engineering	Rev. 2	03/2007	II.A	Conforms
				II.B, II.C	Not applicable. These acceptance criteria apply to changes to existing plants.
19.0	Probabilistic Risk Assessment and Severe Accident Evaluation for New Reactors	Rev. 2	06/2007	II.1, II.2, II.3, II.4, II.5, II.6, II.7	Conforms
				II.8, II.9	Not applicable. Only applies to Westinghouse AP 600 design.
19.1	Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities	Rev. 2	06/2007		Not applicable. There are no plans for risk-informed activities.
19.2	Review of Risk Information Used to Support Permanent Plant Specific Changes to the Licensing Basis: General Guidelines	Initial Issuance	06/2007		Not applicable. There are no plans for risk-informed applications.

VCS COL 1.9-3-A

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Conformance with Regulatory Guides

RG Number	Title	Revision	Date	RG Position	Evaluation
1.1	Net Positive Suction Head for Emergency Core Cooling and Containment Heat Removal System Pumps	Rev. 0	11/1970	General	Not applicable
1.3	Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss of Coolant Accident for Boiling Water Reactors	Rev. 2	06/1974	General	Not applicable. RG 1.183 is used.
1.4	Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss of Coolant Accident for Pressurized Water Reactors	Rev. 2	06/1974	General	Not applicable
1.5	Assumptions Used for Evaluating the Potential Radiological Consequences of a Steam Line Break Accident for Boiling Water Reactors	Rev. 0	03/1971	General	Not applicable. RG 1.183 is used.
1.6	Independence Between Redundant Standby (Onsite) Power Sources and Between Their Distribution Systems	Rev. 0	03/1971	General	Not applicable
1.7	Control of Combustible Gas Concentrations in Containment	Rev. 3	03/2007	General	Conforms
1.8	Qualification and Training of Personnel for Nuclear Power Plants	Rev. 3	05/2000	C.1	Conforms
				C.2	Conforms, except experience requirements cannot be met prior to operations as described in Appendix 13BB.
1.9	Application and Testing of Safety-Related Diesel Generators in Nuclear Power Plants	Rev. 4	03/2007	General	Not applicable

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RG Number	Title	Revision	Date	RG Position	Evaluation
1.11	Instrument Lines Penetrating Primary Reactor Containment (Safety Guide 11)	Rev. 0	03/1971	C.1, C.2	Conforms
	Supplement to Safety Guide 11, Backfitting Considerations	Rev. 0	02/1972	E	Conforms
1.12	Nuclear Power Plant Instrumentation for Earthquakes	Rev. 2	03/1997	C.1, C.2, C.4–C.7	Conforms
				C.3, C.8	Conforms. The seismic monitoring program, including the necessary test and operating procedures, will be implemented before receipt of fuel on site.
1.13	Spent Fuel Storage Facility Design Basis	Rev. 2	03/2007	General	Conforms
1.14	Reactor Coolant Pump Flywheel Integrity	Rev. 1	08/1975	General	Not applicable
1.16	Reporting of Operating Information–Appendix A Technical Specifications	Rev. 4	08/1975	General	Conforms
1.20	Comprehensive Vibration Assessment Program for Reactor Internals During Preoperational and Initial Startup Testing	Rev. 3	03/2007	C.1	Conforms
				C.2	Not applicable. VCS does not have prototype reactor internals.
				C.3	Conforms. Section 3.9.2.4 describes that the Vibration Assessment Program will be completed one year after the time of application.
1.21	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants	Rev. 1	06/1974	General	Conforms. Sections 11.4.2.3 (NEI 07-10) and 11.5.4.5 (NEI 07-09) provide descriptions of the PCP and ODCM, respectively. Implementation milestones are provided in Section 13.4.
1.22	Periodic Testing of Protection System Actuation Functions	Rev. 0	02/1972	General	Conforms. Operational program implementation is described in Section 13.4.

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RG Number	Title	Revision	Date	RG Position	Evaluation
1.23	Meteorological Monitoring Programs for Nuclear Power Plants	Rev. 1	03/2007	General	Conforms
1.24	Assumptions Used for Evaluating the Potential Radiological Consequences of a Pressurized Water Reactor Radioactive Gas Storage Tank Failure	Rev. 0	03/1972	All	Not applicable
1.25	Assumptions Used for Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facility for Boiling and Pressurized Water Reactors	Rev. 0	03/1972	General	Not applicable. RG 1.183 is used.
1.26	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-Waste-Containin g Components of Nuclear Power Plants	Rev. 4	03/2007	All	Conforms
1.27	Ultimate Heat Sink for Nuclear Power Plants	Rev. 2	01/1976	General	The UHS is within the scope of the referenced certified design and is addressed in DCD Section 9.2.5.
1.28	Quality Assurance Program Requirements (Design and Construction)	Rev. 3	08/1985	General	Exception: The QAPD identified in Section 17.5 addresses a QA program based on the newer NQA-1-1994, as provided for in SRP 17.5.
1.29	Seismic Design Classification	Rev. 4	03/2007	General	Conforms
1.30	Quality Assurance Requirements for the Installation, Inspection, and Testing of Instrumentation and Electric Equipment	Rev. 0	08/1972	General	Exception: The QAPD identified in Section 17.5 addresses a QA program based on a newer NQA-1-1994, as discussed in SRP 17.5.

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RG Number	Title	Revision	Date	RG Position	Evaluation
1.31	Control of Ferrite Content in Stainless Steel Weld Metal	Rev. 3	04/1978	General	Conforms. Operational program implementation is described in Section 13.4.
1.32	Criteria for Power Systems for Nuclear Power Plants	Rev. 3	03/2004	General	Conforms
1.33	Quality Assurance Program Requirements (Operation)	Rev. 2	02/1978	General	Exception: The QAPD topical report identified in Section 17.5 follows NQA-1 rather than the older standards referenced in RG 1.33.
1.34	Control of Electroslag Weld Properties	Rev. 0	12/1972	General	Conforms. Operational program implementation is described in Section 13.4.
1.35	Inservice Inspection of Ungrouted Tendons in Prestressed Concrete Containments	Rev. 3	07/1990	General	Not applicable
1.35.1	Determining Prestressing for Inspection of Prestressed Concrete Containments	Rev. 0	07/1990	General	Not applicable
1.36	Nonmetalic Thermal Insulation for Austenitic Stainless Steel	Rev. 0	02/1973	General	Conforms. Operational program implementation is described in Section 13.4.
1.37	Quality Assurance Requirements for Cleaning of Fluid Systems and Associated Components of Water-Cooled Nuclear Power Plants	Rev. 1	03/2007	General	Conforms
1.38	Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage, and Handling of Items for Water-Cooled Nuclear Power Plants	Rev. 2	05/1977	General	Exception: Section 17.5 identifies equivalent quality assurance standards.
1.39	Housekeeping Requirements for Water-Cooled Nuclear Power Plants	Rev. 2	09/1977	General	Exception: Section 17.5 identifies equivalent quality assurance standards.

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Conformance with Regulatory Guides

RG Number	Title	Revision	Date	RG Position	Evaluation
1.40	Qualification Tests of Continuous-Duty Motors Installed Inside the Containment of Water-Cooled Nuclear Power Plants	Rev. 0	03/1973	General	Not applicable
1.41	Preoperational Testing of Redundant On-Site Electric Power Systems to Verify Proper Load Group Assignments	Rev. 0	03/1973	General	Conforms with the following exception: there are no safety-related diesel generators for the ESBWR.
1.43	Control of Stainless Steel Weld Cladding of Low-Alloy Steel Components	Rev. 0	05/1973	General	Conforms
1.44	Control of the Use of Sensitized Stainless Steel	Rev. 0	05/1973	General	Conforms. Operational program implementation is described in Section 13.4.
1.45	Reactor Coolant Pressure Boundary Leakage Detection Systems	Rev. 0	05/1973	General	Conforms. Operational program implementation is described in Section 13.4.
1.47	Bypassed and Inoperable Status Indication for Nuclear Power Plant Safety Systems	Rev. 0	05/1973	General	Conforms. Operational program implementation is described in Section 13.4.
1.50	Control of Preheat Temperature for Welding of Low-Alloy Steel	Rev. 0	05/1973	General	Conforms. Operational program implementation is described in Section 13.4.
1.52	Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Post-Accident Engineered-Safety-Feature Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants	Rev. 3	06/2001	General	Conforms
1.53	Application of the Single-Failure Criterion to Safety Systems	Rev. 2	11/2003	General	Conforms
1.54	Service Level I, II, and III Protective Coatings Applied to Nuclear Power Plants	Rev. 1	07/2000	General	Conforms
1.56	Maintenance of Water Purity in Boiling Water Reactors	Rev. 1	07/1978	General	Conforms

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Conformance with Regulatory Guides

RG Number	Title	Revision	Date	RG Position	Evaluation
1.57	Design Limits and Loading Combinations for Metal Primary Reactor Containment System Components	Rev. 1	03/2007	General	Conforms
1.59	Design Basis Floods for Nuclear Power Plants (Errata Published 7/30/80)	Rev. 2	08/1977	General	Conforms
1.60	Design Response Spectra for Seismic Design of Nuclear Power Plants	Rev. 1	12/1973	General	Conforms
1.61	Damping Values for Seismic Design of Nuclear Power Plants	Rev. 1	03/2007	General	Conforms
1.62	Manual Initiation of Protective Actions	Rev. 0	10/1973	General	Conforms
1.63	Electric Penetration Assemblies in Containment Structures for Nuclear Power Plants	Rev. 3	02/1987	General	Conforms
1.65	Materials and Inspections for Reactor Vessel Closure Studs	Rev. 0	10/1973	General	Conforms
1.68	Initial Test Programs for Water-Cooled Nuclear Power Plants	Rev. 3	03/2007	General	Conforms
1.68.1	Preoperational and Initial Startup Testing of Feedwater and Condensate Systems for Boiling Water Reactor Power Plants	Rev. 1	01/1977	General	Conforms
1.68.2	Initial Startup Test Program to Demonstrate Remote Shutdown Capability for Water-Cooled Nuclear Power Plants	Rev. 1	07/1978	General	Conforms
1.68.3	Preoperational Testing of Instrument and Control Air Systems	Rev. 0	04/1982	General	Conforms
1.69	Concrete Radiation Shields for Nuclear Power Plants	Rev. 0	12/1973	General	Conforms

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RG Number	Title	Revision	Date	RG Position	Evaluation
1.70	Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants LWR Edition	Rev. 3	11/1978	—	Not applicable. RG 1.206 is used, see Table 1.9-203.
1.71	Welder Qualification for Areas of Limited Accessibility	Rev. 1	03/2007	General	Conforms. Operational program implementation is described in Section 13.4.
1.72	Spray Pond Piping Made from Fiberglass-Reinforced Thermosetting Resin	Rev. 2	11/1978	General	Not applicable
1.73	Qualification Tests of Electric Valve Operators Installed Inside the Containment of Nuclear Power Plants	Rev. 0	01/1974	General	Conforms
1.75	Criteria for Independence of Electrical Safety Systems	Rev. 3	02/2005	General	Conforms
1.76	Design Basis Tornado and Tornado Missiles for Nuclear Power Plants	Rev. 1	03/2007	General	Conforms
1.77	Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized Water Reactors	Rev. 0	05/1974	General	Not applicable
1.78	Evaluating the Habitability of a Nuclear Power Plant Control Room During a Postulated Hazardous Chemical Release	Rev. 1	12/2001	General	Conforms
1.79	Preoperational Testing of Emergency Core Cooling Systems for Pressurized Water Reactors	Rev. 1	09/1975	General	Not applicable
1.81	Shared Emergency and Shutdown Electric Systems for Multi-Unit Nuclear Power Plants	Rev. 1	01/1975	General	Conforms. The emergency and shutdown electric systems are not shared systems.
1.82	Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant Accident	Rev. 3	11/2003	General	Conforms

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Conformance with Regulatory Guides

RG Number	Title	Revision	Date	RG Position	Evaluation
1.83	Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes	Rev. 1	07/1975	General	Not applicable
1.84	Design, Fabrication, and Materials Code Case Acceptability, ASME Section III	Rev. 34	10/2007	General	Conforms
1.86	Termination of Operating Licenses for Nuclear Reactors	Rev. 0	06/1974	General	This RG is outside the scope of the FSAR.
1.87	Guidance for Construction of Class 1 Components in Elevated-Temperature Reactors (Supplement to ASME Section III Code Cases 1592, 1593, 1594, 1595, and 1596)	Rev. 1	06/1975	General	Not applicable
1.89	Environmental Qualification of Certain Electric Equipment Important to Safety for Nuclear Power Plants	Rev. 1	06/1984	General	Conforms. Source terms from RG 1.183 used.
1.90	Inservice Inspection of Prestressed Concrete Containment Structures with Grouted Tendons	Rev. 1	08/1977	General	Not applicable
1.91	Evaluations of Explosions Postulated to Occur on Transportation Routes Near Nuclear Power Plants	Rev. 1	02/1978	General	Conforms
1.92	Combining Modal Responses and Spatial Components in Seismic Response Analysis	Rev. 2	07/2006	General	Conforms
1.93	Availability of Electric Power Sources	Rev. 0	12/1974	General	Conforms. Operational program implementation is described in Section 13.4.

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RG Number	Title	Revision	Date	RG Position	Evaluation
1.94	Quality Assurance Requirements for Installation, Inspection, and Testing of Structural Concrete and Structural Steel During the Construction Phase of Nuclear Power Plants	Rev. 1	04/1976	General	Exception: Section 17.5 identifies equivalent QA standards in NQA-1, Subpart 2.5.
1.96	Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water Reactor Nuclear Power Plants	Rev. 1	06/1976	General	Not applicable
1.97	Criteria for Accident Monitoring Instrumentation for Nuclear Power Plants	Rev. 4	06/2006	General	Conforms. Operational program implementation is described in Section 13.4.
1.98	Assumptions Used for Evaluating the Potential Radiological Consequences of a Radioactive Offgas System Failure in a Boiling Water Reactor	Rev. 0	03/1976	General	Not applicable. Superseded by BTP 11-5.
1.99	Radiation Embrittlement of Reactor Vessel Materials	Rev. 2	05/1988	General	Conforms. Operational program implementation is described in Section 13.4.
1.100	Seismic Qualification of Electric and Mechanical Equipment for Nuclear Power Plants	Rev. 2	06/1988	General	Conforms
1.101	Emergency Response Planning and Preparedness for Nuclear Power Reactors	Rev. 5	06/2005	General	Not applicable
1.102	Flood Protection for Nuclear Power Plants	Rev. 1	09/1976	General	Conforms
1.105	Setpoints For Safety-Related Instrumentation	Rev. 3	12/1999	General	Conforms. Operational program implementation is described in Section 13.4.
1.106	Thermal Overload Protection for Electric Motors on Motor-Operated Valves	Rev. 1	03/1977	General	Not applicable

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RG Number	Title	Revision	Date	RG Position	Evaluation
1.107	Qualifications for Cement Grouting for Prestressing Tendons in Containment Structures	Rev. 1	02/1977	General	Not applicable
1.109	Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I	Rev. 1	10/1977	General	Conforms
1.110	Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power Reactors	Rev. 0	03/1976	General	Conforms
1.111	Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors	Rev. 1	07/1977	General	Conforms
1.112	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Light-Water-Cooled Nuclear Power Reactors	Rev. 1	03/2007	General	Conforms, except the suggested breakdown identified in Appendix A to the RG is not used because it is not consistent with the DCD presentation of information.
1.113	Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor Releases for the Purpose of Implementing Appendix I	Rev. 1	04/1977	General	Conforms
1.114	Guidance to Operators at the Controls and to Senior Operators in the Control Room of a Nuclear Power Unit	Rev. 2	05/1989	General	Conforms
1.115	Protection Against Low-Trajectory Turbine Missiles	Rev. 1	07/1977	General	Conforms

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Conformance with Regulatory Guides

RG Number	Title	Revision	Date	RG Position	Evaluation
1.116	Quality Assurance Requirements for Installation, Inspection, and Testing of Mechanical Equipment and Systems	Rev. 0-R	05/1977	General	Exception: Section 17.5 identifies equivalent QA standards in NQA-1, Subpart 2.8.
1.117	Tornado Design Classification	Rev. 1	04/1978	General	Conforms
1.118	Periodic Testing of Electric Power and Protection Systems	Rev. 3	04/1995	General	Conforms. Operational program implementation is described in Section 13.4.
1.121	Bases for Plugging Degraded PWR Steam Generator Tubes	Rev. 0	08/1976	General	Not applicable
1.122	Development of Floor Design Response Spectra for Seismic Design of Floor-Supported Equipment or Components	Rev. 1	02/1978	General	Conforms
1.124	Service Limits and Loading Combinations for Class 1 Linear-Type Supports	Rev. 2	02/2007	General	Conforms
1.125	Physical Models for Design and Operation of Hydraulic Structures and Systems for Nuclear Power Plants	Rev. 1	10/1978	General	Conforms
1.126	An Acceptable Model and Related Statistical Methods for the Analysis of Fuel Densification	Rev. 1	03/1978	General	Conforms
1.127	Inspection of Water-Control Structures Associated with Nuclear Power Plants	Rev. 1	03/1978	General	Conforms
1.128	Installation Design and Installation of Vented Lead-Acid Storage Batteries for Nuclear Power Plants	Rev. 2	02/2007	General	Not Applicable. The ESBWR uses valve regulated lead-acid batteries. IEEE 484 is not applicable.
1.129	Maintenance, Testing, and Replacement of Vented Lead-Acid Storage Batteries for Nuclear Power Plants	Rev. 2	02/2007	General	Not Applicable. The ESBWR uses valve regulated lead-acid batteries. IEEE 450 is not applicable.

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RG Number	Title	Revision	Date	RG Position	Evaluation
1.130	Service Limits and Loading Combinations for Class 1 Plate-and-Shell-Type Supports	Rev. 2	03/2007	General	Conforms
1.131	Qualification Tests of Electric Cables, Field Splices, and Connections for Light-Water-Cooled Nuclear Power Plants	Rev. 0	07/1977	General	Conforms
1.132	Site Investigations for Foundations of Nuclear Power Plants	Rev. 2	10/2003	General	Conforms
1.133	Loose-Part Detection Program for the Primary System of Light Water Cooled Reactors	Rev. 1	05/1981	General	Not applicable
1.134	Medical Evaluation of Licensed Personnel for Nuclear Power Plants	Rev. 3	03/1998	General	Conforms. Although RG 1.134 is not specifically identified in the FSAR, equivalent requirements for medical evaluations for licensed personnel are embedded in policies and procedures of operations and training departments.
1.135	Normal Water Level and Discharge at Nuclear Power Plants	Rev. 0	09/1977	General	Not applicable
1.136	Design Limits, Loading Combinations, Materials, Construction, and Testing of Concrete Containments	Rev. 3	03/2007	General	Conforms
1.137	Fuel-Oil Systems for Standby Diesel Generators	Rev. 1	10/1979	General	Not applicable
1.138	Laboratory Investigations of Soils and Rocks for Engineering Analysis and Design of Nuclear Power Plants	Rev. 2	12/2003	General	Conforms
1.139	Guidance for Residual Heat Removal	Rev. 0	05/1978	General	Conforms

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RG Number	Title	Revision	Date	RG Position	Evaluation
1.140	Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Normal Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants	Rev. 2	06/2001	General	Conforms. Operational program implementation is described in Section 13.4.
1.141	Containment Isolation Provisions for Fluid Systems	Rev. 0	04/1978	General	Conforms
1.142	Safety-Related Concrete Structures for Nuclear Power Plants (Other Than Reactor Vessels and Containments)	Rev. 2	11/2001	General	Conforms
1.143	Design Guidance for Radioactive Waste Management Systems, Structures, and Components Installed in Light-Water-Cooled Nuclear Power Plants	Rev. 2	11/2001	General	Conforms. Operational program implementation is described in Section 13.4.
1.145	Atmospheric Dispersion Models for Potential Accident Consequence Assessments at Nuclear Power Plants	Rev. 1	11/1982	General	Conforms
1.147	Inservice Inspection Code Case Acceptability, ASME Section XI, Division 1	Rev. 15	10/2007	General	Conforms. Operational program implementation is described in Section 13.4.
1.148	Functional Specification for Active Valve Assemblies in Systems Important to Safety in Nuclear Power Plants	Rev. 0	03/1981	General	Conforms
1.149	Nuclear Power Plant Simulation Facilities for Use in Operator Training and License Examinations	Rev. 3	10/2001	General	Conforms
1.150	Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice Examinations				Withdrawn
1.151	Instrument Sensing Lines	Rev. 0	07/1983	General	Conforms. Operational program implementation is described in Section 13.4.

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RG Number	Title	Revision	Date	RG Position	Evaluation
1.152	Criteria for Use of Computers in Safety Systems of Nuclear Power Plants	Rev. 2	01/2006	General	Conforms. Operational program implementation is described in Section 13.4.
1.153	Criteria for Safety Systems	Rev. 1	06/1996	General	Conforms
1.154	Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis Reports for Pressurized Water Reactors	Rev. 0	01/1987	General	Not applicable
1.155	Station Blackout	Rev. 0	08/1988	General	Conforms, except no emergency AC power is required for the ESBWR. Only the coping analysis is applicable. Operational program implementation is described in Section 13.4.
1.156	Environmental Qualification of Connection Assemblies for Nuclear Power Plants	Rev. 0	11/1987	General	Conforms
1.157	Best-Estimate Calculations of Emergency Core Cooling System Performance	Rev. 0	05/1989	General	Conforms
1.158	Qualification of Safety-Related Lead Storage Batteries for Nuclear Power Plants	Rev. 0	02/1989	General	Conforms
1.159	Assuring the Availability of Funds for Decommissioning Nuclear Reactors	Rev. 1	10/2003	General	Conforms. The amount of funds for decommissioning and the method of financial assurance are described in COLA Part 1.
1.160	Monitoring the Effectiveness of Maintenance at Nuclear Power Plants	Rev. 2	03/1997	General	Conforms. Operational program implementation is described in Section 13.4. Maintenance rule activities are discussed in Section 17.6.
1.161	Evaluation of Reactor Pressure Vessels with Charpy Upper-Shelf Energy Less Than 50 Ft-Lb.	Rev. 0	06/1995	General	Not applicable

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RG Number	Title	Revision	Date	RG Position	Evaluation
1.162	Format and Content of Report for Thermal Annealing of Reactor Pressure Vessels	Rev. 0	02/1996	General	This RG is outside the scope of the FSAR.
1.163	Performance-Based Containment Leak-Test Program	Rev. 0	09/1995	General	Conforms
1.165	Identification and Characterization of Seismic Sources and Determination of Safe Shutdown Earthquake Ground Motion	Rev. 0	03/1997	General	Conforms
1.166	Pre-Earthquake Planning and Immediate Nuclear Power Plant Operator Postearthquake Actions	Rev. 0	03/1997	General	Conforms. The seismic monitoring program, including the necessary test and operating procedures, will be implemented prior to receipt of fuel on site.
1.167	Restart of a Nuclear Power Plant Shut Down by a Seismic Event	Rev. 0	03/1997	General	Not applicable
1.168	Verification, Validation, Reviews, and Audits for Digital Computer Software Used in Safety Systems of Nuclear Power Plants	Rev. 1	02/2004	General	Conforms. Procedures addressed in Section 13.5. ITAAC addressed in COLA Part 10.
1.169	Configuration Management Plans for Digital Computer Software Used in Safety Systems of Nuclear Power Plants	Rev. 0	09/1997	General	Conforms. Procedures addressed in Section 13.5. ITAAC addressed in COLA Part 10.
1.170	Software Test Documentation for Digital Computer Software Used in Safety Systems of Nuclear Power Plants	Rev. 0	09/1997	General	Conforms. Procedures addressed in Section 13.5. ITAAC addressed in COLA Part 10.
1.171	Software Unit Testing for Digital Computer Software Used in Safety Systems of Nuclear Power Plants	Rev. 0	09/1997	General	Conforms. Procedures addressed in Section 13.5. ITAAC addressed in COLA Part 10.

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RG Number	Title	Revision	Date	RG Position	Evaluation
1.172	Software Requirements Specifications for Digital Computer Software Used in Safety Systems of Nuclear Power Plants	Rev. 0	09/1997	General	Conforms. Procedures addressed in Section 13.5. ITAAC addressed in COLA Part 10.
1.173	Developing Software Life Cycle Processes for Digital Computer Software Used in Safety Systems of Nuclear Power Plants	Rev. 0	09/1997	General	Conforms. Procedures addressed in Section 13.5. ITAAC addressed in COLA Part 10.
1.174	An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis	Rev. 1	11/2002	General	Not applicable. The approach described in this RG is not being used.
1.175	An Approach for Plant-Specific, Risk-Informed Decisionmaking: Inservice Testing	Rev. 0	08/1998	General	Not applicable. Risk-Informed Inservice Testing is not being used.
1.176	An Approach for Plant-Specific, Risk-Informed Decisionmaking: Graded Quality Assurance				Withdrawn
1.177	An Approach for Plant-Specific, Risk-Informed Decisionmaking: Technical Specifications	Rev. 0	08/1998	General	Not applicable. Risk-Informed Technical Specifications are not being used.
1.178	An Approach For Plant-Specific Risk-informed Decisionmaking for Inservice Inspection of Piping	Rev. 1	09/2003	General	Not applicable. Risk-Informed Inservice Inspection is not being used.
1.179	Standard Format and Content of License Termination Plans for Nuclear Power Reactors	Rev. 0	01/1999	General	This RG is outside the scope of the FSAR.
1.180	Guidelines for Evaluating Electromagnetic and Radio-Frequency Interference in Safety-Related Instrumentation and Control Systems	Rev. 1	10/2003	General	Conforms. Operational program implementation is described in Section 13.4.

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RG Number	Title	Revision	Date	RG Position	Evaluation
1.181	Content of the Updated Final Safety Analysis Report in Accordance with 10 CFR 50.71(e)	Rev. 0	09/1999	General	Conforms
1.182	Assessing and Managing Risk Before Maintenance Activities at Nuclear Power Plants	Rev. 0	05/2000	General	Conforms
1.183	Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors	Rev. 0	07/2000	General	Conforms
1.184	Decommissioning of Nuclear Power Reactors	Rev. 0	07/2000	General	Not applicable. The RG provides guidance on how to conduct decommissioning activities.
1.185	Standard Format and Content for Post-Shutdown Decommissioning Activities Report	Rev. 0	07/2000	General	Not applicable. This RG is outside the scope of the FSAR.
1.186	Guidance and Examples for Identifying 10 CFR 50.2 Design Bases	Rev. 0	10/2000	General	Not applicable. This RG is outside the scope of the FSAR.
1.187	Guidance for Implementation of 10 CFR 50.59, Changes, Tests, and Experiments	Rev. 0	11/2000	General	Conforms
1.188	Standard Format and Content for Applications to Renew Nuclear Power Plant Operating Licenses	Rev. 1	09/2005	General	Not applicable. This RG is outside the scope of the FSAR.
1.189	Fire Protection for Nuclear Power Plants	Rev. 1	03/2007	General	Conforms
1.190	Calculational and Dosimetry Methods for Determining Pressure Vessel Neutron Fluence	Rev. 0	03/2001	General	Conforms. The reactor vessel material surveillance program is described in Section 5.3.1.8. Implementation of the program is described in Section 13.4.
1.191	Fire Protection Program for Nuclear Power Plants During Decommissioning and Permanent Shutdown	Rev. 0	05/2001	General	Not applicable. This RG is outside the scope of the FSAR.

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RG Number	Title	Revision	Date	RG Position	Evaluation
1.192	Operation and Maintenance Code Case Acceptability, ASME OM Code	Rev. 0	06/2003	General	Conforms. Operational program implementation is described in Section 13.4.
1.193	ASME Code Cases Not Approved for Use	Rev. 2	10/2007	General	Conforms
1.194	Atmospheric Relative Concentrations for Control Room Radiological Habitability Assessments at Nuclear Power Plants	Rev. 0	06/2003	General	Conforms
1.195	Methods and Assumptions for Evaluating Radiological Consequences of Design Basis Accidents at Light-Water Nuclear Power Reactors	Rev. 0	05/2003	General	Not applicable. RG 1.183 is used.
1.196	Control Room Habitability at Light-Water Nuclear Power Reactors	Rev. 1	01/2007	General	Conforms
1.197	Demonstrating Control Room Envelope Integrity at Nuclear Power Plant Reactors	Rev. 0	05/2003	General	Conforms
1.198	Procedures and Criteria for Assessing Seismic Soil Liquefaction At Nuclear Power Plant Sites	Rev. 0	11/2003	General	Conforms
1.199	Anchoring Components and Structural Supports in Concrete	Rev. 0	11/2003	General	Conforms
1.200	An Approach for Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities	Rev. 1	01/2007	General	Not applicable
1.201	Guidelines for Categorizing Structures, Systems, and Components in Nuclear Power Plants According to Their Safety Significance	Rev. 1	05/2006	General	Not applicable

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RG Number	Title	Revision	Date	RG Position	Evaluation
1.202	Standard Format and Content of Decommissioning Cost Estimates for Nuclear Power Reactors	Rev. 0	02/2005	General	Not applicable. The RG provides guidance for submitting decommissioning cost estimates to NRC prior to license termination.
1.203	Transient and Accident Analysis Methods	Rev. 0	12/2005	General	Conforms
1.204	Guidelines for Lightning Protection of Nuclear Power Plants	Rev. 0	11/2005	General	Conforms. Operational program implementation is described in Section 13.4.
1.205	Risk-Informed, Performance-Based Fire Protection for Existing Light-Water Nuclear Power Plants	Rev. 0	05/2006	General	Not applicable. Risk-Informed, Performance-Based Fire Protection is not used.
1.206	Combined License Applications for Nuclear Power Plants (LWR Edition)	Rev. 0	06/2007	General	See Table 1.9-203
1.207	Guidelines for Evaluating Fatigue Analyses Incorporating the Life Reduction of Metal Components Due to the Effects of the Light-Water Reactor Environment for New Reactors	Rev. 0	03/2007	General	Conforms
1.208	A Performance-Based Approach to Define the Site-Specific Earthquake Ground Motion	Rev. 0	03/2007	All	Conforms
1.209	Guidelines for Environmental Qualification of Safety-Related Computer-Based Instrumentation and Control Systems in Nuclear Power Plants	Rev. 0	03/2007	General	Conforms. Operational program implementation is described in Section 13.4.

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RG Number	Title	Revision	Date	RG Position	Evaluation
4.7	General Site Suitability Criteria for Nuclear Power Stations	Rev. 2	04/1998	General	Conforms, except that the transient population between 10 and 50 miles cannot be quantified with any certainty. However, compared with the resident population within a 50-mile radius, the transient population is expected to be insignificant.
4.15	Quality Assurance for Radiological Monitoring Programs (Inception Through Normal Operations to License Termination) – Effluent Streams and the Environment	Rev. 2	07/2007	General	Conforms. Section 11.5.4.5 (NEI 07-09) provides a description of the ODCM. The implementation milestone is provided in Section 13.4.
5.44	Perimeter Intrusion Alarm Systems	Rev. 3	10/1997	General	Conforms. See COLA Part 8.
5.62	Reporting of Safeguards Events	Rev. 1	11/1987	General	Conforms, except that the transient population between 10 and 50 miles cannot be qualified with any certainty. However, compared with the resident population within a 50-mile radius, the transient population is expected to be insignificant.
5.66	Access Authorization Program for Nuclear Power Plants	Rev. 0	06/1991	General	Not applicable. NEI 03-01, Revision 1, April 2004 is used.
8.1	Radiation Symbol	Rev. 0	02/1973	General	Conforms. The facility utilizes standard radiation symbols.
8.2	Guide for Administrative Practices in Radiation Monitoring	Rev. 0	02/1973	General	Conforms. Operational program implementation is described in Section 13.4.
8.4	Direct-Reading and Indirect-Reading Pocket Dosimeters	Rev. 0	02/1973	General	Conforms. Operational program implementation is described in Section 13.4.
8.5	Criticality and Other Interior Evacuation Signals	Rev. 1	03/1981	General	Conforms. Operational program implementation is described in Section 13.4.

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Conformance with Regulatory Guides

RG Number	Title	Revision	Date	RG Position	Evaluation
8.6	Standard Test Procedure for Geiger-Muller Counters	Rev. 0	05/1973	General	Conforms. Operational program implementation is described in Section 13.4.
8.7	Instructions for Recording and Reporting Occupational Radiation Dose Data	Rev. 2	11/2005	General	Conforms. Operational program implementation is described in Section 13.4.
8.8	Information Relevant to Ensuring that Occupational Radiation Exposures at Nuclear Power Stations Will Be As Low As Is Reasonably Achievable	Rev. 3	06/1978	General	Conforms. Operational program implementation is described in Section 13.4.
8.9	Acceptable Concepts, Models, Equations, and Assumptions for a Bioassay Program	Rev. 1	07/1993	General	Conforms. Operational program implementation is described in Section 13.4.
8.10	Operating Philosophy for Maintaining Occupational Radiation Exposures As Low As Is Reasonably Achievable	Rev. 1-R	05/1977	General	Conforms. Operational program implementation is described in Section 13.4.
8.11	Applications of Bioassay for Uranium	Rev. 0	06/1974	General	Not applicable. RG 8.11 has been superseded by RG 8.9, Rev 1.
8.13	Instruction Concerning Prenatal Radiation Exposure	Rev. 3	06/1999	General	Conforms. Operational program implementation is described in Section 13.4.
8.15	Acceptable Programs for Respiratory Protection	Rev. 1	10/1999	General	Conforms. Operational program implementation is described in Section 13.4.
8.19	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants – Design Stage Man-Rem Estimates	Rev. 1	06/1979	General	Conforms
8.20	Applications of Bioassay for I-125 and I-131	Rev. 1	09/1979	General	Exception: Per NUREG-1736, RG 8.20 is outdated. RG 8.9 is used. Operational program implementation is described in Section 13.4.
8.25	Air Sampling in the Workplace	Rev. 1	06/1992	General	Not applicable

Table 1.9-202 (Sheet 22 of 22)
Conformance with Regulatory Guides

RG Number	Title	Revision	Date	RG Position	Evaluation
8.26	Applications of Bioassay for Fission and Activation Products	Rev. 0	09/1980	General	Exception: Per NUREG-1736, RG 8.26 is outdated. RG 8.9 is used. Operational program implementation is described in Section 13.4.
8.27	Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power Plants	Rev. 0	03/1981	General	Conforms. Operational program implementation is described in Section 13.4.
8.28	Audible-Alarm Dosimeters	Rev. 0	08/1981	General	Conforms. Operational program implementation is described in Section 13.4.
8.29	Instruction Concerning Risks from Occupational Radiation Exposure	Rev. 1	02/1996	General	Conforms. Operational program implementation is described in Section 13.4.
8.32	Criteria for Establishing a Tritium Bioassay Program	Rev. 0	07/1988	General	Exception: Per NUREG-1736, RG 8.32 is outdated. RG 8.9 is used. Operational program implementation is described in Section 13.4.
8.33	Quality Management Program	Rev. 0	10/1991	General	Not applicable to nuclear power plants. RG 8.33 applies to nuclear medicine.
8.34	Monitoring Criteria and Methods To Calculate Occupational Radiation Doses	Rev. 0	07/1992	General	Conforms. Operational program implementation is described in Section 13.4.
8.35	Planned Special Exposures	Rev. 0	06/1992	General	Conforms. Operational program implementation is described in Section 13.4.
8.36	Radiation Dose to the Embryo/Fetus	Rev. 0	07/1992	General	Conforms. Operational program implementation is described in Section 13.4.
8.38	Control of Access to High and Very High Radiation Areas of Nuclear Plants	Rev. 1	05/2006	General	Conforms. Operational program implementation is described in Section 13.4.

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Conformance With the FSAR Content Guidance In RG 1.206

Section	Section Title	Conformance Evaluation
C.I.1	Introduction and General Description of the Plant	Conforms
C.I.1.1	Introduction	Conforms
C.I.1.2	General Plant Description	Conforms. Addressed in Sections 1.2 and 2.0, Figure 1.1-201, and DCD Figures 1.2-1 through 1.2-33.
C.I.1.3	Comparisons with Other Facilities	Conforms
C.I.1.4	Identification of Agents and Contractors	Conforms
C.I.1.5	Requirements for Further Technical Information	Conforms
C.I.1.6	Material Referenced	Conforms
C.I.1.7	Drawings and Other Detailed Information	Conforms
C.I.1.8	Site and Plant Design Interfaces and Conceptual Design Information	Conforms.See COLA Part 7 for the departure report.
C.I.1.9	Conformance with Regulatory Criteria	Conforms
C.I.2	Site Characteristics	Conforms
C.I.2.1.1	Site Location and Description	Conforms
C.I.2.1.2	Exclusion Area Authority and Control	Conforms
C.I.2.1.3	Population Distribution	Conforms, except that the transient population between 10 and 50 miles cannot be quantified with any certainty. However, compared with the resident population within a 50-mile radius, the transient population is expected to be insignificant.
C.I.2.2	Nearby Industrial, Transportation, and Military Facilities	Conforms
C.I.2.2.1	Locations and Routes	Conforms
C.I.2.2.2	Descriptions	Conforms
C.I.2.2.3	Evaluation of Potential Accidents	Conforms
C.I.2.3.1	Regional Climatology	Conforms
C.I.2.3.2	Local Meteorology	Conforms
C.I.2.3.3	Onsite Meteorological Measurements Program	Conforms
C.I.2.3.4	Short-Term Atmospheric Dispersion Estimates for Accident Releases	Conforms

Table 1.9-203 (Sheet 2 of 35)
Conformance With the FSAR Content Guidance In RG 1.206

Section	Section Title	Conformance Evaluation
C.I.2.3.5	Long-Term Atmospheric Dispersion Estimates for Routine Releases	Conforms
C.I.2.4.1	Hydrologic Description	Conforms
C.I.2.4.2	Floods	Conforms
C.I.2.4.3	Probable Maximum Flood on Streams and Rivers	Conforms
C.I.2.4.4	Potential Dam Failures	Conforms
C.I.2.4.5	Probable Maximum Surge and Seiche Flooding	Conforms
C.I.2.4.6	Probable Maximum Tsunami	Conforms
C.I.2.4.7	Ice Effects	Conforms. Addressed in DCD Appendix 3G.
C.I.2.4.8	Cooling Water Canals and Reservoirs	Conforms
C.I.2.4.9	Channel Diversions	Conforms
C.I.2.4.10	Flooding Protection Requirements	Conforms
C.I.2.4.11	Low Water Considerations	Conforms
C.I.2.4.12	Ground Water	Conforms
C.I.2.4.13	Accidental Release of Radioactive Liquid Effluent in Ground and Surface Waters	Conforms
C.I.2.4.14	Technical Specifications and Emergency Operation Requirements	Conforms
C.I.2.5.1	Basic Geologic and Seismic Information	Conforms
C.I.2.5.2	Vibratory Ground Motion	Conforms
C.I.2.5.3	Surface Faulting	Conforms
C.I.2.5.4	Stability of Subsurface Materials and Foundations	Conforms
C.I.2.5.5	Stability of slopes	Conforms
C.I.3.1	Conformance with NRC General Design Criteria	Conforms. Conformance with the NRC's criteria to 10 CFR 50, Appendix A, is described in DCD Section 3.1 and the applicable DCD system sections.
C.I.3.2.1	Seismic Classification	Conforms. There are no additional safety-related or RTNSS SSCs subject to seismic classification beyond those addressed in the DCD. There are no SSCs outside the referenced certified design that are required to be designed for an OBE.

Table 1.9-203 (Sheet 3 of 35)
Conformance With the FSAR Content Guidance In RG 1.206

Section	Section Title	Conformance Evaluation
C.I.3.2.2	System Quality Group Classification	Conforms. There are no additional safety-related or RTNSS SSCs subject to system quality group classification beyond those addressed in the DCD.
C.I.3.3.1	Wind Loadings	Conforms. There are no safety-related SSCs outside the scope of the certified design. Nonsafety-related facility SSCs that are not included in the referenced certified design meet the requirements of DCD Sections 3.3.1.3 and 3.3.2.3.
C.I.3.3.2	Tornado Loadings	Conforms
C.I.3.4.1	Internal Flood Protection	Conforms. There are no SSCs outside the scope of the referenced certified design that require internal flood protection whose failure could prevent a safe shutdown of the plant or result in the uncontrolled release of significant radioactivity.
C.I.3.4.2	Analysis Procedures	Conforms. There are no Seismic Category I structures outside the scope of the referenced certified design.
C.I.3.5.1.1	Internally Generated Missiles (Outside Containment)	Conforms. There are no SSCs outside the scope of the referenced certified design that are required to be protected against damage from internally generated missiles.
C.I.3.5.1.2	Internally Generated Missiles (Inside Containment)	Conforms
C.I.3.5.1.3	Turbine Missiles	Conforms. Addressed in DCD Section 10.2.3.8.
C.I.3.5.1.4	Missiles Generated by Tornadoes and Extreme Winds	Conforms. Table 2.0-201 demonstrates that the site-specific tornado characteristics are bounded by the parameters assumed in the DCD. DCD Section 3.5.1.4 indicates that resistance to missiles is independent of site topography.
C.I.3.5.1.5	Site Proximity Missiles (Except Aircraft)	Conforms
C.I.3.5.1.6	Aircraft Hazards	Conforms
C.I.3.5.2	Structures, Systems, and Components To Be Protected from Externally Generated Missiles	Conforms. There are no SSCs outside the scope of the referenced certified design that are required to be protected from externally generated missiles.
C.I.3.5.3	Barrier Design Procedures	Conforms. There are no SSCs that require reanalysis for tornado, extreme wind, or site proximity missile impact or for aircraft impact.
C.I.3.6	Protection against Dynamic Effects Associated with the Postulated Rupture of Piping	Conforms

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Section	Section Title	Conformance Evaluation
C.I.3.6.1	Plant Design for Protection against Postulated Piping Failures in Fluid Systems Outside of Containment	Conforms
C.I.3.6.2	Determination of Rupture Locations and Dynamic Effects Associated with the Postulated Rupture of Piping	Conforms
C.I.3.6.3	Leak-Before-Break Evaluation Procedures	Not Applicable. ESBWR design does not rely on a Leak-Before-Break Evaluation.
C.I.3.7.1	Seismic Design Parameters	Conforms. Addressed in DCD Sections 3.7 and 3.7.1.
C.I.3.7.1.1	Design Ground Motion	Conforms
C.I.3.7.1.2	Percentage of Critical Damping Values	Conforms
C.I.3.7.1.3	Supporting Media for Seismic Category I Structures	Conforms
C.I.3.7.2	Seismic System Analysis	Conforms. Addressed in DCD Section 3.7.2.
C.I.3.7.2.1	Seismic Analysis Methods	Conforms
C.I.3.7.2.2	Natural Frequencies and Responses	Conforms. Addressed in DCD Section 3.7.2.2.
C.I.3.7.2.3	Procedures Used for Analytical Modeling	Conforms
C.I.3.7.2.4	Soil/Structure Interaction	Conforms
C.I.3.7.2.5	Development of Floor Response Spectra	Conforms. Addressed in DCD Section 3.7.2.5.
C.I.3.7.2.6	Three Components of Earthquake Motion	Conforms
C.I.3.7.2.7	Combination of Modal Responses	Conforms
C.I.3.7.2.8	Interaction of Nonseismic Category I Structures with Seismic Category I Structures	Conforms. There are no Seismic Category I structures outside the scope of the referenced certified design. In lieu of providing the plant-specific distances between structures and the heights of structures, the distance and height requirements for Non-Seismic Category I structures are addressed in DCD Section 3.7.2.8.
C.I.3.7.2.9	Effects of Parameter Variations on Floor Response Spectra	Conforms. Addressed in DCD Section 3.7.2.9.
C.I.3.7.2.10	Use of Constant Vertical Static Factors	Conforms
C.I.3.7.2.11	Method Used to Account for Torsional Effects	Conforms

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Section	Section Title	Conformance Evaluation
C.I.3.7.2.12	Comparison of Responses	Conforms. Addressed in DCD Section 3.7.2.12.
C.I.3.7.2.13	Methods for Seismic Analysis of Dams	Not applicable. There are no Seismic Category I dams in the ESBWR design per DCD Section 3.7.3.14.
C.I.3.7.2.14	Determination of Dynamic Stability of Seismic Category I Structures	Conforms. Addressed in DCD Sections 3.7.2.14 and 3.8.5.5.
C.I.3.7.2.15	Analysis Procedure for Damping	Conforms
C.I.3.7.3.1	Seismic Analysis Methods	Conforms
C.I.3.7.3.2	Procedures Used for Analytical Modeling	Conforms
C.I.3.7.3.3	Analysis Procedure for Damping	Conforms
C.I.3.7.3.4	Three Components of Earthquake Motion	Conforms
C.I.3.7.3.5	Combination of Modal Responses	Conforms. Addressed in DCD Section 3.7.3.7.
C.I.3.7.3.6	Use of Constant Vertical Static Factors	Conforms
C.I.3.7.3.7	Buried Seismic Category I Piping, Conduits, and Tunnels	Conforms. Addressed in DCD Section 3.7.3.13.
C.I.3.7.3.8	Methods for Seismic Analysis of Seismic Category I Concrete Dams	Not applicable. There are no Seismic Category I dams for VCS.
C.I.3.7.3.9	Methods for Seismic Analysis of Aboveground Tanks	Conforms. Addressed in DCD Section 3.7.3.15.
C.I.3.7.4	Seismic Instrumentation	Conforms
C.I.3.8.1	Concrete Containment	Conforms
C.I.3.8.2	Steel Containment	Conforms
C.I.3.8.3	Concrete and Steel Internal Structures of Steel or Concrete Containments	Conforms
C.I.3.8.4	Other Seismic Category I Structures	Conforms. There are no Seismic Category I Structures that are outside the scope of the DCD.
C.I.3.8.5	Foundations	Conforms
C.I.3.9.1	Special Topics for Mechanical Components	Conforms. There are no Seismic Category I components or supports beyond those evaluated in the reference certified design.
C.I.3.9.1.1	Design Transients	Conforms. There are no Seismic Category I components or supports beyond those evaluated in the reference certified design.
C.I.3.9.1.2	Computer Programs Used in Analysis	Conforms. There are no Seismic Category I components or supports beyond those evaluated in the reference certified design.

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Section	Section Title	Conformance Evaluation
C.I.3.9.1.3	Experimental Stress Analysis	Conforms. There are no Seismic Category I components or supports beyond those evaluated in the reference certified design.
C.I.3.9.1.4	Considerations for the Evaluation of the Faulted Condition	Conforms. There are no Seismic Category I components or supports beyond those evaluated in the reference certified design.
C.I.3.9.2	Dynamic Testing and Analysis of Systems, Components, and Equipment	Conforms. There are no systems outside the scope of the referenced certified design that require dynamic testing and analysis.
C.I.3.9.2.1	Piping Vibration, Thermal Expansion, and Dynamic Effects	Conforms. There are no ASME Code Class 1, 2, and 3 systems; other high-energy piping systems inside Seismic Category I structures; high-energy portions of systems for which failure could reduce the functioning of any Seismic Category I plant feature to an unacceptable level; or Seismic Category I portions of moderate-energy piping systems located outside containment outside the scope of the referenced certified design.
C.I.3.9.2.2	Seismic Analysis and Qualification of Seismic Category I Mechanical Equipment	Conforms
C.I.3.9.2.3	Dynamic Response Analysis of Reactor Internals under Operational Flow Transients and Steady-State Conditions	Conforms. There are no ESBWR pressure vessel internals that the referenced certified design does not cover.
C.I.3.9.2.4	Preoperational Flow-Induced Vibration Testing of Reactor Internals	Conforms. There are no BWR pressure vessel internals that the referenced certified design does not cover. DCD Sections 3.9.2.3 and 3.9.2.4 adequately cover the analysis of potential adverse flow effects that could impact BWR vessel internals.
C.I.3.9.2.5	Dynamic System Analysis of the Reactor Internals under Faulted Condition	Conforms. Addressed in DCD Section 3.9.3.1 and DCD Table 3.9-2.
C.I.3.9.2.6	Correlations of Reactor Internals Vibration Tests with the Analytical Results	Conforms. Addressed in DCD Section 3.9.2.6.
C.I.3.9.3	ASME Code Class 1, 2, and 3 Components and Component Supports and Core Support Structures	Conforms. There are no pressure-retaining components or component supports designed or constructed in accordance with ASME Code Class 1, 2, or 3, or GDC 1, 2, 4, 14, or 15, beyond those evaluated in the referenced certified design.
C.I.3.9.4	Control Rod Drive Systems	Conforms
C.I.3.9.5.1	Design Arrangements	Conforms

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Section	Section Title	Conformance Evaluation
C.I.3.9.5.2	Loading Conditions	Conforms
C.I.3.9.5.3	Design Bases	Conforms
C.I.3.9.5.4	BWR Reactor Pressure Vessel Internals Including Steam Dryer	Conforms. There are no reactor pressure vessel internals (including the steam dryer) or other main steam system components that are not covered by the referenced certified design. The reactor is classified as non-prototype.
C.I.3.9.6.1	Functional Design and Qualification of Pumps, Valves, and Dynamic Restraints	Conforms. There is no safety-related equipment beyond the scope of the referenced certified design.
C.I.3.9.6.2	Inservice Testing Program for Pumps	Not applicable. There are no safety-related pumps.
C.I.3.9.6.3	Inservice Testing Program for Valves	Conforms. Addressed in DCD Section 3.9.6; the list of valves included in the IST program is provided in DCD Table 3.9-8. IST Program test procedures and schedules are addressed in Technical Specifications 5.5.5. Justification for cold shutdown and refueling outage test schedules is addressed in DCD Section 3.9.6 and DCD Table 3.9-8. The implementation milestones for the IST and MOV Programs are addressed in Section 13.4.
C.I.3.9.6.3.1	Inservice Testing Program for Motor-Operated Valves	Conforms. Addressed in DCD Section 3.9.6.
C.I.3.9.6.3.2	Inservice Testing Program for Power-Operated Valves Other Than Motor-Operated Valves	Conforms. Addressed in DCD Section 3.9.6.
C.I.3.9.6.3.3	Inservice Testing Program for Check Valves	Conforms. Addressed in DCD Section 3.9.6.
C.I.3.9.6.3.4	Pressure Isolation Valve Leak Testing	Not applicable. The ESBWR plant does not have any Pressure Isolation Valves.
C.I.3.9.6.3.5	Containment Isolation Valve Leak Testing	Conforms
C.I.3.9.6.3.6	Inservice Testing Program for Safety and Relief Valves	Conforms. Addressed in DCD Table 3.9-8.
C.I.3.9.6.3.7	Inservice Testing Program for Manually Operated Valves	Conforms. Addressed in DCD Table 3.9-8.
C.I.3.9.6.3.8	Inservice Testing Program for Explosively Activated Valves	Conforms. Addressed in DCD Table 3.9-8.
C.I.3.9.6.4	Inservice Testing Program for Dynamic Restraints	Conforms with the following exception: A plant specific snubber table will be prepared in conjunction with closure of ITAAC Table 3.1-1.

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Section	Section Title	Conformance Evaluation
C.I.3.9.6.5	Relief Requests and Alternative Authorizations to ASME OM Code	Conforms
C.I.3.10.1	Seismic Qualification Criteria	Conforms. There is no seismic or dynamic qualification required for equipment that is outside the scope of the referenced certified design.
C.I.3.10.2	Methods and Procedures for Qualifying Mechanical and Electrical Equipment and Instrumentation	Conforms
C.I.3.10.3	Methods and Procedures of Analysis or Testing of Supports of Mechanical and Electrical Equipment and Instrumentation	Conforms
C.I.3.10.4	Test and Analyses Results and Experience Database	Conforms
C.I.3.11	Environmental Qualification of Mechanical and Electrical Equipment	Conforms. There is no other equipment beyond that which has been evaluated in the referenced certified design.
C.I.3.11.1	Equipment Location and Environmental Conditions	Conforms
C.I.3.11.2	Qualification Tests and Analysis	Conforms
C.I.3.11.3	Qualification Test Results	Conforms
C.I.3.11.4	Loss of Ventilation	Conforms
C.I.3.11.5	Estimated Chemical and Radiation Environment	Conforms
C.I.3.11.6	Qualification of Mechanical Equipment	Conforms
C.I.3.12.1	Introduction	Conforms
C.I.3.12.2	Codes and Standards	Conforms. Addressed in DCD Sections 3.2, 3.6, 3.7, and in Chapters 5 and 14.
C.I.3.12.3	Piping Analysis Methods	Conforms. Addressed in DCD Sections 3.7.2.2 and 3.7.3.9.
C.I.3.12.3.1	Experimental Stress Analyses	Conforms. Addressed in DCD Section 3.9.1.3.
C.I.3.12.3.2	Modal Response Spectrum Method	Conforms. Addressed in DCD Section 3.7.2.1.
C.I.3.12.3.3	Response Spectra Method (or Independent Support Motion Method)	Conforms. Addressed in DCD Section 3.7.2.1.2.
C.I.3.12.3.4	Time History Method	Conforms. Addressed in DCD Section 3.7.2.1.1.

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Section	Section Title	Conformance Evaluation
C.I.3.12.3.5	Inelastic Analyses Method	Not Applicable. Per DCD Section 3.9.1.4 (Inelastic Analyses Methods), except for pipe whip restraints, inelastic analyses methods are not used in the ESBWR piping design and analysis.
C.I.3.12.3.6	Small-Bore Piping Method	Conforms. Addressed in DCD Section 3.7.3.16.
C.I.3.12.3.7	Nonseismic/Seismic Interaction (II/I)	Conforms with the following exception: The location and distance between piping systems will be established as part of the completion of ITAAC Table 3.1-1.
C.I.3.12.3.8	Seismic Category I Buried Piping	Not Applicable. Per DCD Section 3.7.3.13, there is no buried Seismic Category I piping.
C.I.3.12.4	Piping Modeling Technique	Conforms. Addressed in DCD Section 3.7.3.3.1 and Appendix 3D for the PISYS computer code.
C.I.3.12.4.1	Computer Codes	Conforms. Addressed in DCD Appendix 3D.
C.I.3.12.4.2	Dynamic Piping Model	Conforms. Addressed in DCD Section 3.7.3.3.1.
C.I.3.12.4.3	Piping Benchmark Program	Conforms. Addressed in DCD Appendix 3D.
C.I.3.12.4.4	Decoupling Criteria	Conforms. Addressed in DCD Sections 3.7.2.3 and 3.7.3.16.
C.I.3.12.5.1	Seismic Input Envelope vs. Site-Specific Spectra	Conforms. Addressed in DCD Section 3.7.1.
C.I.3.12.5.2	Design Transients	Conforms. Addressed in DCD Section 3.9.1.1 and DCD Table 3.9-1.
C.I.3.12.5.3	Loadings and Load Combination	Conforms. Addressed in DCD Section 3.9.1.1 and DCD Table 3.9-8.
C.I.3.12.5.4	Damping Values	Conforms. Addressed in DCD Section 3.7.1.2 and DCD Table 3.7-1.
C.I.3.12.5.5	Combination of Modal Responses	Conforms. Addressed in DCD Section 3.7.3.7.
C.I.3.12.5.6	High-Frequency Modes	Conforms. Addressed in DCD Sections 3.7.1.1 and 3.7.1.2.
C.I.3.12.5.7	Fatigue Evaluation of ASME Code Class 1 Piping	Conforms. Addressed in DCD Section 3.9.3.4 and DCD Table 3.9-8.
C.I.3.12.5.8	Fatigue Evaluation of ASME Code Class 2 and 3 Piping	Conforms. Addressed in DCD Section 3.9.
C.I.3.12.5.9	Thermal Oscillations in Piping Connected to the Reactor Coolant System	Conforms
C.I.3.12.5.10	Thermal Stratification	Conforms. Addressed in DCD Section 3.9.2.1.2.
C.I.3.12.5.11	Safety Relief Valve Design, Installation, and Testing	Conforms. Addressed in DCD Figures 5.2-3 and 5.4-3, and DCD Table 3.9-8.

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Section	Section Title	Conformance Evaluation
C.I.3.12.5.12	Functional Capability	Conforms. Addressed in DCD Table 3.9-2, Note 13, and DCD Chapters 5 and 6.
C.I.3.12.5.13	Combination of Inertial and Seismic Anchor Motion Effects	Conforms. Addressed in DCD Section 3.7.3.9.
C.I.3.12.5.14	Operating-Basis Earthquake as a Design Load	Not applicable. The SSE establishes the design load for the ESBWR.
C.I.3.12.5.15	Welded Attachments	Conforms. Addressed in DCD Section 3.9.3.7.1.
C.I.3.12.5.16	Modal Damping for Composite Structures	Conforms. Addressed in DCD Section 3.7.2.13.
C.I.3.12.5.17	Minimum Temperature for Thermal Analyses	Conforms. Addressed in DCD Sections 3.9.1.1 and 3.9.3.1.
C.I.3.12.5.18	Intersystem Loss-of-Coolant Accident	Conforms. Addressed in DCD Appendix 3K.
C.I.3.12.5.19	Effects of Environment on Fatigue Design	Conforms. Addressed in DCD Section 3.9.3.4. The reference in RG 1.206 to RG 1.76 appears to be in error, and should have referenced RG 1.207.
C.I.3.12.6.1	Applicable Codes	Conforms. Addressed in DCD Section 3.9.3.7.1.
C.I.3.12.6.2	Jurisdictional Boundaries	Conforms. Addressed in DCD Section 3.9.3.7.1.
C.I.3.12.6.3	Loads and Load Combinations	Conforms. Addressed in DCD Section 3.9 and DCD Appendix 3B.
C.I.3.12.6.4	Pipe Support Baseplate and Anchor Bolt Design	Conforms. Addressed in DCD Section 3.9.3.7.
C.I.3.12.6.5	Use of Energy Absorbers and Limit Stops	Conforms. Addressed in DCD Section 3.9.3.7.
C.I.3.12.6.6	Use of Snubbers	Conforms. Addressed in DCD Section 3.9.3.7.1(3).
C.I.3.12.6.7	Pipe Support Stiffnesses	Conforms. Addressed in DCD Section 3.9.3.7.1.
C.I.3.12.6.8	Seismic Self-Weight Excitation	Conforms. Addressed in DCD Section 3.9.3.7.1.
C.I.3.12.6.9	Design of Supplementary Steel	Conforms. Addressed in DCD Section 3.9.3.7.1.
C.I.3.12.6.10	Consideration of Friction Forces	Conforms. Addressed in DCD Section 3.9.3.7.1(5).
C.I.3.12.6.11	Pipe Support Gaps and Clearances	Conforms. Addressed in DCD Section 3.9.3.7.1.
C.I.3.12.6.12	Instrumentation Line Support Criteria	Conforms. Addressed in DCD Section 3.9.3.7.1.
C.I.3.12.6.13	Pipe Deflection Limits	Conforms. Addressed in DCD Section 3.9.2.1.1 and Chapter 14.
C.I.3.13	Threaded Fasteners — ASME code Class 1, 2, and 3	Conforms
C.I.3.13.1.1	Materials Selection	Conforms
C.I.3.13.1.2	Special Materials Fabrication Processes and Special Controls	Conforms

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Section	Section Title	Conformance Evaluation
C.I.3.13.1.3	Fracture Toughness Requirements for Threaded Fasteners Made of Ferritic Materials	Conforms
C.I.3.13.1.5	Certified Material Test Reports	Conforms
C.I.3.13.2	Inservice Inspection Requirements	Conforms
C.I.4.1	Summary Description	Conforms
C.I.4.2	Fuel System Design	Conforms
C.I.4.3	Nuclear Design	Conforms
C.I.4.4	Thermal and Hydraulic Design	Conforms
C.I.4.5.1	Control Rod Drive Structural Materials	Conforms
C.I.4.5.2	Reactor Internal and Core Support Materials	Conforms
C.I.4.6	Functional Design of Control Rod Drive System	Conforms
C.I.5.1	Summary Description	Conforms
C.I.5.2.1	Compliance with ASME Codes and Code Cases	Conforms
C.I.5.2.2.1	Design Bases	Conforms
C.I.5.2.2.2	Design Evaluation	Conforms
C.I.5.2.2.3	Piping and Instrumentation Diagrams	Conforms
C.I.5.2.2.4	Equipment and Component Description	Conforms
C.I.5.2.2.5	Mounting of Pressure-Relief Devices	Conforms
C.I.5.2.2.6	Applicable Codes and Classification	Conforms
C.I.5.2.2.7	Material Specification	Conforms
C.I.5.2.2.8	Process Instrumentation	Conforms
C.I. 5.2.2.9	System Reliability	Conforms
C.I.5.2.2.10	Testing and Inspection	Conforms. Addressed in DCD Section 5.2.2.4, and in Section 3.9 and Chapter 14.
C.I.5.2.3.1	Material Specifications	Conforms
C.I.5.2.3.2	Compatibility with Reactor Coolant	Conforms. Addressed in DCD Section 5.2.3.
C.I.5.2.3.3	Fabrication and Processing of Ferritic Materials	Conforms
C.I.5.2.3.4	Fabrication and Processing of Austenitic Stainless Steels	Conforms

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Section	Section Title	Conformance Evaluation
C.I.5.2.3.5	Prevention of Primary Water Stress-Corrosion Cracking for Nickel-Based Alloys (PWRs only)	Not applicable. Applies only to PWRs.
C.I.5.2.3.6	Threaded Fasteners	Conforms. Addressed in DCD Section 3.9.3.9.
C.I.5.2.4.1	Inservice Inspection and Testing Program	Conforms. Addressed in DCD Section 5.2.4 and in Section 5.2.4.
C.I.5.2.4.2	Preservice Inspection and Testing Program	Conforms. Addressed in DCD Section 5.2.4.
C.I.5.2.5	Reactor Coolant Pressure Boundary Leakage Detection	Conforms
C.I.5.3.1.1	Material Specifications	Conforms
C.I.5.3.1.2	Special Processes Used for Manufacturing and Fabrication	Conforms
C.I.5.3.1.3	Special Methods for Nondestructive Examination	Conforms
C.I.5.3.1.4	Special Controls for Ferritic and Austenitic Stainless Steels	Conforms
C.I.5.3.1.5	Fracture Toughness	Conforms
C.I.5.3.1.6	Material Surveillance	Conforms. Addressed in DCD Section 5.3.1.6 and Section 5.3.1.8.
C.I.5.3.1.7	Reactor Vessel Fasteners	RG 1.206 does not contain any guidance in this section.
C.I.5.3.2.1	Limit Curves	Conforms
C.I.5.3.2.2	Operating Procedures	Conforms. Addressed in DCD Sections 5.3.2.1, 5.3.2.2, and 5.3.3.6, and in Section 5.3.3.6.
C.I.5.3.2.3	Pressurized Thermal Shock (PWRs only)	Not applicable. Applies only to PWRs.
C.I.5.3.2.4	Upper-Shelf Energy	Conforms
C.I.5.3.3	Reactor Vessel Integrity	Conforms. Identification of a specific manufacturer is not required.
C.I.5.3.3.1	Design	Conforms
C.I.5.3.3.2	Materials of Construction	Conforms
C.I.5.3.3.3	Fabrication Methods	Conforms
C.I.5.3.3.4	Inspection Requirements	Conforms. Addressed in DCD Section 5.3.3.4.
C.I.5.3.3.5	Shipment and Installation	Conforms. Addressed in DCD Section 5.3.3.5.
C.I.5.3.3.6	Operating Conditions	Conforms. Addressed in DCD Section 5.3.3.6.
C.I.5.3.3.7	Inservice Surveillance	Conforms. Addressed in DCD Section 5.3.3.7.
C.I.5.3.3.8	Threaded Fasteners	Conforms. Addressed in DCD Section 3.9.3.9.

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Section	Section Title	Conformance Evaluation
C.I.5.4.1	Reactor Coolant Pumps or Circulation Pumps (BWR)	Conforms
C.I.5.4.1.1	Pump Flywheel Integrity (PWR)	Not applicable. Applies only to PWRs.
C.I.5.4.2	Steam Generators (PWR)	Not applicable. Applies only to PWRs.
C.I.5.4.3	Reactor Coolant System Piping and Valves	Conforms
C.I.5.4.4	Main Steamline Flow Restrictions	Conforms
C.I.5.4.5	Pressurizer	Not applicable. Applies only to PWRs.
C.I.5.4.6	Reactor Core Isolation Cooling System (BWRs)/Isolation Condenser System (Economic Simplified BWR)	Conforms
C.I.5.4.7	Residual Heat Removal System/Passive Residual Heat Removal System (Advanced Light-Water Reactor/Shutdown Cooling Mode of the Reactor Water Cleanup System (Economic Simplified BWR)	Conforms
C.I.5.4.8	Reactor Water Cleanup System (BWR)/Reactor Water Cleanup/Shutdown Cooling System (Economic Simplified BWR)	Conforms
C.I.5.4.9	Reactor Coolant System Pressure Relief Devices/Reactor Coolant Depressurization Systems	Conforms
C.I.5.4.10	Reactor Coolant System Component Supports	Conforms
C.I.5.4.11	Pressurizer Relief Discharge System (PWRs only)	Not applicable. Applies only to PWRs.
C.I.5.4.12	Reactor Coolant System High-Point Vents	Conforms
C.I.5.4.13	Main Steamline, Feedwater, and Auxiliary Feedwater Piping	Conforms
C.I.6.1	Engineered Safety Feature Materials	Conforms. Addressed in DCD Section 6.1.
C.I.6.1.1.1	Materials Selection and Fabrication	Conforms
C.I.6.1.1.2	Composition and Compatibility of Core Cooling Coolants and Containment Sprays	Conforms. Addressed in DCD Sections 5.2.3.2, 5.2.3.4.1, 5.4.8, 6.1.1.3.4, 6.1.1.4, 6.1.2, 9.1.3, and 9.3.10.

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Section	Section Title	Conformance Evaluation
C.I.6.1.2	Organic Materials	Exception. The information requested by the RG is not available at this time, but commitments and a milestone for completing COL Item 6.1.3-1-A, which pertains to this guidance, are addressed in Section 6.1.2.3.
C.I.6.2	Containment Systems	Conforms
C.I.6.2.1	Containment Functional Design	Conforms
C.I.6.2.2	Containment Heat Removal Systems	Conforms
C.I.6.2.3	Secondary Containment Functional Design	Not Applicable. The ESBWR plant does not have a secondary containment.
C.I.6.2.4	Containment Isolation System	Conforms
C.I.6.2.5	Combustible Gas Control in Containment	Conforms
C.I.6.2.6	Containment Leakage Testing	Conforms. Addressed in DCD Sections 6.2.6.1 through 6.2.6.5, and in Section 13.4. Special testing requirements in RG 1.206, Section C.I.6.2.6.5 are not applicable to the ESBWR.
C.I.6.2.7	Fracture Prevention of Containment Pressure Vessel	Conforms
C.I.6.3	Emergency Core Cooling System	Conforms. There are no aspects of the site-specific design that affect the LOCA analyses in the DCD.
C.I.6.4	Habitability Systems	Conforms
C.I.6.5	Fission Product Removal and Control Systems	Conforms
C.I.6.6	Inservice Inspection of Class 2 and 3 Components	Conforms. Addressed in DCD Section 6.6 and in Section 6.6.10.3.
C.I.6.6.1	Components Subject to Examination	Conforms
C.I.6.6.2	Accessibility	Conforms
C.I.6.6.3	Examination Techniques and Procedures	Conforms. Addressed in DCD Section 6.6.3.2. There are no special examination techniques required to meet the ASME Code.
C.I.6.6.4	Inspection Intervals	Conforms. Addressed in DCD Section 6.6.4.
C.I.6.6.5	Examination Categories and Requirements	Conforms. Addressed in DCD Section 6.6.3.1.

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Section	Section Title	Conformance Evaluation
C.I.6.6.6	Evaluation of Examination Results	Conforms (addressed in DCD Section 6.6.5), except that RG 1.206 references ASME Code Sections IWC-4000 and IWD-4000 for Class 2 and Class 3, respectively, whereas DCD Section 6.6.5 references IWA-4000. Later editions of ASME Code Section XI do not contain Sections IWC-4000 and IWD-4000, only IWA-4000. Therefore, the intent of the RG is met.
C.I.6.6.7	System Pressure Tests	Conforms. Addressed in DCD Section 6.6.6.
C.I.6.6.8	Augmented Inservice Inspection to Protect against Postulated Piping Failures	Conforms. Addressed in DCD Section 6.6.7.
C.I.6.7	Main Steamline Isolation Valve Leakage Control Steam (BWRs)	Not applicable to the ESBWR.
C.I.7	Instrumentation and Controls	Conforms. Addressed in DCD Chapter 7, Tier 1, and design-related ITAAC (DAC). There are no departures from the referenced certified design.
C.I.7.1	Introduction	Conforms. There is no safety-related instrumentation, control, or supporting system that has not been addressed in the referenced certified design or other parts of the COL application.
C.I.7.2	Reactor Trip System	Conforms. There is no reactor trip system instrumentation, control, or supporting system that has not been addressed in the referenced certified design or other parts of the COL application.
C.I.7.3	Engineered Safety Feature Systems	Conforms. There are no ESF systems I&C or supporting systems that have not been addressed in the referenced certified design or other parts of the COL application.
C.I.7.4	Systems Required for Safe Shutdown	Conforms. There are no safe-shutdown systems I&C or supporting systems that have not been addressed in the referenced certified design or other parts of the COL application.
C.I.7.5	Information Systems Important to Safety	Conforms. There are no information systems important to safety that have not been addressed in the referenced certified design or other parts of the COL application.
C.I.7.6	Interlock Systems Important to Safety	Conforms. There are no interlock systems important to safety that have not been addressed in the referenced certified design or other parts of the COL application.

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Section	Section Title	Conformance Evaluation
C.I.7.7	Control Systems Not Required for Safety	Conforms. There is no control system instrumentation or supporting system that has not been addressed in the referenced certified design or other parts of the COL application.
C.I.7.8	Diverse Instrumentation and Control Systems	Conforms. There is no diverse I&C system that has not been addressed in the referenced certified design or other parts of the COL application.
C.I.7.9	Data Communication Systems	Conforms. There are no data communication systems that have not been addressed in the referenced certified design or other parts of the COL application.
C.I.8	Electrical Power	Conforms
C.I.8.1	Introduction	Conforms. There are no safety-related or RTNSS onsite AC or DC loads that are added to the referenced certified design. There are no safety-related or RTNSS electrical systems that are beyond the scope of the referenced certified design.
C.I.8.2.1	Description	Conforms. Addressed in Section 8.2.
C.I.8.2.2	Analysis	Conforms. Addressed in Section 8.2.
C.I.8.3.1.1	Description	Conforms. Addressed in DCD Section 8.3.1 and in Section 8.3.
C.I.8.3.1.2	Analysis	Not applicable. Does not request information for passive designs.
C.I.8.3.1.3	Electrical Power System Calculations and Distribution System Studies for AC Systems	Conforms
C.I.8.3.2.1	Description	Not applicable. Does not request information for passive designs.
C.I.8.3.2.2	Analysis	Not applicable. Does not request information for passive designs.
C.I.8.3.2.3	Electrical Power System Calculations, and Distribution System Studies for DC Systems	Conforms
C.I.8.4.1	Description	Conforms. Addressed in DCD Section 15.5.5.
C.I.8.4.2	Analysis	Not applicable. Does not request information for passive designs.
C.I.9.1.1	Criticality Safety of Fresh and Spent Fuel Storage and Handling	Conforms. Addressed in DCD Sections 9.1.1 and 9.1.2.
C.I.9.1.2	New and Spent Fuel Storage	Conforms. Addressed in DCD Section 9.1.2.
C.I.9.1.3	Spent Fuel Pool Cooling and Cleanup System	Conforms. Addressed in DCD Section 9.1.3.

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Section	Section Title	Conformance Evaluation
C.I.9.1.4	Light Load Handling System (Related to Refueling)	Conforms
C.I.9.1.5	Overhead Heavy Load Handling System	Conforms. Addressed in DCD Section 9.1.5.5 and in Sections 9.1.4 and 9.1.5.
C.I.9.2.1.1	Design Bases	Conforms. Addressed in DCD Section 9.2.1.1.
C.I.9.2.1.2	System Description	Conforms. Addressed in DCD Section 9.2.1.2 and in Section 9.2.1.2.
C.I.9.2.1.3	Safety Evaluation	Conforms. Addressed in DCD Section 9.2.1.3 and in Section 9.2.1.2 (for long-term corrosion and fouling).
C.I.9.2.1.4	Inspection and Testing Requirements	Conforms. Addressed in DCD Section 9.2.1.4.
C.I.9.2.1.5	Instrumentation Requirements	Conforms. Addressed in DCD Section 9.2.1.5.
C.I.9.2.2	Cooling System for Reactor Auxiliaries (Closed Cooling Water Systems)	Conforms
C.I.9.2 (for DCD Section 9.2.3)	Makeup Water System Design Bases	Conforms. Design Bases, Safety Evaluation, Inspection and Testing Requirements, and Instrumentation are addressed in DCD Section 9.2.3. System Description is addressed in Section 9.2.3.
C.I.9.2.4	Potable and Sanitary Water Systems	Conforms
C.I.9.2.5	Ultimate Heat Sink	The design of the UHS is within the scope of the referenced certified design, and inspection and testing requirements are addressed in DCD Section 9.2.5.
C.I.9.2.6	Condensate Storage Facilities	Conforms. There are no safety-related or RTNSS condensate storage facilities outside the scope of the referenced certified design that are sources of water for residual heat removal or sources of coolant inventory makeup for safety-related systems.
C.I.9.2 (for DCD Section 9.2.7)	Chilled Water System	Conforms. Addressed in DCD Section 9.2.7.
C.I.9.2 (for DCD Section 9.2.8)	Turbine Component Cooling Water System	Conforms. Addressed in DCD Section 9.2.8.

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Section	Section Title	Conformance Evaluation
C.I.9.2 (for DCD Section 9.2.10)	Station Water System	Conforms. Design Bases, Safety Evaluation, Inspection and Testing Requirements, and Instrumentation are addressed in DCD Section 9.2.10. System Description is addressed in Section 9.2.10.
C.I.9.3	Process Auxiliaries	Conforms. Hydrogen Water Chemistry is addressed in Section 9.3.9, Oxygen Injection System is addressed in Section 9.3.10, Zinc Injection System is addressed in Section 9.3.11, and Auxiliary Boiler System is addressed in DCD Section 9.3.12.
C.I.9.3 1	Compressed Air Systems	Conforms. Instrument Air is addressed in DCD Section 9.3.6, Service Air is addressed in DCD Section 9.3.7, and High Pressure Nitrogen Supply System is addressed in DCD Section 9.3.8.
C.I.9.3.2	Process and Postaccident Sampling Systems	Conforms
C.I.9.3.3	Equipment and Floor Drain System	Conforms. Addressed in DCD Section 9.3.3.
C.I.9.3.4	Chemical and Volume Control System (PWRs) (Including Boron Recovery System)	Not applicable. Applies only to PWRs.
C.I.9.3.5	Standby Liquid Control System (BWRs)	Conforms
C.I.9.4	Air Conditioning, Heating, Cooling, and Ventilation Systems	Conforms. Reactor Building HVAC System is addressed in DCD Section 9.4.6. Electric Building HVAC System is addressed in DCD Section 9.4.7, and Drywell Cooling System is addressed in DCD Section 9.4.8.
C.I.9.4.1	Control Room Area Ventilation System	Conforms
C.I.9.4.2	Spent Fuel Pool Area Ventilation Systems	Conforms
C.I.9.4.3	Auxiliary and Radwaste Area Ventilation System	Conforms
C.I.9.4.4	Turbine Building Area Ventilation System	Conforms
C.I.9.4.5	Engineered Safety Feature Ventilation System	Conforms
C.I.9.5.1	Fire Protection Program	Conforms
C.I.9.5.1.1(1)	Design Bases	Conforms
C.I.9.5.1.1(2)	Design Bases	Conforms
C.I.9.5.1.1(3)	Design Bases	Conforms. Addressed in Section 1.7.

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Section	Section Title	Conformance Evaluation
C.I.9.5.1.1(4)	Design Bases	Conforms. Will be completed in accordance with the milestones in Section 13.4.
C.I.9.5.1.1(5)	Design Bases	Conforms. Will be completed in accordance with the milestones in Section 13.4.
C.I.9.5.1.1(6)	Design Bases	Conforms
C.I.9.5.1.1(7)	Design Bases	Conforms. Will be completed in accordance with the milestones in Section 13.4.
C.I.9.5.1.1(8)	Design Bases	Conforms
C.I.9.5.1.1(9)	Design Bases	Conforms. Addressed in DCD Sections 9.5.1.15 and 14.3, and in Section 13.4.
C.I.9.5.2	Communication System	Conforms. Addressed in DCD Section 9.5.2 and in Section 9.5.2.
C.I.9.5.3	Lighting System	Conforms. Addressed in DCD Section 9.5.3.
C.I.9.5.4	Diesel Generator Fuel Oil Storage and Transfer Systems	Conforms. Addressed in DCD Section 9.5.4 and in Section 9.5.4.
C.I.9.5.4.1	Design Basis	Conforms. Addressed in DCD Section 9.5.4.
C.I.9.5.4.2	System Description	Conforms
C.I.9.5.4.3	Safety Evaluation	Conforms
C.I.9.5.5	Diesel Generator Cooling Water System	Conforms. Addressed in DCD Section 9.5.5.
C.I.9.5.6	Diesel Generator Starting System	Conforms. Addressed in DCD Section 9.5.6.
C.I.9.5.7	Diesel Generator Lubrication System	Conforms. Addressed in DCD Section 9.5.7.
C.I.9.5.8	Diesel Generator Combustion Air Intake and Exhaust System	Conforms. Addressed in DCD Section 9.5.8.
C.I.10.1	Introduction	Conforms. There are no principal design features of the steam and power conversion system that are outside the scope of the referenced certified design.
C.I.10.2.1 (1)	Design Bases	Conforms. Addressed in DCD Section 10.2.1.
C.I.10.2.1 (2)	Design Bases	Conforms. Addressed in DCD Section 10.2.2.
C.I.10.2.1 (3)	Design Bases	Conforms. Addressed in DCD Sections 3.5.1, 3.5.3, 3.6, and 10.2.4, and DCD Figure 3.5-2.
C.I.10.2.2 (1)	Description	Conforms. Addressed in DCD Sections 10.2.2, 10.2.3, and DCD Figures 1.2-12 to 1.2-20, 3.5-2, and Figure 10.1-1R.
C.I.10.2.2 (2)	Description	Conforms. Addressed in DCD Sections 10.2.2 and 10.2.3.
C.I.10.2.2 (3)	Description	Conforms. Addressed in DCD Section 10.2.2 and DCD Figures 10.2-1, 10.2-2, and 10.2-3.

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Section	Section Title	Conformance Evaluation
C.I.10.2.2 (4)	Description	Conforms. Addressed in DCD Sections 10.2.3 and 14.2.8.
C.I.10.2.2 (5)	Description	Conforms. Addressed in DCD Sections 12.2.1, 12.2.3, 12.4.4, DCD Table 12.2-23 and DCD Figures 12.3-12 to 12.3-18 and 12.3-32 to 12.3-38.
C.I.10.2.2 (6)	Description	Conforms. Addressed in DCD Sections 3.6, 10.2.2, and 10.2.4.
C.I.10.2.3 (1)	Turbine Rotor Integrity	Conforms. Addressed in DCD Section 10.2.3 and Section 10.2.3.8.
C.I.10.2.3 (2)	Turbine Rotor Integrity	Conforms. Addressed in DCD Section 10.2.3 and Section 10.2.3.8.
C.I.10.2.3 (3)	Turbine Rotor Integrity	Conforms. Addressed in DCD Section 10.2.3 and Section 10.2.3.8.
C.I.10.2.3 (4)	Turbine Rotor Integrity	Conforms. Addressed in DCD Section 10.2.3 and Section 10.2.3.8.
C.I.10.2.3 (5)	Turbine Rotor Integrity	Conforms. Addressed in DCD Sections 10.2.2 and 10.2.3, and Section 10.2.3.8.
C.I.10.3	Main Steam Supply System	Conforms. Addressed in DCD Section 10.3.
C.I.10.3.1 (1)	Design Bases	Conforms. Addressed in DCD Section 10.3.1.
C.I.10.3.1 (2)	Design Bases	Conforms. Addressed in DCD Section 10.3.
C.I.10.3.1 (3)	Design Bases	Conforms. Addressed in DCD Sections 10.3.2 and 10.3.3.
C.I.10.3.1 (4)	Design Bases	Conforms. Addressed in DCD Section 10.3.
C.I.10.3.1 (5)	Design Bases	Conforms. Addressed in DCD Section 10.3.
C.I.10.3.1 (6)	Design Bases	Conforms. Addressed in DCD Section 10.3.
C.I.10.3.2	Description	Conforms. Addressed in DCD Section 10.3.
C.I.10.3.3	Evaluation	Conforms. Addressed in DCD Section 10.3.
C.I.10.3.4	Inspection and Testing Requirements	Conforms. Addressed in DCD Section 10.3.4.
C.I.10.3.5	Water Chemistry (PWR Only)	Not applicable. Only applies to PWRs.
C.I.10.3.6 (1)	Steam and Feedwater System Materials	Conforms. Addressed in DCD Section 10.3.6.
C.I.10.3.6 (2)	Steam and Feedwater System Materials	Conforms. Addressed in DCD Sections 6.6 and 10.3.4.
C.I.10.3.6 (3)	Steam and Feedwater System Materials	Not applicable. DCD Section 10.3.6 states that there are no austenitic stainless steels in the steam and feedwater system piping.

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Section	Section Title	Conformance Evaluation
C.I.10.3.6 (4)	Steam and Feedwater System Materials	Not Applicable. DCD Section 10.3.6 states that there are no austenitic stainless steels in the ASME Code Section III Class 1 and 2 portions of steam and feedwater piping.
C.I.10.3.6 (5)	Steam and Feedwater System Materials	Conforms. Addressed in DCD Section 10.3.
C.I.10.3.6 (6)	Steam and Feedwater System Materials	Not applicable
C.I.10.4	Other Features of the Steam and Power Conversion System	Conforms
C.I.10.4.1	Main Condensers	Conforms. Sampling points for detection are discussed in DCD Section 10.4.1.5.4. Although sodium content and sampling for sodium content is not specifically mentioned in DCD Section 10.4.1, monitoring condensate for an increase in conductivity is considered an acceptable means to detect condenser tube leakage. A table of key parameters and associated action levels are provided in Table 10.4-201. Alarm setpoints are established to provide an indication of abnormal chemistry conditions prior to reaching a recommended action level.
C.I.10.4.2	Main Condenser Evacuation System	Conforms. There are no design features of the main condenser evacuation system that are outside the scope of the referenced certified design.
C.I.10.4.3 (1)	Turbine Gland Sealing System	Conforms. Addressed in DCD Section 10.4.3.
C.I.10.4.3 (2)	Turbine Gland Sealing System	Conforms with the following exception: For the operational phase, the QA Program is described in Chapter 17, and is based on NQA-1, rather than RG 1.33.
C.I.10.4.4	Turbine Bypass System	Conforms. The Turbine Bypass System is consistent with the referenced certified design.
C.I.10.4.5 (1)	Circulating Water System	Conforms
C.I.10.4.5 (2)	Circulating Water System	Not applicable. The Circulating Water System does not interface with the UHS.
C.I.10.4.6 (1)	Condensate Cleanup System	Conforms
C.I.10.4.6 (2)	Condensate Cleanup System	Conforms. Addressed in DCD Section 10.4.1, 10.4.6, and 5.2.3, DCD Table 5.2-5, and in Table 10.4-201.
C.I.10.4.6 (3)	Condensate Cleanup System	Conforms
C.I.10.4.6 (4)	Condensate Cleanup System	Not applicable. Only applies to PWRs.

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Section	Section Title	Conformance Evaluation
C.I.10.4.7 (1)	Condensate and Feedwater Systems	Not applicable. Only applies to PWRs.
C.I.10.4.7 (2)	Condensate and Feedwater Systems	Conforms. Addressed in DCD Sections 1.2.2 and 5.2.4, and DCD Tables 1.9-22 and 1.11-1.
C.I.10.4.7 (3)	Condensate and Feedwater Systems	Not applicable. The condensate and feedwater systems are consistent with the referenced certified design.
C.I.10.4.8	Steam Generator Blowdown System (PWR)	Not applicable. Only applies to PWRs.
C.I.10.4.9	Auxiliary Feedwater System (PWR)	Not applicable. Only applies to PWRs.
C.I.11.1	Source Terms	Conforms
C.I.11.2.1(1)	Design Bases	Conforms. Addressed in DCD Section 11.2 and in Section 11.2.
C.I.11.2.1(2)	Design Bases	Conforms. Addressed in DCD Section 11.2.
C.I.11.2.1(3)	Design Bases	Conforms. Addressed in DCD Section 11.2.1 and DCD Table 11.2-3. Conformance with RG 1.140 is addressed in DCD Section 9.4.3.
C.I.11.2.1(4)	Design Bases	Conforms. Addressed in DCD Section 9.4.3.
C.I.11.2.1(5)	Design Bases	Conforms. Addressed in DCD Sections 11.2.3 and 15.3.16 and in Section 2.4.13.
C.I.11.2.1(6)	Design Bases	Conforms. Quality Assurance Program requirements are addressed in Chapter 17.
C.I.11.2.1(7)	Design Bases	Conforms. Addressed in DCD Section 11.2.4.
C.I.11.2.1(8)	Design Bases	Conforms
C.I.11.2.1(9)	Design Bases	Conforms. Addressed in DCD Section 11.2.2 and in Section 11.2.
C.I.11.2.2(1)	System Description	Conforms. Addressed in DCD Section 11.2.2.
C.I.11.2.2(2)	System Description	Conforms. Addressed in DCD Section 11.2.2.
C.I.11.2.2(3)	System Description	Conforms. Addressed in DCD Section 11.2.2.
C.I.11.2.2(4)	System Description	Conforms. Addressed in DCD Section 11.2.2.
C.I.11.2.3(1)	Radioactive Effluent Releases	Conforms. Addressed in DCD Sections 11.2 and 12.2, and in Section 12.2.
C.I.11.2.3(2)	Radioactive Effluent Releases	Conforms. Addressed in DCD Sections 11.2 and 12.2, and in Section 12.2.
C.I.11.3.1(1)	Design Bases	Addressed in DCD Section 11.3. Conforms with the following exception: No discussion is provided regarding the capability of and requirements for using portable processing equipment for refueling outages.
C.I.11.3.1(2)	Design Bases	Conforms. Addressed in DCD Section 11.3.

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Section	Section Title	Conformance Evaluation
C.I.11.3.1(3)	Design Bases	Conforms. Addressed in DCD Section 11.3.
C.I.11.3.1(4)	Design Bases	Conforms. Quality Assurance Program requirements are addressed in Chapter 17.
C.I.11.3.1(5)	Design Bases	Conforms. Addressed in DCD Section 11.3.5.
C.I.11.3.1(6)	Design Bases	Conforms. Addressed in DCD Section 12.6 and in Section 12.6.
C.I.11.3.1(7)	Design Bases	Conforms. Addressed in DCD Section 11.3.
C.I.11.3.2(1)	System Description	Conforms. Addressed in DCD Section 11.3.2.
C.I.11.3.2(2)	System Description	Conforms. Addressed in DCD Section 11.3.2.
C.I.11.3.2(3)	System Description	Conforms. Addressed in DCD Section 11.3.2.
C.I.11.3.2(4)	System Description	Conforms. Addressed in DCD Sections 11.3.2, 11.3.3, and 9.4.
C.I.11.3.3	Radioactive Effluent Releases	Conforms. Addressed in DCD Sections 11.3 and 12.2, and in Section 12.2.
C.I.11.4.1(1)	Design Bases	Conforms. Addressed in DCD Section 11.4 and in Section 11.4.
C.I.11.4.1(2)	Design Bases	Conforms. Addressed in DCD Section 11.4 and in Section 11.4.
C.I.11.4.1(3)	Design Bases	Conforms. Addressed in DCD Section 11.4 and in Section 11.4.
C.I.11.4.1(4)	Design Bases	Conforms. Addressed in DCD Section 11.4 and in Sections 11.4, 13.5, and 17.5.
C.I.11.4.1(5)	Design Bases	Conforms. Addressed in DCD Section 11.4 and in Section 11.4.
C.I.11.4.1(6)	Design Bases	Conforms
C.I.11.4.1(7)	Design Bases	Conforms. Addressed in DCD Section 11.4.
C.I.11.4.2(1)	System Description	Addressed in DCD Section 11.4 and in Section 11.4. Conforms with the following exception: The FSAR provides a description of the PCP. Detailed waste packaging methodologies will be provided in the PCP. The implementation milestone is provided in Section 13.4.
C.I.11.4.2(2)	System Description	Addressed in DCD Section 11.4 and in Section 11.4. Conforms with the following exception: The FSAR provides a description of the PCP. Detailed waste packaging methodologies will be provided in the PCP. The implementation milestone is provided in Section 13.4.

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Conformance With the FSAR Content Guidance In RG 1.206

Section	Section Title	Conformance Evaluation
C.I.11.4.2(3)	System Description	Addressed in DCD Section 11.4 and in Section 11.4. Conforms with the following exception: The FSAR provides a description of the PCP. Detailed waste packaging methodologies will be provided in the PCP. The implementation milestone is provided in Section 13.4. There are no temporary onsite storage facilities.
C.I.11.4.2 (4)	System Description	Conforms. Addressed in DCD Section 11.4.
C.I.11.4.3 (1)	Radioactive Effluent Releases	Addressed in DCD Section 11.4 and in Section 11.4. Conforms with the following exception: The FSAR provides a description of the PCP. Detailed waste packaging methodologies will be provided in the PCP. The implementation milestone is provided in Section 13.4.
C.I.11.4.3 (2)	Radioactive Effluent Releases	Conforms. Addressed in DCD Sections 3.1 and 11.4.
C.I.11.4.3 (3)	Radioactive Effluent Releases	Conforms. Addressed in DCD Section 12.2.
C.I.11.5.1	Design Bases	Conforms
C.I.11.5.2(1)	System Description	Conforms. Addressed in DCD Section 11.5.
C.I.11.5.2 (2)	System Description	Conforms with the following exception: Section 11.5 provides a description of the ODCM. The implementation milestone is provided in Section 13.4.
C.I.11.5.2 (3)	System Description	Conforms with the following exception: Section 11.5 and TS Section 5 provide a description of the radiological effluent controls. The implementation milestone is provided in Section 13.4.
C.I.11.5.2 (4)	System Description	Conforms with the following exception: FSAR Section 11.5 and TS Section 5 provide a description of the REMP. The implementation milestone is provided in Section 13.4.
C.I.11.5.2 (5)	System Description	Conforms. Addressed in DCD Sections 3.1 and 11.5.
C.I.11.5.2 (6)	System Description	Conforms
C.I.11.5.2 (7)	System Description	Conforms
C.I.11.5.3	Effluent Monitoring and Sampling	Conforms
C.I.11.5.4	Process Monitoring and Sampling	Conforms
C.I.12.1.1	Policy Considerations	Conforms. Addressed in Sections 12.1 and 12.5.
C.I.12.1.2	Design Considerations	Conforms. Addressed in Section 12.5.
C.I.12.1.3	Operational Consideration	Conforms. Addressed in Sections 12.1 and 12.5.
C.I.12.2.1	Contained Sources	Conforms

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Section	Section Title	Conformance Evaluation
C.I.12.2.2	Airborne Radioactive Material Sources	Conforms
C.I.12.3.1	Facility Design Features	Conforms
C.I.12.3.2	Shielding	Conforms
C.I.12.3.3	Ventilation	Conforms. Addressed in DCD Sections 9.4.1 and 12.3.
C.I.12.3.4	Area Radiation and Airborne Radioactivity Monitoring Instrumentation	Conforms
C.I.12.3.5	Dose Assessment	Conforms. Addressed in DCD Section 12.4 and in Section 12.4.
C.I.12.4	Dose Assessment	Conforms
C.I.12.5 (1) (a)	Operational Radiation Protection Program: Organization	Conforms. Addressed in Sections 12.5 and 13.1.
C.I.12.5 (1) (b)	Facilities	Conforms
C.I.12.5 (1) (c)	Instrumentation and Equipment	Conforms
C.I.12.5 (1) (d)	Procedures	Conforms
C.I.12.5 (1) (e)	Training	Conforms. Addressed in Sections 12.5 and 13.2.
C.I.12.5 (2)		Conforms. Addressed in DCD Section 12.3.
C.I.12.5 (3)		Conforms. Addressed in Sections 12.5 and 13.1, and 13.4.
C.I.12.5 (4)		Conforms. Addressed in Section 13.4.
C.I.12.5, last paragraph		Conforms. Addressed in Sections 12.5, 13.1, 13.2, and 13.5.
C.I.12.5.1	Organization	Conforms. Addressed in Sections 12.5 and 13.1.
C.I.12.5.2	Equipment, Instrumentation, and Facilities	Conforms
C.I.12.5.3	Procedures	Addressed in Sections 12.5, 13.2, 13.5, and 17.5. Conforms with one exception: With respect to RG 1.33, Exelon's QA procedures follow NQA-1 rather than the older standards referenced in RG 1.33. The QA requirements are described in Section 17.5.
C.I.13.1.1(1)	Management and Technical Support Organization	Conforms. Addressed in Sections 13.1and 14.2.
C.I.13.1.1(2)	Management and Technical Support Organization	Conforms
C.I.13.1.1(3)	Management and Technical Support Organization	Conforms

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Section	Section Title	Conformance Evaluation
C.I.13.1.1(4)	Management and Technical Support Organization	Conforms
C.I.13.1.1(5)	Management and Technical Support Organization	Conforms
C.I.13.1.1(6)	Management and Technical Support Organization	Conforms
C.I.13.1.1(7)	Management and Technical Support Organization	Conforms. Addressed in in Sections 13.1and 14.2.
C.I.13.1.1.1	Design, Construction, and Operating Responsibilities	Conforms
C.I.13.1.1.2	Organizational Arrangement	Conforms. Addressed in Sections 13.1 and 17.5. The VCS site does not have an existing operating nuclear unit.
C.I.13.1.1.3	Qualifications	Conforms. Addressed in Sections 13.1 and 17.5.
C.I.13.1.2(1)	Operating Organization	Exception. The guidelines of RG 1.33 are met through equivalent administrative controls described in Chapter 17.
C.I.13.1.2(2)	Operating Organization	Exception. The guidelines of RG 1.33 are met through equivalent administrative controls described in Chapter 17.
C.I.13.1.2(3)	Operating Organization	Conforms. Addressed in Sections 9.5.1 and 13.1.
C.I.13.1.2(4)	Operating Organization	Conforms
C.I.13.1.2(5)	Operating Organization	Conforms
C.I.13.1.2(6)	Operating Organization	Conforms
C.I.13.1.2(7)	Operating Organization	Conforms
C.I.13.1.2(8)	Operating Organization	Conforms. Addressed in Appendix 13AA.
C.I.13.1.2.1	Plant Organization	Conforms. Addressed in Sections 13.1 and 17.5.
C.I.13.1.2.2(1)	Plant Personnel Responsibilities and Authorities	Conforms. Addressed in Sections 13.1 and 17.5.
C.I.13.1.2.2(2)	Plant Personnel Responsibilities and Authorities	Conforms
C.I.13.1.2.2(3)	Plant Personnel Responsibilities and Authorities	Conforms
C.I.13.1.2.3	Operating Shift Crews	Conforms
C.I.13.1.3.1	Qualification Requirements	Conforms. Addressed in Sections 13.1 and 17.5.
C.I.13.1.3.2	Qualifications of Plant Personnel	Exception. Résumés will not be included in the application, but will be available for inspection at corporate headquarters upon request.
C.I.13.2.1	Plant Staff Training Program	Conforms

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Conformance With the FSAR Content Guidance In RG 1.206

Section	Section Title	Conformance Evaluation
C.I.13.2.1.1 Licensed Staff (1)	Program Description	Conforms with the following exceptions: 1) this item discusses inclusion of details of the licensed training program. As noted in NEI 06-13, which is incorporated by reference, the systematic approach to training (SAT) process is used to establish and maintain training programs. Course duration and content are determined by the SAT process and by administrative procedure and are not included in the FSAR section; 2) the requirement for a “contingency plan…in the event fuel loading is subsequently delayed”’ is met by the operator re-qualification program; and 3) the industry standard content for this section does not include a discussion of proposed schedule for licensed personnel.
C.I.13.2.1.1 Licensed Staff (2)	Program Description	Conforms
C.I.13.2.1.1 Licensed Staff (3)	Program Description	Conforms
C.I.13.2.1.1 Licensed Staff (4)	Program Description	Conforms
C.I.13.2.1.1 Licensed Staff (5)	Program Description	Conforms
C.I.13.2.1.1 Licensed Staff (6)	Program Description	Conforms
C.I.13.2.1.1 Non-licensed Staff (1)	Program Description	Conforms
C.I.13.2.1.1 Non-licensed Staff (2)	Program Description	Conforms
C.I.13.2.1.1 Non-licensed Staff (3)	Program Description	Exception: This item discusses programs not covered under 10 CFR 50.120. As noted in NEI 06-13, which is incorporated by reference, the systematic approach to training (SAT) process is used to establish and maintain training programs. Course duration and content are determined by the SAT process and by administrative procedure and are not included in the FSAR section.

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Conformance With the FSAR Content Guidance In RG 1.206

Section	Section Title	Conformance Evaluation
C.I.13.2.1.1 Non-licensed Staff (4)	Program Description	Conforms. Addressed in Section 9.5.1.
C.I.13.2.1.1 Non-licensed Staff (5)	Program Description	Conforms
C.I.13.2.1.1 Non-licensed Staff (6)	Program Description	Conforms with the following exception: The first part of this item discusses detailed course descriptions. As noted in NEI 06-13, which is incorporated by reference, the systematic approach to training (SAT) process is used to establish and maintain training programs. Course duration and content are determined by the SAT process and by administrative procedure and are not included in the FSAR section. The implementation milestone is addressed in Section 13.4.
C.I.13.2.1.1 Non-licensed Staff (7)	Program Description	Conforms
C.I.13.2.1.2	Coordination with Preoperational Tests and Fuel Loading	Conforms with the following exception: Rather than providing contingency plans for training in the event of significantly delayed fuel loading the retraining programs are used, as described in NEI 06-13. Figure 13.1-202 shows the training schedule relative to fuel loading.
C.I.13.2.2(1)	10 CFR Part 19	Conforms
C.I.13.2.2(2)	10 CFR Part 26	Conforms
C.I.13.2.2(3)	10 CFR Part 50	Conforms
C.I.13.2.2(4)	10 CFR Part 50, Appendix E	Conforms
C.I.13.2.2(5)	10 CFR Part 52	Conforms
C.I.13.2.2(6)	10 CFR Part 55	Conforms
C.I.13.2.2(7)	RG 1.8	Addressed in Table 1.9-202.
C.I.13.2.2(8)	RG 1.149	Addressed in Table 1.9-202.
C.I.13.2.2(9)	NUREG-0711	Conforms. HFE addressed in DCD Chapter 18.
C.I.13.2.2(10)	NUREG-1021	Exception: Industry standard content for this section does not explicitly include discussion of compliance with NUREG-1021, Operator Licensing Examination Standards for Power Reactors.
C.I.13.2.2(11)	NUREG-1220	Not applicable. NUREG-1220 provides instructions for NRC inspectors.
C.I.13.2.2(12)	GL 86-04	Conforms

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Conformance With the FSAR Content Guidance In RG 1.206

Section	Section Title	Conformance Evaluation
C.I.13.2.2(13)	RG 1.134	Conforms. Industry standard content for this section does not explicitly include a discussion of compliance with RG 1.134, Medical Evaluations.
C.I.13.3(1)	Emergency Planning	Conforms. Addressed in the Emergency Plan in COLA Part 5.
C.I.13.3(2)	Emergency Planning	Conforms. Addressed in the Emergency Plan in COLA Part 5.
C.I.13.3(3)	Emergency Planning	Conforms. Addressed in the Emergency Plan in COLA Part 5.
C.I.13.3(4)	Emergency Planning	Conforms. Addressed in Chapter 2, and the Emergency Plan and Evacuation Time Estimate in COLA Part 5.
C.I.13.3(5)	Emergency Planning	Conforms. Addressed in COLA Part 5.
C.I.13.3(6)	Emergency Planning	Not applicable. Applies when state and/or local governments decline to participate in emergency planning and preparedness.
C.I.13.3(7)	Emergency Planning	Conforms
C.I.13.3.1 (1)	Combined License Application and Emergency Plan Content	Conforms. Addressed in COLA Part 5.
C.I.13.3.1 (2)	Combined License Application and Emergency Plan Content	Conforms. Addressed in COLA Parts 5 and 10.
C.I.13.3.1 (3)	Combined License Application and Emergency Plan Content	Conforms. Addressed in Chapter 1 and the Emergency Plan in COLA Part 5.
C.I.13.3.1 (4)	Combined License Application and Emergency Plan Content	Conforms. Addressed in the Emergency Plan in COLA Part 5.
C.I.13.3.1 (5)	Combined License Application and Emergency Plan Content	Conforms. Addressed in the Emergency Plan in COLA Part 5.
C.I.13.3.1 (6)	Combined License Application and Emergency Plan Content	Conforms. Addressed in the Emergency Plan in COLA Part 5.
C.I.13.3.1 (7)	Combined License Application and Emergency Plan Content	Conforms. Addressed in Chapter 1.
C.I.13.3.1 (8)	Combined License Application and Emergency Plan Content	Conforms. Addressed in the Emergency Plan in COLA Part 5.
C.I.13.3.1 (9)	Combined License Application and Emergency Plan Content	Conforms. Addressed in the Emergency Plan in COLA Part 5.
C.I.13.3.2	Emergency Plan Considerations for Multiunit Sites	Not applicable. There are no nearby operating reactors.
C.I.13.3.3	Emergency Planning Inspections, Tests, Analyses, and Acceptance Criteria	Conforms. Addressed in COLA Part 10.

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Section	Section Title	Conformance Evaluation
C.I.13.4	Operational Program Implementation	Conforms
C.I.13.5.1	Administrative Procedures	Conforms. Addressed in Sections 13.5 and 17.5.
C.I.13.5.2.1	Operating and Emergency Operating Procedures	Conforms with the following exception: Section 13.5.1 identifies classes of procedures by topic or type in lieu of the specific title. Operating procedures will be developed after activities such as job and task analyses have been completed.
C.I.13.5.2.2	Maintenance and Other Operating Procedures	Conforms
C.I.13.6	Security	Conforms. Addressed in Sections 13.4 and 13.6, and COLA Part 8.
C.I.13.7	Fitness for Duty	Conforms
C.I.14.1	Specific Information to be Addressed for the Initial Plant Test Program	Conforms. Addressed in Sections 14.2 and 14.3.
C.I.14.2	Initial Plant Test Program	Conforms
C.I.14.2.1	Summary of Test Program and Objectives	Conforms
C.I.14.2.2	Organization and Staffing	Conforms. Addressed in DCD Section 14.2 and in Sections 13.1, 14.2, and 17.5.
C.I.14.2.3	Test Procedures	Conforms. Addressed in DCD Section 14.2.
C.I.14.2.4	Conduct of Test Program	Conforms. Addressed in DCD Section 14.2.
C.I.14.2.5	Review, Evaluation, and Approval of Test Results	Conforms. Addressed in DCD Section 14.2.
C.I.14.2.6	Test Records	Conforms
C.I.14.2.7	Conformance of Test Programs with Regulatory Guides	Conforms. Addressed in DCD Section 14.2.3.
C.I.14.2.8	Utilization of Reactor Operating and Testing Experiences in Development of Test Program	Conforms. Addressed in DCD Section 14.2 and in Section 14.2.
C.I.14.2.9	Trial Use of Plant Operating and Emergency Procedures	Conforms. Addressed in DCD Section 14.2.5 and in Section 13.2.
C.I.14.2.10	Initial Fuel Loading and Initial Criticality	Conforms. Addressed in DCD Section 14.2.6.
C.I.14.2.11	Test Program Schedule	Conforms. Addressed in DCD Section 14.2.7 and in Section 14.2.7.
C.I.14.2.12	Individual Test Descriptions	Conforms. Addressed in DCD Section 14.2.8 and in Section 14.2.9.

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Section	Section Title	Conformance Evaluation
C.I.14.3	Inspections, Tests, Analyses, and Acceptance Criteria	Conforms. Addressed in COLA Part 10.
C.I.15.1	Transient and Accident Classification	Conforms. There are no aspects of the site-specific design that affect the transient and accident analyses in the DCD.
C.I.15.2	Frequency of Occurrence	Conforms
C.I.15.3	Plant Characteristics Considered in the Safety Evaluation	Conforms
C.I.15.4	Assumed Protection System Actions	Conforms
C.I.15.5	Evaluation of Individual Initiating Events	Conforms
C.I.15.6.1	Identification of Causes and Frequency Classification	Conforms
C.I.15.6.2	Sequence of Events and Systems Operation	Conforms
C.I.15.6.3	Core and System Performance	Conforms
C.I.15.6.4	Barrier Performance	Conforms
C.I.15.6.5	Radiological Consequences	Conforms. Table 2.0-201 compares the site-specific short-term χ/Qs for the EAB, LPZ, and control room to the χ/Qs assumed in the DCD.
C.I.16.1	Technical Specifications and Bases	Conforms. Addressed in COLA Part 4. There are no deviations from the generic TS bases.
C.I.16.2	Content and Format of Technical Specifications and Bases	Conforms. Addressed in COLA Part 4. No plant-specific deviations from the referenced certified generic Technical Specifications or Bases are required and none are being requested (e.g., incorporation of TSTF travelers).
C.I.17.1	Quality Assurance During the Design and Construction Phase	Conforms
C.I.17.2	Quality Assurance During the Operations Phase	Conforms
C.I.17.3	Quality Assurance Program Description	Conforms
C.I.17.4.1	New Section 17.4 in the Standard Review Plan	Conforms
C.I.17.4.2	Reliability Assurance Program Scope, Stages, and Goals	Not applicable
C.I.17.4.3	Reliability Assurance Program Implementation	Conforms. Addressed in Sections 17.4 and 17.6.

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Section	Section Title	Conformance Evaluation
C.I.17.4.4	Reliability Assurance Program Information Needed in a COL Application	Conforms. Addressed in DCD Section 17.4 and in Sections 17.4, 17.5, and 17.6.
C.I.17.5.1	COL Applicant QA Program Responsibilities	Conforms
C.I.17.5.2	Updated SRP Section 17.5 and the QA Program Description	Conforms. QA applied to safety-related activities performed before the start of construction (e.g., site investigation, design and safety analysis, early procurements) is described in the Exelon Nuclear QA Topical Report. QA applied during activities to adapt the design to specific plant implementation, construction, and operations is addressed in Section 17.5.
C.I.17.5.3	Evaluation of the QAPD Against the SRP and QAPD Submittal Guidance	Conforms
C.I.17.6	Description of the Applicant's Program for Implementation of 10 CFR 50.65, the Maintenance Rule	Conforms
C.I.17.6.1	Scoping per 10 CFR 50.65(b)	Conforms
C.I.17.6.2	Monitoring per 10 CFR 50.65(a)	Conforms
C.I.17.6.3	Periodic Evaluation per 10 CFR 50.65(a)(3)	Conforms
C.I.17.6.4	Risk Assessment and Management per 10 CFR 50.65(a)(4)	Conforms
C.I.17.6.5	Maintenance Rule Training and Qualification	Conforms
C.I.17.6.6	Maintenance Rule Program Role in Implementation of Reliability Assurance Program (RAP) in the Operations Phase	Conforms
C.I.17.6.7	Maintenance Rule Program Implementation	Conforms
C.I.18	Human Factors Engineering	Conforms
	HFE principles incorporated into:	
	(1) Planning and management	Conforms. Addressed in DCD Section 18.2.
	(2) Plant design processes not closed with design certification	Conforms. Addressed in DCD Tier 1, ITAAC Table 3.3-1.
	(3) HSI, procedures, and training	Conforms. Addressed in DCD Tier 1, ITAAC Table 3.3-1, Items 6, 7, and 8.

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Section	Section Title	Conformance Evaluation
	(4) Implementation of the design	Conforms. Addressed in DCD Tier 1, ITAAC Table 3.3-1, Item 10.
	(5) Monitoring of performance at the site	Conforms. Addressed in DCD Tier 1, ITAAC Table 3.3-1, Item 11.
	Applicant program addresses normal and emergency, maintenance, test, inspection and surveillance activities	Conforms. Addressed in DCD Section 18.1.
	FSAR/DCD describe objectives and scope of the applicant's activities related to element, methodology, and results for (12 HFE elements)	Conforms. Addressed in DCD Sections 18.3 through 18.13.
	Applicant should reference detailed implementation plan reviewed and approved as part of design certification	Conforms. Addressed in DCD Section 18.2.1.
C.I.18.1	HFE Program Management	Conforms. Addressed in DCD Sections 18.2.2 and 18.2.3.
C.I.18.1.1	General HFE Program and Scope	Conforms. Addressed in DCD Sections 18.2.1 and 18.2.2.
C.I.18.1.2	HFE Team and Organization	Conforms. Addressed in DCD Section 18.2.3.
C.I.18.1.3	HFE Process and Procedures	Conforms. Addressed in DCD Sections 18.2.1 and 18.2.2.
C.I.18.1.4	HFE Issues Tracking	Conforms. Addressed in DCD Section 18.2.2.
C.I.18.1.5	HFE Technical Program	Conforms. Addressed in DCD Sections 18.3 through 18.13.
C.I.18.2.1	Objectives and Scope	Conforms. Addressed in DCD Section 18.3.1.
C.I.18.2.2.1	OER Process	Conforms. Addressed in DCD Section 18.3.2.
C.I.18.2.2.2	Predecessor Plants and Systems	Conforms. Addressed in DCD Section 18.3.2.1.
C.I.18.2.2.3	Risk-Important Human Actions	Conforms. Addressed in DCD Section 18.3.2.2.
C.I.18.2.2.4	HFE Technology	Conforms. Addressed in DCD Section 18.3.2.3.
C.I.18.2.2.5	Recognized Industry Issues	Conforms. Addressed in DCD Section 18.3.2.4.
C.I.18.2.2.6	Issues Identified by Plant Personnel	Conforms. Addressed in DCD Section 18.3.2.5.
C.I.18.2.2.7	Issue Analysis, Tracking, and Review	Conforms. Addressed in DCD Section 18.3.2.6.
C.I.18.2.3	Results	Conforms. Addressed in DCD Section 18.3.3.
C.I.18.3.1	Objectives and Scope	Conforms. Addressed in DCD Section 18.4.2.
C.I.18.3.1.1	Functional Requirements Analysis	Conforms. Addressed in DCD Section 18.4.1.
C.I.18.3.1.2	Function Allocation Analysis	Conforms. Addressed in DCD Section 18.4.2.

Table 1.9-203 (Sheet 34 of 35)
Conformance With the FSAR Content Guidance In RG 1.206

Section	Section Title	Conformance Evaluation
C.I.18.3.2.1	Methodology for Functional Requirements Analysis	Conforms. Addressed in DCD Section 18.4.1.
C.I.18.3.2.2	Methodology for Function Allocation Analysis	Conforms. Addressed in DCD Section 18.4.2.
C.I.18.3.3	Results	Conforms. Addressed in DCD Sections 18.4.1 and 18.4.2.
C.I.18.4.1	Objectives and Scope	Conforms. Addressed in DCD Section 18.5.1.
C.I.18.4.2	Methodology	Conforms. Addressed in DCD Section 18.5.1.
C.I.18.4.3	Results	Conforms. Addressed in DCD Section 18.5.1.
C.I.18.5.1	Objectives and Scope	Conforms. Addressed in DCD Section 18.6.2.
C.I.18.5.2	Methodology	Conforms. Addressed in DCD Sections 18.6.4 and 18.6.5.
C.I.18.5.3	Results	Conforms. Addressed in DCD Section 18.6.6.
C.I.18.6.1	Objectives and Scope	Conforms. Addressed in DCD Section 18.7.1.
C.I.18.6.2	Methodology	Conforms. Addressed in DCD Section 18.7.2.
C.I.18.6.3	Results	Conforms. Addressed in DCD Section 18.7.3.
C.I.18.7.1	Objectives and Scope	Conforms. Addressed in DCD Section 18.8.1.
C.I.18.7.2.1	HSI Design Inputs	Conforms. Addressed in DCD Section 18.8.1.
C.I.18.7.2.2	Concept of Operations	Conforms. Addressed in DCD Section 18.8.1.
C.I.18.7.2.3	Functional Requirements Specification	Conforms. Addressed in DCD Section 18.8.1.
C.I.18.7.2.4	HSI Concept Design	Conforms. Addressed in DCD Section 18.8.1.
C.I.18.7.2.5	HSI Detailed Design and Integration	Conforms. Addressed in DCD Section 18.8.1.
C.I.18.7.2.6	HSI Tests and Evaluations	Conforms. Addressed in DCD Section 18.8.1.
C.I.18.7.3.1	Overview of HSI Design and Its Key Features	Conforms. Addressed in DCD Section 18.8.1(3).
C.I.18.7.3.2	Safety Aspects of the HSI	Conforms. Addressed in DCD Section 18.8.1(3).
C.I.18.7.3.3	HSI Change Process	Conforms. Addressed in DCD Section 18.8.1(4).
C.I.18.8.1	Objectives and Scope	Conforms. Addressed in DCD Section 18.9.1.
C.I.18.8.2	Methodology	Conforms. Addressed in DCD Section 18.9.2.
C.I.18.8.3	Results	Conforms. Addressed in DCD Section 18.9.3.
C.I.18.9.1	Objectives and Scope	Conforms. Addressed in DCD Sections 18.10.1 and 18.10.2.
C.I.18.9.2	Methodology	Conforms. Addressed in DCD Sections 18.10.3 and 18.10.4.
C.I.18.9.3	Results	Conforms. Addressed in DCD Section 18.10.5.

Table 1.9-203 (Sheet 35 of 35)
Conformance With the FSAR Content Guidance In RG 1.206

Section	Section Title	Conformance Evaluation
C.I.18.10.1	Objectives and Scope	Conforms. Addressed in DCD Section 18.11 and 18.11.1.
C.I.18.10.2	Methodology	Conforms. Addressed in DCD Section 18.11.
C.I.18.10.2.1	Operational Conditions Sampling	Conforms. Addressed in DCD Section 18.11.
C.I.18.10.2.2	Design Verification	Conforms. Addressed in DCD Section 18.11.
C.I.18.10.2.3	Integrated System Validation	Conforms. Addressed in DCD Section 18.11.
C.I.18.10.2.4	Human Engineering Discrepancy Resolution	Conforms. Addressed in DCD Section 18.11.
C.I.18.10.3	Results	Conforms. Addressed in DCD Section 18.11.2.
C.I.18.11.1	Objectives and Scope	Conforms. Addressed in DCD Section 18.12.1.
C.I.18.11.2	Methodology	Conforms. Addressed in DCD Section 18.12.2.
C.I.18.11.3	Results	Conforms. Addressed in DCD Section 18.12.3.
C.I.18.12.1	Objectives and Scope	Conforms. Addressed in DCD Sections 18.13.1 and 18.13.2.
C.I.18.12.2	Methodology	Conforms. Addressed in DCD Sections 18.13.2 and 18.13.3.
C.I.18.12.3	Results	Conforms. Addressed in DCD Section 18.13.4.
C.I.19	Probabilistic Risk Assessment and Severe Accident Evaluation	Conforms

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Table 1.9-204 (Sheet 1 of 3)
Industrial Codes and Standards

Code or Standard Number	Year	Title
American Society of Mechanical Engineers (ASME)		
A17.1	2007	Safety Code for Elevators and Escalators
B31.1	2007	Power Piping
NQA-1	2004	Quality Assurance Programs Requirements for Nuclear Facilities
Boiler and Pressure Vessel Code, Section IX	2007	Qualification Standard for Welding and Brazing Procedures, Welders, Brazers, and Welding and Brazing Operators
ASTM International		
ASTM E-84	2007	Standard Test Method for Surface Burning Characteristics of Building Materials
ASTM E-119	2007	Standard Test Methods for Fire Tests of Building Construction and Materials
ASTM E-814	2006	Standard Test Method for Fire Tests of Through-Penetration Fire Stops
Applicable Building Codes		
International Building Code	2007	International Building Code
International Fire Code	2007	International Fire Code
28 CFR 36	2007	Nondiscrimination on the Basis of Disability by Public Accommodations and in Commercial Facilities
Factory Mutual		
Data Sheet 7-42	2006	Guidelines for Evaluating the Effects of Vapor Cloud Explosions Using a TNT Equivalency Method
	2007	Approval Guide
Institute of Electrical and Electronics Engineers (IEEE)		
C2	2007	National Electric Safety Code
National Fire Protection Association (NFPA)		
NFPA 1	2006	Uniform Fire Code Handbook
NFPA 10	2007	Standard for Portable Fire Extinguishers
NFPA 11	2005	Standard for Low-, Medium-, and High-Expansion Foam
NFPA 12	2008	Standard on Carbon Dioxide Extinguishing Systems
NFPA 13	2007	Standard for the Installation of Sprinkler Systems
NFPA 14	2007	Standard for the Installation of Sandpipe and Hose Systems

Table 1.9-204 (Sheet 2 of 3)
Industrial Codes and Standards

Code or Standard Number	Year	Title
NFPA 15	2007	Standard for Water Spray Fixed Systems for Fire Protection
NFPA 16	2007	Standard for the Installation of Foam-Water Sprinkler and Foam-Water Spray Systems
NFPA 20	2007	Standard for the Installation of Stationary Pumps for Fire Protection
NFPA 22	2008	Standard for Water Tanks for Private Fire Protection
NFPA 24	2007	Standard for the Installation of Private Fire Service Mains and their Appurtenances
NFPA 25	2008	Standard for the Inspection, Testing, and Maintenance of Water-Based Fire Protection Systems
NFPA 30	2008	Flammable and Combustible Liquids Code
NFPA 37	2006	Standard for the Installation and Use of Stationary Combustion Engines and Gas Turbines
NFPA 58	2008	Liquefied Petroleum Gas Code
NFPA 69	2008	Standard on Explosion Prevention Systems
NFPA 70	2008	National Electric Code
NFPA 72	2007	National Fire Alarm Code
NFPA 80	2007	Standard for Fire Doors and Other Opening Protectives
NFPA 80A	2007	Recommended Practice for Protection of Buildings from Exterior Fire Exposures
NFPA 90B	2006	Standard for the Installation of Warm Air Heating and Air-Conditioning Systems
NFPA 101	2006	Life Safety Code
NFPA 101A	2007	Guide on Alternative Approaches to Life Safety
NFPA 110	2005	Standard for Emergency and Standby Power Systems
NFPA 204	2007	Standard for Smoke and Heat Venting
NFPA 214	2005	Standard on Water-Cooling Towers
NFPA 252	2008	Standard Methods of Fire Tests of Door Assemblies
NFPA 255	2006	Standard Method of Test of Surface Burning Characteristics of Building Materials
NFPA 259	2008	Standard Test Method for Potential Heat of Building Material
NFPA 497	2008	Recommended Practice for the Classification of Flammable Liquids, Gases, or Vapors and of Hazardous (Classified) Locations for Electrical Installation in Chemical Process Areas
NFPA 750	2006	Standard on Water Mist Fire Protection Systems

Table 1.9-204 (Sheet 3 of 3)
Industrial Codes and Standards

Code or Standard Number	Year	Title
NFPA 780	2008	Standard for the Installation of Lightning Protection Systems
NFPA 801	2008	Standard for Fire Protection Practices for Facilities Handling Radioactive Materials
NFPA 1962	2008	Standard for the Inspection, Care, and Use of Fire Hose, Couplings, and Nozzles and the Service Testing of Fire Hose
NFPA 1964	2008	Standard for Spray Nozzles
NFPA 2001	2008	Standard for Clean Agent Fire Extinguishing Systems
Occupational Safety and Health Act (OSHA)		
29 CFR 1910	2006	Occupational Safety and Health Standards
29 CFR 1926	2006	Safety and Health Regulations for Construction
Underwriters Laboratories (UL)		
	2007	Fire Protection Equipment Directory
Environmental Protection Agency (EPA)		
40 CFR 60	2006	EPA Standards of Performance for Stationary Compression Ignition Internal Combustion Engines

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Table 1.9-205 (Sheet 1 of 2)
NUREG Reports Cited

NUREG No.	Issue Date	Title	Comment/Section Where Discussed
0612	07/1980	Control of Heavy Loads at Nuclear Power Plants	13.5
0654, Rev 1	03/2002	Criteria for Preparation and Evaluation of Radiological Emergency Response Plans and Preparedness in Support of Nuclear Power Plants	2.3
0696	02/1981	Functional Criteria for Emergency Response Facilities	2.3
0737	11/1980	Clarification of TMI Action Plan Requirements	2.3, 13.1, 13.5
1488	04/1994	Revised Livermore Seismic Hazard Estimates for Sixty-Nine Nuclear Power Plant Sites East of the Rocky Mountains	2.5
1736	10/2001	Consolidated Guidance: 10 CFR 20 - Standards for Protection Against Radiation	1.9
1853	01/2007	History and Framework of Commercial Low-Level Radioactive Waste Management in the United States	11.4
CP-0105	10/1989	Paleoseismic History of the Meers Fault, Southwestern Oklahoma, and its Implications for Evaluations of Earthquake Hazards in the Central and Eastern United States	2.5
CR-1486	06/1980	Seasonal Variation of 10-Square-Mile Probable Maximum Precipitation Extremes, United States East of the 105th Meridian	2.3
CR-1805	12/2004	Fire Dynamics Tools (FDTs) Quantitative Fire Hazard Analysis Methods for the U.S. Nuclear Regulatory Commission Fire Protection Inspection Program	2.2
CR-2858	11/1982	PAVAN: An Atmospheric Dispersion Program for Evaluating Design Basis Accidental Releases of Radioactive Materials from Nuclear Power Stations	2.3
CR-2919	09/1982	XOQDOQ Computer Program for the Meteorological Evaluation of Routine Effluent Releases at Nuclear Power Stations	2.3
CR-4461	02/2007	Tornado Climatology of the Contiguous United States	2.3
CR-4653	04/1987	GASPAR II Technical Reference and User Guide	12.2

Table 1.9-205 (Sheet 2 of 2)
NUREG Reports Cited

NUREG No.	Issue Date	Title	Comment/Section Where Discussed
CR-4852	03/1987	The Meers Fault: Tectonic Activity in Southwestern Oklahoma	2.5
CR-5250, Vol 1	01/1989	Seismic Characterization of 69 Nuclear Plant Sites East of the Rocky Mountains: Methodology	2.5
CR-5250, Vol 5	01/1989	Seismic Characterization of 69 Nuclear Plant Sites East of the Rocky Mountains: Results and Discussions for Batch 4 Sites	2.5
CR-5503	07/1999	Techniques for Identifying Faults and Determining Their Origins	2.5
CR-5512, Vol 1	04/2001	Residual Radioactive Contamination from Decommissioning, Vol. 1	2.4
CR-6034	07/1993	Oklahoma Seismic Network	2.5
CR-6331	05/1997	Atmospheric Relative Concentrations in Building Wakes	2.3
CR-6372	04/1997	Recommendations for Probabilistic Seismic Hazard Analysis: Guidance on Uncertainty and Use of Experts	2.5
CR-6728	10/2001	Technical Basis for Revision of Regulatory Guidance on Design Ground Motions: Hazard- and Risk-consistent Ground Motion Spectra Guidelines	2.5
CR-6769	04/2002	Technical Basis for Revision of Regulatory Guidance on Design Ground Motions: Development of Hazard- and Risk-Consistent Seismic Spectra for Two Sites	2.5

1.10 Summary of COL Items

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

Add the following at the end of this section.

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Table 1.10-201 lists the FSAR location(s) where the individual COL items from the DCD are addressed.

VCS SUP 1.10-1

Table 1.10-201 (Sheet 1 of 5)
Summary of FSAR Sections Where DCD COL Items Are Addressed

Item No.	Subject/Description of Item	FSAR Section
1.1-1-A	Establish Rated Electrical Output	1.1.2.7
1.3-1-A	Update Table 1.3-1	1.3
1.7-1-H	Final Design Configuration Confirmation	1.7.2
1.9-3-A	SRP and Regulatory Guide Applicability	SRP: Table 1.9-201 RGs: Table 1.9-202 RG 1.206: Table 1.9-203
1.11-1-A	Address Table 1.11-1 Items That Refer to Notes (2) and (7)	1.11.1 and Table 1.11-201
1C.1-1-A	Handling of Safeguards Information	1C.1, Table 1C-201
1C.1-2-A	Emergency Preparedness and Response Actions	1C.1, Table 1C-202, COLA Part 5
2.0-1-A	Site Characteristics Demonstration	2.0
2.0-2-A	Site Location and Description Information in Accordance with SRP 2.1.1	2.0 and 2.1.1
2.0-3-A	Site-Specific Exclusion Area Authority and Control Information in Accordance with SRP 2.1.2.	2.0 and 2.1.2
2.0-4-A	Describe the Population Distribution in Accordance with SRP 2.1.3	2.0 and 2.1.3
2.0-5-A	Identify Potential Hazards in the Site Vicinity, in Accordance with SRP 2.2.1 - 2.2.2	2.0 and 2.2.1 and 2.2.2
2.0-6-A	Evaluation of Potential Accidents in Accordance with SRP 2.2.3	2.0 and 2.2.3
2.0-7-A	Regional Climatology in Accordance with SRP 2.3.1	2.0 and 2.3.1
2.0-8-A	Local Meteorology in Accordance with SRP 2.3.2	2.0 and 2.3.2
2.0-9-A	Onsite Meteorological Measurement Programs in Accordance with SRP 2.3.3	2.0 and 2.3.3
2.0-10-A	Short-Term Diffusion Estimates for Accidental Atmospheric Releases in Accordance with SRP 2.3.4	2.0 and 2.3.4
2.0-11-A	Long-Term Diffusion Estimates in Accordance with SRP 2.3.5	2.0 and 2.3.5
2.0-12-A	Hydraulic Description Maximum Ground Water Level in Accordance with SRP 2.4.1	2.0 and 2.4.1
2.0-13-A	Protection of Below-Grade Penetrations and Access Openings from Floods in Accordance with SRP 2.4.2	2.0 and 2.4.2
2.0-14-A	Probable Maximum Flood on Streams and Rivers in Accordance with SRP 2.4.3	2.0 and 2.4.3
2.0-15-A	Potential Dam Failures Seismically Induced in Accordance with SRP 2.4.4	2.0 and 2.4.4
2.0-16-A	Probable Maximum Surge and Seiche Flooding in Accordance with SRP 2.4.5	2.0 and 2.4.5
2.0-17-A	Probable Maximum Tsunami in Accordance with SRP 2.4.6	2.0 and 2.4.6

Table 1.10-201 (Sheet 2 of 5)
Summary of FSAR Sections Where DCD COL Items Are Addressed

Item No.	Subject/Description of Item	FSAR Section
2.0-18-A	Ice Effects in Accordance with SRP 2.4.7	2.0 and 2.4.7
2.0-19-A	Cooling Water Canals and Reservoirs in Accordance with SRP 2.4.8	2.0 and 2.4.8
2.0-20-A	Channel Diversion in Accordance with SRP 2.4.9	2.0 and 2.4.9
2.0-21-A	Flooding Protection Requirements in Accordance with SRP 2.4.10	2.0 and 2.4.10
2.0-22-A	Cooling Water Supply in Accordance with SRP 2.4.11	2.0 and 2.4.11
2.0-23-A	Groundwater in Accordance with SRP 2.4.12	2.0 and 2.4.12
2.0-24-A	Accidental Releases of Liquid Effluents in Ground and Surface Waters in Accordance with SRP 2.4.13	2.0 and 2.4.13
2.0-25-A	Technical Specifications and Emergency Operation Requirements in Accordance with SRP 2.4.14	2.0 and 2.4.14
2.0-26-A	Basic Geologic and Seismic Information in Accordance with SRP 2.5.1	2.0 and 2.5.1
2.0-27-A	Vibratory Ground Motion in Accordance with SRP 2.5.2	2.0 and 2.5.2
2.0-28-A	Surface Faulting in Accordance with SRP 2.5.3	2.0 and 2.5.3
2.0-29-A	Stability of Subsurface Materials and Foundations in Accordance with SRP 2.5.4	2.0 and 2.5.4
2.0-30-A	Stability of Slopes in Accordance with SRP 2.5.5	2.0 and 2.5.5
3.6.5-1-A	Pipe Break Analysis Results and Protection Methods	3.6.2.5
3.9.9-1-H	Reactor Internals Vibration Analysis, Measurement and Inspection Program	3.9.2.4
3.9.9-2-H	ASME Class 2 or 3 or Quality Group D Components with 60-Year Design Life	3.9.3.1
3.9.9-3-A	Inservice Testing Programs	3.9.6
3.9.9-4-A	Snubber Inspection and Test Program	3.9.3.7.1(3)e and 3.9.3.7.1(3)f
3.10.4-1-A	Dynamic Qualification Report	3.10.1.4
3.11-1-A	Environmental Qualification Document (EQD)	3.11.2.2
4.3-1-A	Variances from Certified Design	4.3
4A-1-A	Variances from Certified Design	4A
5.2-1-H	Preservice and Inservice Inspection Program Plan	5.2.4 and 5.2.4.11
5.2-2-H	Leak Detection Monitoring	5.2.5.9
5.3-2-A	Materials and Surveillance Capsule	5.3.1.8
6.1.3-1-A	Protective Coatings and Organic Materials	6.1.2.3
6.2-1-H	Information indicated in Tables 6.2-16 through 6.2-42	6.2.4.2
6.4-1-A	Control Room Habitability Area (CRHA) Procedures and Training	6.4.4
6.4-2-A	Toxic Gas Analysis	2.2.3.1.3.4, 2.2.3.1.3.5, 6.4.5 and Table 2.2-202

Table 1.10-201 (Sheet 3 of 5)
Summary of FSAR Sections Where DCD COL Items Are Addressed

Item No.	Subject/Description of Item	FSAR Section
6.6-1-A	Preservice Inspection (PSI) and Inservice Inspection (ISI) Program Description	6.6, 6.6.10.1, 6.6.10.2, and 6.6.10.3
8.2.4-1-A	Transmission System Description	8.2.1.1
8.2.4-2-A	Switchyard Description	8.2.1.2.1
8.2.4-3-A	Normal Preferred Power	8.2.1.2
8.2.4-4-A	Alternate Preferred Power	8.2.1.2
8.2.4-5-A	Protective Relaying	8.2.1.2.1
8.2.4-6-A	Switchyard DC Power	8.2.1.2.1
8.2.4-7-A	Switchyard AC Power	8.2.1.2.1
8.2.4-8-A	Switchyard Transformer Protection	8.2.1.2.1
8.2.4-9-A	Stability and Reliability of the Offsite Transmission Power Systems	8.2.2.1
8.2.4-10-A	Interface Requirements	8.2.2.1
8A.2.3-1-A	Cathodic Protection System	8A.2.1
9.1.6-4-A	Fuel Handing Operations	9.1.4.13 and 9.1.4.19
9.1.6-5-A	Handling of Heavy Loads	9.1.5.6, 9.1.5.8 and 9.1.5.9
9.2.1-1-A	Material Selection	9.2.1.2
9.2.5-1-A	Post 7-Day Makeup to Ultimate Heat Sink (UHS)	9.2.5
9.3.2-1-A	Post-Accident Sampling Program	9.3.2.2
9.3.9-1-A	Implementation of Hydrogen Water Chemistry	9.3.9
9.3.9-2-A	Hydrogen and Oxygen Storage and Supply	9.3.9.2
9.3.10-1-A	Oxygen Storage Facility	9.3.10.2
9.3.11-1-A	Determine Need for Zinc Injection System	9.3.11.2
9.3.11-2-A	Provide System Description for Zinc Injection System	9.3.11.4
9.5.1-1-A	Secondary Firewater Storage Source	9.5.1.4
9.5.1-2-A	Secondary Firewater Capacity	9.5.1.4
9.5.1-4-A	Piping and Instrument Diagrams	9.5.1.2, 9.5.1.4, 9.5.1.5, Figures 9.5-201 and 9.5-202
9.5.1-5-A	Fire Barriers	9.5.1.10
9.5.1-6-H	Smoke Control	9.5.1.11
9.5.1-7-H	Fire Hazards Analysis (FHA) Compliance Review	9.5.1.12
9.5.1-8-A	Fire Protection (FP) Program Description	9.5.1.15
9.5.1-9-A	FP Licensing Changes	9.5.1.15.2
9.5.1-10-H	Fire Brigade	9.5.1.15.4 and 13.1.2.2
9.5.1-11-A	Quality Assurance	9.5.1.15.9
9.5.2.5-1-A	Offsite Interfaces	9.5.2.2

Table 1.10-201 (Sheet 4 of 5)
Summary of FSAR Sections Where DCD COL Items Are Addressed

Item No.	Subject/Description of Item	FSAR Section
9.5.2.5-2-A	Grid Transmission Operator	9.5.2.2
9.5.4-1-A	Fuel Oil Capacity	9.5.4.2
9.5.4-2-A	Protection of Underground Piping	9.5.4.2
9A.7-1-A	Yard Fire Zone Drawings	9A.4.7
9A.7-2-A	Fire Hazards Analysis for Site Specific Areas	9A.4.7, 9A.5.7, 9A.5.8 9A.5.9, and 9A.5.10
10.2-1-H	Turbine Missile Probability Analysis	10.2.3.8
10.4-1-A	Leakage (of Circulating Water Into the Condenser)	10.4.6.3
11.2-1-A	Implementation of IE Bulletin 80-10	11.2.2.3
11.2-2-A	Implementation of Part 20.1406	11.2.2.3
11.4-1-A	Mobile System Regulatory Guide Compliance	11.4.2.3
11.4-2-A	Compliance with IE Bulletin 80-10	11.4.2.3
11.4-3-A	Process Control Program	11.4.2.3
11.4-4-A	Temporary Storage Facility	11.4.1
11.4-5-A	Compliance with Part 20.1406	11.4.1
11.5-1-A	Subsystem Lower Limit of Detection	11.5.4.7
11.5-2-A	Offsite Dose Calculation Manual	11.5.4.4, 11.5.4.5, 11.5.5.8
11.5-3-A	Process and Effluent Monitoring Program	11.5, 11.5.4.6, and Table 11.5-201
11.5-4-A	Site Specific Offsite Dose Calculation	11.5.4.8
11.5-5-A	Instrument Sensitivities	11.5.4.9
12.1-1-A	Regulatory Guide 8.10	12BB
12.1-2-A	Regulatory Guide 1.8	12BB
12.1-3-A	Operational Considerations	12BB
12.1-4-A	Regulatory Guide 8.8	12BB
12.2-2-A	Airborne Effluents and Doses	12.2.2.1, 12.2.2.2, and Table 2.0-201
12.2-3-A	Liquid Effluents and Doses	12.2.2.4
12.3-2-A	Operational Considerations	12.3.4
12.3-3-A	Controlled Access	12.3.1.3
12.5-1-A	Equipment, Instrumentation, and Facilities	12BB
12.5-2-A	Compliance with Paragraph 50.34 (f)(2)(xxvii) of 10 CFR 50 and NUREG-0737 Item III.D.3.3	12BB
12.5-3-A	Radiation Protection Program	12BB
13.1-1-A	Organizational Structure	13.1.1 through 13.1.3
13.2-1-A	Reactor Operator Training	13.2.1

Table 1.10-201 (Sheet 5 of 5)
Summary of FSAR Sections Where DCD COL Items Are Addressed

Item No.	Subject/Description of Item	FSAR Section
13.2-2-A	Training for Non-Licensed Plant Staff	13.2.2
13.3-1-A	Identification of OSC and Communication Interfaces with Control Room and TSC	13.3, 13.3.2, COLA Part 5, Sections II.F and II.H
13.3-2-A	Identification of EOF and Communication Interfaces with Control Room and TSC	13.3, 13.3.2, COLA Part 5, Sections II.F and II.H
13.3-3-A	Decontamination Facilities	13.3, 13.3.2 COLA Part 5, Section II.J
13.4-1-A	Operation Programs	13.4
13.4-2-A	Implementation Milestones	13.4
13.5-1-A	Administrative Procedures Development Plan	13.5.1
13.5-2-A	Plant Operating Procedures Development Plan	13.5.2
13.5-3-A	Emergency Procedures Development	13.5.2
13.5-4-A	Implementation of the Plant Procedures Plan	13.5, 13.5.2
13.5-5-A	Procedures Included in Scope of Plan	13.5.2
13.5-6-H	Procedures for Calibration, Inspection and Testing	13.5.2
14.2-1-H	Startup Administration Manual	14.2.2.1
14.2-2-H	Approved Plant Pre-Operational and Startup Test Procedure	14.2.2.2
14.2-3-H	Detailed Testing Schedule	14.2.7
14.2-4-H	Approved Test Procedures for Site-Specific System	14.2.9
14.3-1-A	Emergency Planning Inspections, Tests, Analyses and Acceptance Criteria (ITAAC)	14.3.8 and COLA Part 10
14.3-2-A	Site-Specific ITAAC	14.3.9 and COLA Part 10
16.0-1	Replace Tech Spec Information in Brackets with Plant-Specific Information	16.0 and COLA Part 4
17.2-1-A	QA Program for the Construction and Operations Phases	17.2
17.2-2-A	QA Program for Design Activities	17.2
17.3-1-A	Quality Assurance Program Document	17.3
17.4-1-A	Operation Reliability Assurance Activities	17.4.1, 17.4.6, 17.4.9, 17.4.10, and 17.6
19.2.6-1-H	Seismic High Confidence Low Probability of Failure Margins	19.2.3.2.4

1.11 Technical Resolutions of Task Action Plan Items, New Generic Issues, New Generic Safety Issues and Chernobyl Issues

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

1.11.1 Approach

Add the following at the end of this section.

VCS COL 1.11-1-A Table 1.11-201 supplements DCD Table 1.11-1 to address the site-specific aspects of items that refer to Notes (2) and (7).

VCS SUP 1.11-1 Table 1.11-202 supplements DCD Table 1.11-1 to provide references to FSAR locations that provide additional information on specific issues.

1.11.2 COL Information

1.11-1-A Address Table 1.11-1 Items that refer to Notes (2) and (7)

VCS COL 1.11-1-A This COL item is addressed in Section 1.11 and Table 1.11-201.

VCS COL 1.11-1-A

Table 1.11-201
COL Item Resolutions Related to NUREG-0933 Table II Task Action Plan Items and New Generic Issues

Action Plan Item/Issue Number	Description	Associated Location(s) Where Discussed and/or Technical Resolution
	Task Action Plan Items	
A-33	NEPA Review of Accident Risks	This environmental issue involves consideration of accidents on a risk specific basis. This subject is addressed in COLA Part 3, Chapter 7.
B-1	Environmental Technical Specifications	Issue is addressed in COLA Part 4, Sections 5.5.1 and 5.5.3, which address the Offsite Dose Calculation Manual and Radioactive Effluent Controls Program. See also Sections 11.5.4.5 and 11.5.4.6.
B-28	Radionuclide/Sediment Transport Program	Issue is addressed in COLA Part 4, Sections 5.5.1 and 5.5.3, which address the Offsite Dose Calculation Manual and Radioactive Effluent Controls Program. See also Sections 11.5.4.5 and 11.5.4.6.
B-37	Chemical Discharges to Receiving Waters	Issue is addressed in COLA Part 3, Sections 3.3, 3.6, and 5.2.
B-38	Reconnaissance Level Investigations	Issue is addressed in COLA Part 3, Chapter 2, and FSAR Chapter 2.
B-39	Transmission Lines	Issue is addressed in COLA Part 3, Sections 3.7, 4.3, and 5.6.
B-40	Effects of Power Plant Entrainment on Plankton	Issue is addressed in COLA Part 3, Section 5.3.1.2.
B-41	Impacts on Fisheries	Impact of power plant operation on fishery resources is addressed in COLA Part 3, Section 5.3.
B-42	Socioeconomic Environmental Impacts	Issue is addressed in COLA Part 3, Sections 2.5, 4.4, and 5.8.
B-43	Value of Aerial Photographs for Site Evaluation	Work completed to date on this issue is published in NUREG/CR-2861. The use of aerial photography is discussed in Section 2.5. Results of a visual impact study are presented in COLA Part 3, Section 5.8.
C-16	Assessment of Agricultural Land in Relation to Power Plant Siting and Cooling System Selection	(3) The impact of construction and power plant operation on agricultural land use is addressed in COLA Part 3, Sections 4.1 and 5.1. Water use for agricultural lands is addressed in COLA Part 3, Sections 2.3.2 and 2.3.3.
	New Generic Issues	
184	Endangered Species	Issue is addressed in COLA Part 3, Sections 2.4.1, 2.4.2, 4.3, and 5.1.1.

VCS SUP 1.11-1

Table 1.11-202
Supplementary Resolutions Related to NUREG-0933 Table II TMI Action Plan Items and Human Factors Issues

Action Plan Item/Issue Number	Description	Associated Location(s) Where Discussed and/or Technical Resolution
	TMI Action Plan Items	
1.A.1.1	Shift Technical Advisor	Subsection 13.1.2.2 and DCD Section 18.6
1.A.1.2	Shift Supervisor Administrative Duties	Subsection 13.1.2.2
1.A.1.3	Shift Manning	Subsection 13.1.2.1, Table 13.1-202, Figure 13.1-202, and DCD Section 18.6
1.A.2.1(1)	Qualifications — Experience	Subsection 13.1.3, Table 13.1-201, Section 17.5, and DCD Section 18.6
1.C.3	Shift Supervisor Responsibilities	Subsection 13.1.2.2
1.F.2(6)	Increase the Size of Licensees' QA Staff	Table 13.1-201
1.F.2(9)	Clarify Organizational Reporting Levels for the QA Organization	23, Table 13.1-201, and Section 17.5
II.B.3	Post Accident Sampling	Subsection 9.3.2.2
III.D.3.3	In-Plant Radiation Monitoring	Appendix 12BB
	Human Factors Issues	
HF1.1	Shift Staffing	Table 13.1-202 and Subsection 13.1.2.1

VCS SUP 1.12-1

1.12 Impact of Unit 2 Construction on Unit 1

1.12.1 Introduction

Paragraph 10 CFR 52.79(a)(31) requires that the FSAR include the following information:

> For nuclear power plants to be operated on multi-unit sites, an evaluation of the potential hazards to the structures, systems, and components important to safety of operating units resulting from construction activities, as well as a description of the managerial and administrative controls to be used to provide assurance that the limiting conditions for operation are not exceeded as a result of construction activities at the multi-unit sites.

Accordingly, the evaluation of the potential impact of the construction of Unit 2 on Unit 1 important to safety structures, systems, and components (SSCs) is summarized below, along with a description of the managerial and administrative controls used to provide assurance that Unit 1 limiting conditions for operation are not exceeded as a result of Unit 2 construction activities. This evaluation involves several sequential steps:

- Identification of potential construction activity hazards
- Identification of SSCs important to safety
- Identification of limiting conditions for operation (LCOs)
- Identification of impacted SSCs and LCOs
- Identification of applicable managerial and administrative controls

1.12.2 Potential Construction Activity Hazards

Based on experience from similar projects, the scope of work necessary to construct Units 1 and 2 is well understood. In general, it includes, but is not necessarily limited to, activities such as site exploration, grading, clearing and installation of drainage and erosion control measures; boring, drilling, dredging, demolition and excavating; storage and warehousing of equipment; and construction, erection, and fabrication of new facilities. These activities involve major ESBWR standard plant structures such as the reactor building, control building, fuel building, turbine building, radioactive waste building, and electrical building; as well as related support facilities such as transformers, switchyard(s), transmission lines, cooling water structures and systems, water

treatment facilities, storage tanks, etc. Construction impacts on security controls are discussed in COLA Part 8.

The applicable time period for assessing such activities for their impact on Unit 1 starts when Unit 1 begins fuel load and ends for Unit 2 when construction activities are completed.

Each of the types of construction activities necessary to build a new unit were examined to identify the potential hazard. The resulting list of construction activities and potential hazards is shown in Table 1.12-201.

1.12.3 Structures, Systems and Components Important to Safety

Consistent with 10 CFR 50.34 and 10 CFR 50, Appendix A, Units 1 and 2 SSCs important to safety are identified in Chapter 3. Additional information on important-to-safety SSCs is provided in Chapters 4, 5, 6, 7, 8 and 9.

1.12.4 Limiting Conditions for Operation

Pursuant to 10 CFR 50.36, LCOs are the lowest functional capability or performance levels of equipment required for safe operation of a facility and are established in operating unit technical specifications for each item meeting one or more of the following criteria:

- Criterion 1 – Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.
- Criterion 2 – A process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
- Criterion 3 – An SSC that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
- Criterion 4 – An SSC that operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

The applicable LCOs are found in the Unit 1 Technical Specifications.

1.12.5 Impacted Structures, Systems, and Components and Limiting Conditions for Operation

The information described in Sections 1.12.2 through 1.12.4 was evaluated to identify Unit 1 SSCs and LCOs that might be impacted by Unit 2 construction activities. Potentially impacted SSCs, and/or SSCs with potentially impacted LCOs, are shown in Table 1.12-202, along with the associated hazards.

1.12.6 Managerial and Administrative Controls

Managerial and administrative controls are used to identify preventive and mitigative measures and provide notification of hazardous activity initiation in order to prevent or minimize exposure of SSCs to the identified hazards. Applicable managerial and administrative controls are listed in Table 1.12-203.

Specific hazards, impacted SSCs, and managerial and administrative controls will be developed and implemented as work progresses on site. In addition, these controls ensure that any associated LCOs specified in the Unit 1 Technical Specifications are not exceeded as a result of Unit 2 construction activities based on the following discussion.

Most of the SSCs important to safety are contained and protected within safety-related structures. The managerial and administrative controls established will protect these internal SSCs from construction hazards by maintaining the integrity and design basis of the safety-related structures and foundations. Heavy load drop controls, crane boom failure standoff requirements, groundwater depression monitoring, ground vibration controls, and construction-generated missile(s) control are examples of managerial and administrative controls that would provide this reasonable assurance.

The VCS units are physically separated such that approximately 1000 feet separate Unit 1 SSCs from the corresponding Unit 2 SSC. This separation provides additional assurance that work on Unit 2 SSCs will not impact Unit 1.

Other managerial and administrative controls will reasonably ensure that offsite power is not disrupted, hazardous materials and gasses are controlled, cooling water supplies are protected, instrumentation is protected from vibrations, and the SSCs are protected from site excavation issues. These managerial and administrative controls prevent or mitigate unnecessary challenges to safety systems caused by plant

construction hazards, such as disruption of offsite transmission lines or impact to plant cooling water supplies.

Lessons learned from industry experience with working near an operating unit will be factored into these managerial and administrative controls.

VCS SUP 1.12-1

Table 1.12-201 (Sheet 1 of 2)
Potential Hazards to Unit 1 from Unit 2 Construction Activities

Construction Activity	Potential Hazards
Site Exploration, Grading, Clearing, Installation of Drainage and Erosion Control Measures, etc.	Impact on Overhead Power Lines
	Impact on Transmission Towers
	Impact on Underground Conduits, Piping, Tunnels, etc.
	Impact on Site Access and Egress
	Impact on Drainage Facilities and Structures
	Impact on Onsite Transportation Routes
	Impact on Slope Stability
	Impact of Increased Soil Erosion and Local Flooding
	Impact of Construction-Generated Dust and Equipment Exhausts
	Impact of Encroachment on Plant Protected or Vital Areas
	Impact of Encroachment on Structures and Facilities
Boring, Drilling, Pile Driving, Dredging, Demolition, Excavation, etc.	Impact on Underground Conduits, Piping, Tunnels, etc.
	Impact on Foundation Integrity
	Impact on Structural Integrity
	Impact on Slope Stability
	Impact of Ground Vibration
Equipment Movement, Material Delivery, Vehicle Traffic, etc.	Impact on Overhead Power Lines
	Impact on Transmission Towers
	Impact on Underground Conduits, Piping, Tunnels, etc.
	Impact of Crane Load Drops
	Impact of Crane or Crane Boom Failures
	Impact of Vehicle Accidents
	Impact of Vehicle Runaways
Equipment and Material Laydown, Storage, Warehousing, etc.	Impact of Releases of Stored Flammable, Hazardous or Toxic Materials
	Impact of Increased Local Flooding
	Impact of Wind-Generated, Construction-Related Debris and Missiles
General Construction, Erection, Fabrication, etc.	Impact on Instrumentation and Control Systems and Components
	Impact on Electrical Systems and Components
	Impact on Cooling Water Systems and Components
	Impact on Radioactive Waste Release Points and Parameters
Connection, Integration, Tie-In, Testing, etc.	Impact on Instrumentation and Control Systems and Components
	Impact on Electrical and Power Systems and Components
	Impact on Cooling Water Systems and Components

Table 1.12-201 (Sheet 2 of 2)
Potential Hazards to Unit 1 from Unit 2 Construction Activities

Construction Activity	Potential Hazards
General Site Construction Activities	Impact on Site Security Systems

VCS SUP 1.12-1

Table 1.12-202 (Sheet 1 of 2)
Unit 1 Systems, Structures and Components Potentially Impacted by Unit 2 Construction Activities, and Associated Hazards

Potential Hazard	Potential Concern
CONTAINMENT STRUCTURE	
Impact of Crane or Crane Boom Failures	Building Degradation Due to Crane Boom Failure
Impact of Wind-Generated Construction-Related Debris and Missiles	Effects of Construction-Related Debris or Missiles
CONTROL ROOM EMERGENCY HVAC SYSTEMS	
Impact of Construction-Generated Dust and Equipment Exhausts	Effects of Construction-Generated Dust and Equipment Exhausts on Control Room Habitability Systems Air Intakes
Impact of Releases of Flammable, Hazardous or Toxic Materials	Effects of Releases of Flammable, Hazardous or Toxic Materials on Control Room Habitability Systems Design Basis
Impact of Vehicle Accidents	Effects of Releases of Flammable, Hazardous or Toxic Materials on Control Room Habitability Systems Design Basis
FIRE PROTECTION SYSTEM	
Impact on Underground Conduits, Piping, Tunnels, etc.	Degradation of Fire Protection System Availability or Capacity
FUEL BUILDING	
Impact of Wind-Generated Construction-Related Debris and Missiles	Effects of Construction-Related Debris or Missiles
GASEOUS RADIOACTIVE WASTE MANAGEMENT SYSTEM	
Impact on Radioactive Waste Release Points and Parameters	Building and Facility Effects on Gaseous Release X/Q and D/Q Assumptions
ONSITE POWER SYSTEMS	
Impact of Ground Vibration	Operability Disruptions Due to Vibration Induced Spurious Trips
Impact on Electrical Systems and Components	Operability Disruptions Due to Vibration Induced Spurious Trips, System Interconnections, etc.
OFFSITE POWER SYSTEM	
Impact on Overhead Power Lines	Transmission Line Disruptions Due to Grading or Clearing, Equipment Movement, Crane Boom Failures, etc.
Impact on Transmission Towers	Transmission Line Disruptions Due to Grading or Clearing, Equipment Movement, Crane Boom Failures, etc.
Impact of Ground Vibration	Operability Disruptions Due to Vibration Induced Spurious Trips

Table 1.12-202 (Sheet 2 of 2)
Unit 1 Systems, Structures and Components Potentially Impacted by Unit 2 Construction Activities, and Associated Hazards

Potential Hazard	Potential Concern
Impact on Electrical Systems and Components	Operability Disruptions Due to Equipment Movement, System Interconnections, etc.
SERVICE BUILDING	
Impact of Crane or Crane Boom Failures	Building Degradation Due to Crane Boom Failure
Impact of Wind-Generated, Construction-Related Debris and Missiles	Construction-Related Debris or Missile
SERVICE WATER SYSTEM	
Impact on Underground Conduits, Piping, Tunnels, etc.	Degradation of Service Water System Availability or Capacity
Impact on Cooling Water Systems and Structures	Degradation of Service Water System Availability or Capacity
SITE SECURITY SYSTEMS	
Impact on Site Security Systems	Degradation of Site Security System Capabilities

VCS SUP 1.12-1

Table 1.12-203 (Sheet 1 of 2)
Managerial and Administrative Controls for Unit 2 Construction Activity Hazards

Hazard	Control
Impact on Overhead Power Lines	Managerial and Administrative Controls for Appropriate Standoff and/or Installation of Temporary Warning or Caution Barriers and Signage
Impact on Transmission Towers	Managerial and Administrative Controls for Appropriate Standoff and/or Installation of Temporary Warning or Caution Barriers and Signage
Impact on Underground Conduits, Piping, Tunnels, etc.	Managerial and Administrative Controls to Identify Potentially Affected SSCs; Evaluation to Ensure Structural Integrity During Construction; and/or Temporary Measures to Mitigate Impacts
Impact of Construction-Generated Dust and Equipment Exhausts	Managerial and Administrative Controls to Avoid or Minimize Construction Dust (For Example, Use of Water Spray Trucks, Speed Limits) and/or Enhanced Monitoring of Potentially Affected System Intakes, Filters, etc.
Impact of Vehicle Accidents	Managerial and Administrative Controls to Respond to Site Accidents (For Example, Construction Fire Brigade and/or Hazardous Materials Response Team)
Impact of Ground Vibration	Managerial and Administrative Controls to Identify Potentially Affected SSCs, and/or Temporary Measures to Mitigate Impacts
Impact of Crane or Crane Boom Failures	Managerial and Administrative Controls for Appropriate Standoff and/or Load Limits (For Example, Minimum Standoff Distances and/or Load Limitations), Drop Analysis if Minimum Distance or Load Limits Are Not Practical
Impact of Releases of Flammable, Hazardous or Toxic Materials	Managerial and Administrative Controls on Quantities and Types of Flammable, Hazardous or Toxic Materials
Impact of Wind-Generated, Construction-Related Debris and Missiles	Managerial and Administrative Controls on Equipment and Material Storage and Transport, and for Reducing Power or Shutting Down Unit 1 During High Winds or High Wind Warnings
Impact on Electrical Systems and Components	Managerial and Administrative Controls to Identify Potentially Affected SSCs; Evaluation to Ensure System and Component Integrity During Construction; and/or Temporary Measures to Mitigate Impacts
Impact on Cooling Water Systems and Components	Managerial and Administrative Controls to Identify Potentially Affected SSCs; Evaluation/Monitoring to Ensure System and Component Integrity/Water Quality During Construction; and/or Temporary Measures to Mitigate Impacts

Table 1.12-203 (Sheet 2 of 2)
Managerial and Administrative Controls
for Unit 2 Construction Activity Hazards

Hazard	Control
Impact on Radioactive Waste Release Points and Parameters	Enhanced Monitoring and Control to Assure Releases Are Within Limits
Impact on Site Security Systems	Managerial and Administrative Controls to Coordinate Construction Activities with Unit 1 Physical Protection Personnel and Procedures

Appendix 1A Response to TMI Related Matters

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

Table 1A-1, 10 CFR 50.34(f)(3)(i), TMI Item I.C.5

Add the following to the end of the ESBWR Resolution statement:

STD SUP 1A.1-1

ESBWR construction and operations engineers are also continually involved in reviewing industry experience from these same sources in accordance with the administrative procedures described in DCD Section 18.3.2.

Table 1A-1, 10 CFR 50.34(f)(3)(iii), TMI Item I.F.2

Add the following to the end of the ESBWR Resolution statement:

STD SUP 1A.1-1

The Quality Assurance Program described in Chapter 17 also meets the requirements of issue I.F.2 as they apply to the construction and operation of the ESBWR.

Table 1A-1, 10 CFR 50.34(f)(3)(vii), TMI Item II.J.3.1

Add “13.1” as an “Associated Location(s)” and add the following to the end of the ESBWR Resolution statement:

STD SUP 1A.1-1

The ESBWR construction and operations teams have also developed a management plan for the ESBWR project that consists of a properly structured organization with open lines of communication, clearly defined responsibilities, well-coordinated technical efforts, and appropriate control channels.

The organizational structure is discussed in Section 13.1.

Appendix 1B Plant Shielding to Provide Access to Areas and Protect Safety Equipment for Post-Accident Operation [II.B.2]

This section of the referenced DCD is incorporated by reference with no departures or supplements.

Appendix 1C Industry Operating Experience

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

Appendix 1C.1 Evaluation

Replace the last paragraph with the following.

STD COL 1C.1-1-A
STD COL 1C.1-2-A
STD SUP 1C-1

Tables 1C-201 and 1C-202. These tables address Generic Letters and Bulletins that have been in effect/issued up to six months before the COL application submittal date, and after the SRP revisions that are applicable to this FSAR. They also address Generic Letter 82-39 and IE Bulletin 2005-02, which were identified in the DCD as the responsibility of the COL applicant.

Appendix 1C.2 COL Information

1C.1-1-A Handling of Safeguards Information

STD COL 1C.1-1-A

This COL item is addressed in Section 1C.1 and the Table 1C-201 entry for Generic Letter 82-39.

1C.1-2-A Emergency Preparedness and Response Actions

STD COL 1C.1-2-A

This COL item is addressed in Section 1C.1 and the Table 1C-202 entry for IE Bulletin 2005-02.

STD COL 1C.1-1-A

Table 1C-201
Operating Experience Review Results Summary—Generic Letters

No.	Issue Date	Title	Evaluation Result or Location(s) Where Discussed
82-39	12/22/82	Problems with the Submittals of 10 CFR 73.21 Safeguards Information Licensing Review	Not Applicable. Is an administrative communication. The site has an approved procedure for handling Safeguards Information including how to mail such information to authorized recipients.

STD COL 1C.1-2-A

Table 1C-202
Operating Experience Review Results Summary—IE Bulletins

No.	Issue Date	Title	Evaluation Result or Location(s) Where Discussed
2005-02	07/18/05	Emergency Preparedness and Response Actions for Security-Based Events	COLA Part 5, Emergency Plan