

RAS M-308

Sample ID 028

IV REACTOR VESSEL, INTERNALS, AND REACTOR COOLANT SYSTEM							
B1 Reactor Vessel Internals (BWR)							
Item	Link	Structure and/or Component	Material	Environment	Aging Effect/ Mechanism	Aging Management Program (AMP)	Further Evaluation
IV.B1-13 (R-100)	IV.B1.4-a	Jet pump assemblies Thermal sleeve Inlet header Riser brace arm Holddown beams Inlet elbow Mixing assembly Diffuser Castings	Nickel alloy; stainless steel	Reactor coolant	Cracking/ stress corrosion cracking, intergranular stress corrosion cracking, irradiation-assisted stress corrosion cracking	Chapter XI.M9, "BWR Vessel Internals," for jet pump assembly and Chapter XI.M2, "Water Chemistry," for BWR water	No
IV.B1-14 (R-53)	IV.B1.6-b IV.B1.3-b IV.B1.4-b IV.B1.2-b IV.B1.5-b IV.B1.1-c	Reactor vessel internals components	Stainless steel; nickel alloy	Reactor coolant	Cumulative fatigue damage/ fatigue	Fatigue is a time-limited aging analysis (TLAA) to be evaluated for the period of extended operation. See the Standard Review Plan, Section 4.3 "Metal Fatigue," for acceptable methods for meeting the requirements of 10 CFR 54.21(c)(1).	Yes, TLAA
IV.B1-15 (RP-26)	IV.B1.	Reactor vessel internals components	Stainless steel; nickel alloy	Reactor coolant	Loss of material/ pitting and crevice corrosion	Chapter XI.M1, "ASME Section XI Inservice Inspection, Subsections IWB, IWC, and IWD," for Class 1 components and Chapter XI.M2, "Water Chemistry" for BWR water	No
IV.B1-16 (RP-18)	IV.B1.	Steam Dryers	Stainless steel	Reactor coolant	Cracking/ flow-induced vibration	A plant-specific aging management program is to be evaluated.	Yes, plant-specific

NUREG-1801, Rev. 1

IV B1-6

September 2005

DS-03

~~U.S. NUCLEAR REGULATORY COMMISSION~~

In the Matter of Energy Nuclear Vermont Yankee LLC

Docket No. 50-271 Official Exhibit No. E3-08-V4

OFFERED by: Applicant/Licensee Intervenor _____

NRC Staff Other _____

IDENTIFIED on 7/23/08 Witness/Panel NEC3

Action Taken: ADMITTED REJECTED WITHDRAWN

Reporter/Clerk MAC