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3.0 PLANT DESCRIPTION

3.1 EXTERNAL APPEARANCE AND PLANT LAYOUT

{The site for Callaway Plant Unit 2 is the 7,354 acre (2,976 hectares) Callaway Plant Unit 1 (AmerenUE) property located in Callaway County approximately 10 miles (16 km) southeast of Fulton, Missouri, and 80 miles (130 km) west of the St. Louis metropolitan area. The Missouri River flows by the site in an easterly direction approximately 5 miles (8 km) south of the site at its closest point. The terrain is gently rolling upland, once part of an old glacial till plain. Plant site topography is found in [Figure 2.2-3](#). Ground surface elevations range from 858 ft (262 m) at the crest of the plateau to approximately 520 ft (158 m) at the Missouri River}, with an average elevation of approximately 689 ft (210 m). Erosion and downcutting of the Missouri River and its tributary streams have dissected the plain, leaving a nearly isolated plateau of approximately 8 sq miles (21 sq km). References to elevation values in this section are based on National Geodetic Vertical Datum of 1929 (NGVD, 29), unless otherwise stated.

The AmerenUE property contains one existing pressurized water reactor (PWR) designated as Callaway Plant Unit 1. Existing plant structures occupy approximately 160 acres (65 hectares). In this and the paragraphs to follow, the orientation of the buildings is given with reference to both plant and geographical directions. The Turbine Building is located plant north (geographically southwest) of the Reactor Building. The Service Building is located plant northeast (geographically west) of the Turbine Building. Access to the existing plant is via an onsite road which intersects State Route CC west of the site.}

The plant will be a U.S. Evolutionary Power Reactor (EPR), referred to as {Callaway Plant Unit 2}. The U.S. EPR is a pressurized water reactor design with a rated core thermal power of 4,590 MWt. The rated and design gross electrical output for the EPR is approximately 1,710 MWe. {Electrical power consumption for auxiliary loads resulted in a rated and design net electrical output of approximately 1,600 MWe.} The plant will be constructed {plant east (geographically northwest) of the existing Callaway Plant Unit 1}. Construction related and new plant structures will occupy approximately {195 acres (79 hectares)}. Including the permanent laydown areas, widened transmission corridor, and Collector Well River Intake System (Collector Well System), a total of 647 acres (262 hectares) will be occupied by new plant structures and supporting infrastructure. The Callaway Plant Unit 2 reactor will be separated from the Callaway Plant Unit 1 reactor by a distance of approximately 1,350 ft (411 m).}

The {Callaway Plant Unit 2} design is a four-loop, pressurized water reactor, with a Reactor Coolant System composed of a reactor pressure vessel that contains the fuel assemblies, a pressurizer including ancillary systems to maintain system pressure, one reactor coolant pump per loop, one steam generator per loop, associated piping, and related control systems and protection systems. The {Callaway Plant Unit 2} Reactor Building and Turbine Building will be oriented side by side, with the Reactor Building oriented towards the {plant south (geographic northeast) of the Turbine Building}.

The Reactor Building will be surrounded by the Fuel Building, four Safeguard Buildings, two Emergency Power Generating Buildings, the Nuclear Auxiliary Building, the Radioactive Waste Processing Building and the Access Building. [Figure 3.1-1](#) shows the layout for {Callaway Plant Unit 2}, depicting the following features: site exclusion area boundary (EAB), site boundary, liquid and gaseous release points (i.e., {Missouri River outfall, Unit 2 vent stack, and cooling towers}, respectively) and their elevations and distances from the Reactor Building, meteorological towers, and other buildings and structures.

The {Callaway Plant Unit 1} Reactor Building is a vertical cylinder, concrete structure with a dome and flat base, measuring approximately {150 ft (45.7 m)} in diameter with an overall height of about {245 ft (74.7 m)}. With the existing plant grade at an elevation of approximately {840 ft (256 m)} and the top of the Reactor Building at an elevation of approximately {1,039 ft (317 m)}, the Reactor Building rises about {199 ft (61 m)} above grade. Other existing plant buildings are {either concrete or steel with exterior metal siding}.

The {Callaway Plant Unit 2} Reactor Building is an upright cylinder concrete structure, capped with a spherical dome. The Reactor Building is 186 ft (56.7 m) in diameter with an overall height of {240 ft (73.2 m)} from the bottom of the Reactor Building foundation. The plant grade for {Callaway Plant Unit 2} will be at an elevation of approximately {845 ft (258 m)}. With the bottom of the Reactor Building foundation {36 ft (11 m)} below grade, the new Reactor Building will rise {204 ft (62 m)} above grade. The top of the Reactor Building will be at an elevation of approximately {1,049 ft (320 m)}.

{The cooling towers for Callaway Plant Unit 2 will be the tallest new structures at approximately 550 ft (168 m) above grade. Callaway Plant Unit 2 will have a closed-loop cooling system. The Callaway Plant Unit 2 Cooling Towers will be round concrete structures approximately 414 ft (126 m) in diameter at the base and 550 ft (168 m) tall. Similar to Callaway Plant Unit 1, other Callaway Plant Unit 2 buildings will be concrete or steel with metal siding. Figure 3.1-2 depicts an aerial view of Callaway Plant Unit 1 with the Callaway Plant Unit 2 superimposed on the photograph.}

{The Callaway Plant Unit 2 Ultimate Heat Sink (UHS) function will be provided by mechanical forced draft Essential Service Water System (ESWS) cooling towers situated above storage basins. Each of the four basins will be approximately 0.19 acres (0.08 hectares) in size. The basins will normally be supplied with treated makeup water from the non-safety-related Collector Well System located on the north bank of the Missouri River.}

In the event of a design basis accident, the basins will be supplied with water from the safety-related Essential Service Water Emergency Makeup System (ESWEMS) Pond. The ESWS cooling towers will be 96 ft (29 m) tall.}

{Figure 3.1-3} through {Figure 3.1-6} are ground-level photographs of the {Callaway Plant} site taken from adjacent properties generally to the {north, south, west and east}. Major structures associated with {Callaway Plant Unit 2} have been superimposed on these photographs to depict potential visual impacts as viewed from {the intersection of State Route D and State Route O to the northeast, the Callaway R-II schools on State Route C to the southwest, from the Callaway Plant entrance on State Route CC to the west and from the intersection of State Route CC and State Route O to the northwest}.

{Due to heavily forested onsite areas, screening is provided by trees so that only the tops of the taller structures may be visible from adjacent properties at ground level. Many of the Callaway Plant Unit 2 buildings will not be visible since the taller structures will mask the lower rise structures. Due to onsite elevation changes, topographical features such as hills and valleys will also help to screen and seclude plant structures from surrounding properties even when foliage is seasonally absent. In addition, since Callaway Plant Unit 2 will be located approximately 6,300 ft (1,920 m) from the nearest residential property, distance will help shield the plant from view.}

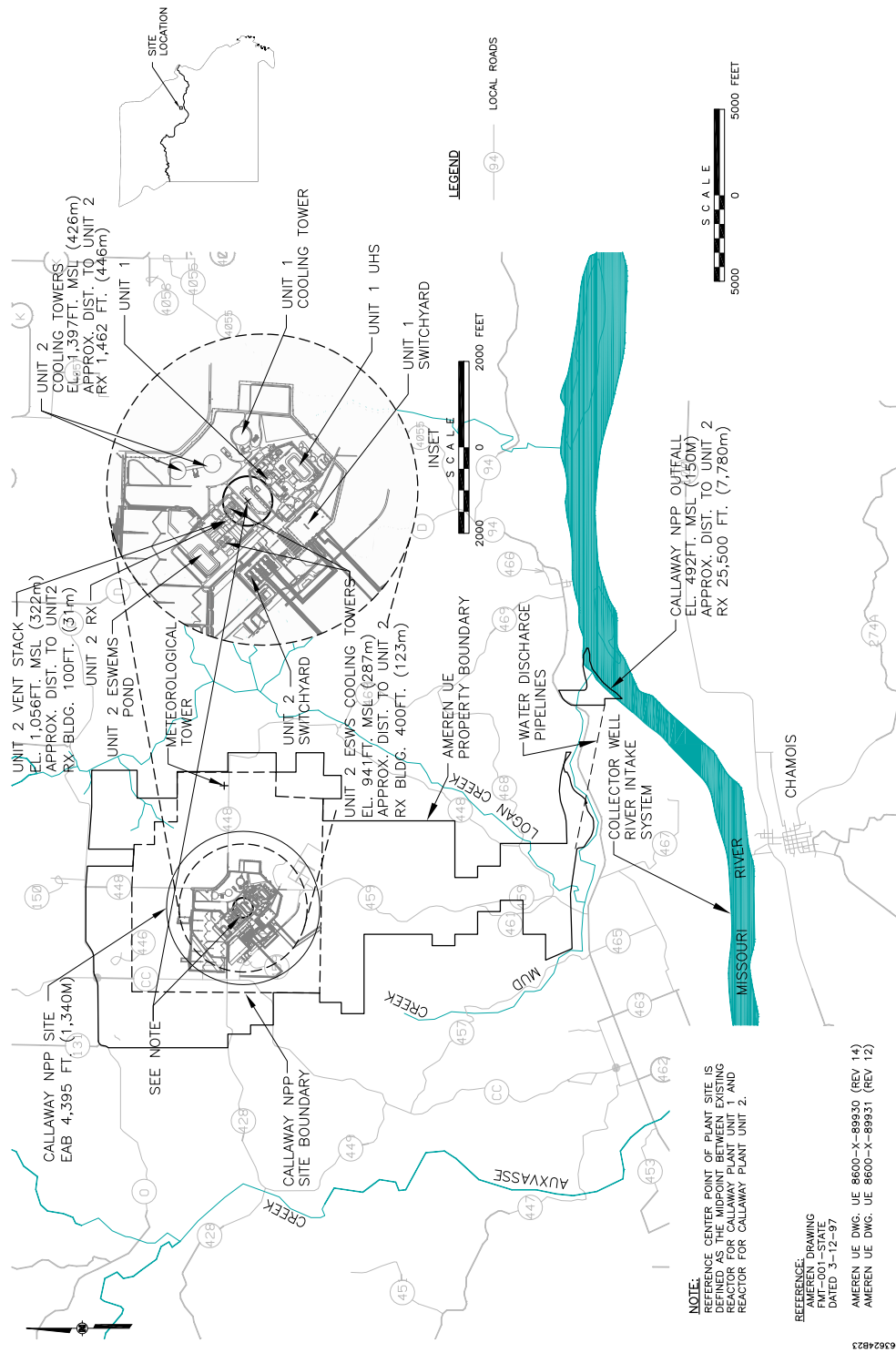
Callaway Plant Unit 2 will share Callaway Plant Unit 1 Collector Well System, and associated discharge piping at the shoreline and, therefore, there will be no additional visual impact. No other visual impacts from nearby ground level vantage points are expected.}

Aesthetic principles and concepts used in the design and layout of {Callaway Plant Unit 2} include the following:

- ◆ {Preserving most woodlands on site.
- ◆ Constructing buildings similar in shape, size and material to existing buildings.
- ◆ Minimizing tree removal by locating the concrete batch plant, construction lay-down areas, parking areas and construction offices and warehouses in either cleared fields or lightly forested areas.
- ◆ Transporting excavated material to an onsite disposal area outside designated wetlands.
- ◆ Upgrading existing onsite roads as applicable to minimize the addition of new roads. However, new roads will provide direct routes to Callaway Plant Unit 2 for construction and will minimize disruption of Callaway Plant Unit 1 traffic patterns.

In addition to the above, exterior finishes for plant buildings will be similar in color and texture to those of the Callaway Plant Unit 1 buildings. This provides for a consistent, overall appearance, by architecturally integrating the buildings on the Callaway Plant site.} Areas that are cleared supporting construction activities will be either maintained or restored by reseeding and replanting with native trees and vegetation, so that the {Callaway Plant Unit 2} landscape blends with the {Callaway Plant Unit 1} landscapes and the remaining undisturbed areas on the {Callaway Plant} site.

Figure 3.1-1—{Callaway Plant Site Area Map}



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Figure 3.1-2—{Aerial View Looking Northwest with Callaway Plant Unit 2 Superimposed}



Figure 3.1-3—{Ground Level View Looking Southeast with Callaway Plant Unit 2 Structures Superimposed}



Figure 3.1-4—{Ground Level View Looking East from Nearest School with Callaway Plant Unit 2 Structures Superimposed}

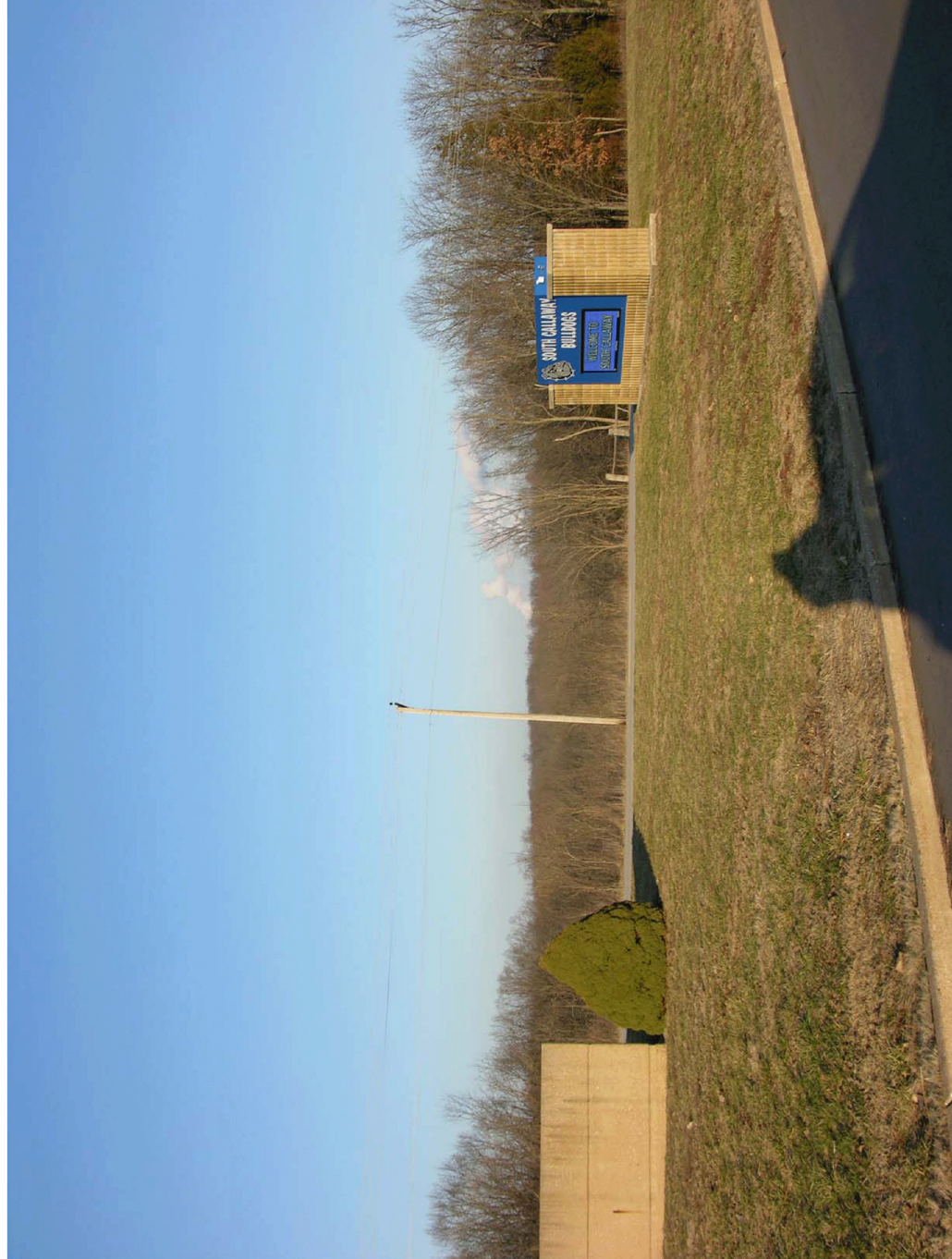


Figure 3.1-5—{Ground Level View Looking East with Callaway Plant Unit 2 Structures Superimposed}



Figure 3.1-6—{Ground Level View Looking Southwest from Intersection of Routes D & O with Callaway Plant Unit 2 Structures Superimposed}



3.2 REACTOR POWER CONVERSION SYSTEM

3.2.1 GENERAL

{AmerenUE will construct and operate a new nuclear power plant to be designated (Callaway Plant Unit 2) located on the existing Callaway Plant site in Callaway County, Missouri. AmerenUE is applying for a Combined Operating License for the nuclear power plant.}

The U.S. EPR design has a rated core thermal power of 4,590 MWt. The rated and design gross electrical output for the U.S. EPR is approximately 1,710 MWe. {Electrical power consumption for auxiliary loads results in a rated and design net electrical output of approximately 1,600 MWe.} Although the U.S. EPR is to be licensed for 40 years, the proposed operating life of the U.S. EPR is 60 years.

The U.S. EPR design is a four-loop, pressurized water reactor, with a Reactor Coolant System (RCS) composed of a reactor pressure vessel that contains the fuel assemblies, a pressurizer including ancillary systems to maintain system pressure, one reactor coolant pump per loop, one steam generator per loop, associated piping, and related control systems and protection systems. Referring to [Figure 3.2-1](#), which provides a simplified depiction of the reactor power conversion system for the U.S. EPR, the RCS transfers the heat generated in the reactor core to the steam generators where steam is produced to drive the turbine generator. Water is utilized to remove the heat formed inside the reactor core. The reactor coolant pumps provide forced circulation of water through the RCS and a pressurizer, connected to one of the four loops, maintains the pressure within a specified range. Each of the four reactor coolant loops comprises a hot leg from the reactor pressure vessel to a steam generator, a cross-over leg from the steam generator to a reactor coolant pump, and a cold leg from the reactor coolant pump to the reactor pressure vessel. In each of the four loops, the primary water leaves the reactor pressure vessel through an outlet nozzle and then goes to a steam generator. The primary water flows inside the steam generator tube bundle and transfers heat to the secondary water. The primary water then goes to a reactor coolant pump before returning to the reactor pressure vessel through an inlet nozzle. The feedwater entering the secondary side of the steam generators absorbs the heat transferred from the primary side and evaporates to produce saturated steam. The steam is dried in the steam generators then routed to the turbine to drive it. The steam is then condensed and returns as feedwater to the steam generators. The alternating current, synchronous type generator, driven by the turbine, generates electricity. The generator rotor will be hydrogen cooled and the generator stator will be cooled with water.

The U.S. EPR reactor core consists of 241 fuel assemblies. The fuel assembly structure supports the fuel rod bundles. Inside the assembly, the fuel rods are vertically arranged according to a square lattice with a 17x17 array. There are 265 fuel rods per assembly with the remaining locations used for control rods or instrumentation. The fuel rods are composed of enriched uranium dioxide sintered pellets contained in a cladding tube made of M5 advanced zirconium alloy. Percentage of uranium enrichment and total quantities of uranium for the U.S. EPR core are as follows:

- ◆ Cycle 1 (initial) – average batch enrichment is between 2.23 to 3.14 weight percent U-235 and 2.66 weight percent U-235 for core reload with an enriched uranium weight of 285,483 pounds (129,493 kg).
- ◆ Cycle 2 (transition) – average batch enrichment is between 4.04 to 4.11 weight percent U-235 and 4.07 weight percent U-235 for core reload with an enriched uranium weight of 141,909 pounds (64,369 kg).

- ◆ Cycle 3 (transition) – average batch enrichment is between 4.22 to 4.62 weight percent U-235 and 4.34 weight percent U-235 for core reload with an enriched uranium weight of 113,395 pounds (51,435 kg).
- ◆ Cycle 4 (equilibrium) – average batch enrichment is between 4.05 to 4.58 weight percent U-235 and 4.30 weight percent U-235 for core reload with an enriched uranium weight of 113,417 pounds (51,445 kg).

Average batch enrichment is the average enrichment for each fuel assembly comprising a batch of fuel. The enrichment for core reload is the average enrichment for all fuel assemblies loaded in the core which is derived from the mass weighted average for the batches of fuel. The above values are 'beginning of life' enrichment values. Discharged enrichment values will be less at the 'end of life' of the assembly. Assembly enrichment reduction is directly proportional to the assembly burnup.

Discharge burnups for equilibrium cores are approximately between 45,000 MWd/MTU to 59,000 MWd/MTU. The batch average discharge burnups for equilibrium cores is about 52,000 MWd/MTU.

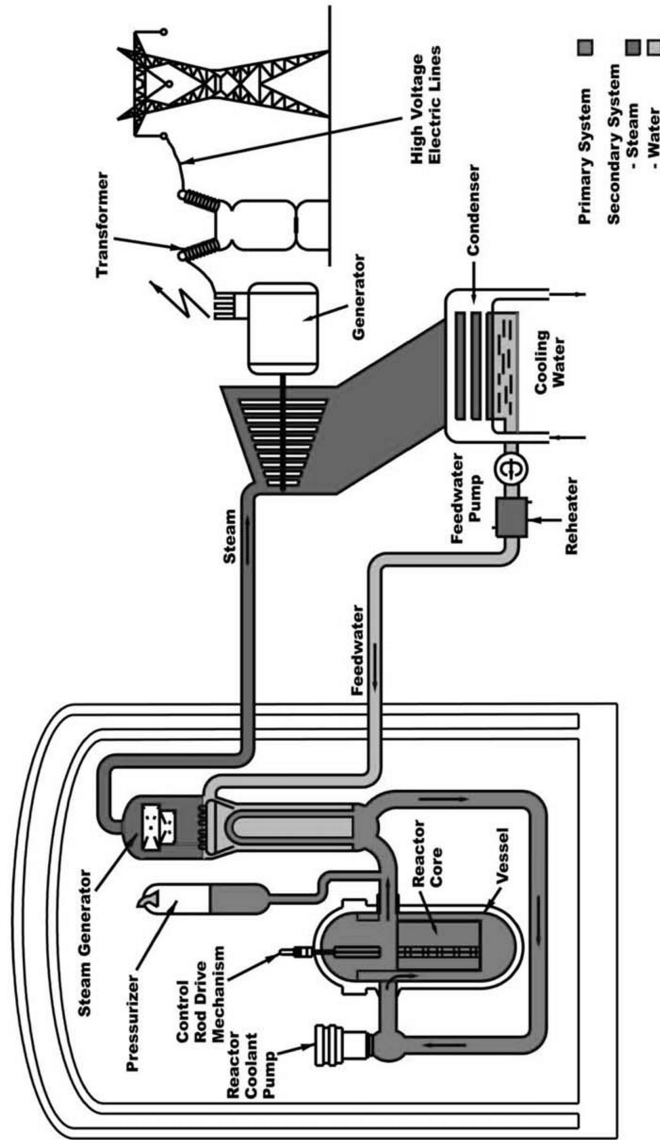
Engineered safety features for the U.S. EPR are designed to directly mitigate the consequences of a design basis accident (DBA) and include the following systems and functions:

- ◆ Containment - provided to contain radioactivity following a loss of coolant accident (LOCA).
- ◆ Containment heat removal - associated with the removal of energy from the containment after a DBA.
- ◆ Containment isolation and leakage testing - provided to minimize leakage from the containment.
- ◆ Combustible gas control – configured to reduce hydrogen concentrations in order to maintain containment integrity during and immediately following a DBA LOCA.
- ◆ Safety injection – designed to provide the emergency core cooling function.
- ◆ Control room habitability - designed so that control room occupants can remain in the control room to operate the plant safely under normal and accident conditions.
- ◆ Fission product removal and control systems – configured to reduce or limit the release of fission products following a postulated DBA, severe accident or fuel handling accident.
- ◆ Emergency heating, ventilation and air conditioning and filtration – provided to reduce radioiodine released as assumed during design basis events.
- ◆ Emergency feedwater – designed to supply water to the steam generators following the loss of normal feedwater supplies.
- ◆ Control of pH – associated with the control of pH in the containment following a DBA.

The U.S. EPR utilizes a standard nuclear steam turbine arrangement consisting of a tandem compound, six- flow steam turbine, operating at 1,800 revolutions per minute. The generator is an alternating current, synchronous type, with a hydrogen cooled rotor and water cooled stator. The main condenser condenses the steam exhausted from the three low pressure turbine elements, and is a multipressure, three-shell unit with {stainless steel} tubes and tubesheet overlay. The condenser heat transfer area for all three shells is estimated to be approximately 1.6 million sq ft (149 thousand sq m).

The operational back pressure range at guaranteed performance (100% load) is based on the condenser operating at 3.20 inches HgA (108.36 mbar), 2.44 inches HgA (82.63 mbar) and 1.85 HgA (62.65 mbar) in the high pressure, intermediate pressure and low pressure condenser shells, respectively. For 100% unit load, at the average plant back pressure of 2.5 inches HgA (84.7 mbar), the anticipated turbine heat rate is approximately 9,200 BTU/kwhr (2,318 kcal/kwhr). Circulating water for {Callaway Plant Unit 2 is cooled by two closed-loop, natural draft cooling towers}.

Figure 3.2.1—Reactor Power Conversion System



3.3 PLANT WATER USE

{Callaway Plant Unit 2} requires water for cooling and operational uses. Sources for water include the {Missouri River/Missouri River Alluvial Aquifer (Aquifer) provided via collector wells. The Aquifer is largely replenished by water from the Missouri River.} Water from the {Aquifer} provides makeup water for plant cooling. {Deep wells} supply makeup water for power plant operations. Figure 3.3-1 and Table 3.3-1 quantitatively illustrate the average and maximum water flows to and from various plant systems for normal plant operating conditions and normal shutdown/cooldown conditions, respectively. Flow rates for other plant modes are not applicable since there is no change in demand during startup or refueling operating conditions. The average flows represent continuous plant water usage requirements whereas the maximum flows represent intermittent demands. Water use by non-plant facilities includes potable and sanitary needs for administrative buildings and warehouses, and water required for landscaping maintenance. Potable water demand is based on projected staffing during normal plant operation. Other station water users, as noted above, have not been included in the estimated demand.

3.3.1 WATER CONSUMPTION

Primary water consumption is for turbine condenser cooling. {Cooling water for the turbine condenser and closed cooling heat exchanger for normal plant operating conditions is provided by the Circulating Water Supply System (CWS), which is a non-safety-related interface system. Makeup water to the CWS cooling towers is required due to evaporation and blowdown. The average makeup rate to the CWS is 22,283 gpm (84,340 lpm) which corresponds to 4.8 cycles of concentration. The maximum makeup rate to the CWS is 29,886 gpm (113,118 lpm) which corresponds to 3 cycles of concentration. The CWS discharges the heated water from the condenser to the CWS cooling towers. For the closed-loop CWS cooling tower, approximately 80% of the makeup water will be lost to the atmosphere as evaporation and to cooling tower drift. The other 20% will be released as blowdown. Therefore, the average consumptive use of Aquifer water during normal operating conditions will be approximately $9.76 \text{ E}+08$ gallons per month ($3.69 \text{ E}+09$ liters per month). Water consumption for the CWS system is a function of inlet water chemistry, ambient temperature, and humidity. During normal operating conditions, the maximum flow of water required by the CWS will be 29,886 gpm (113,118 lpm). Water consumption during normal shutdown/cool down is not projected to be different than normal operation. The aquifer is largely replenished by water from the Missouri River.}

Mechanical draft cooling towers with water storage basins (i.e., one basin for each of the four trains) comprise the Ultimate Heat Sink (UHS) which functions to dissipate heat rejected from the Essential Service Water System (ESWS). The ESWS is vital for all phases of plant operation and is designed to provide cooling water during power operation and shutdown of the plant. Under normal operating and normal shutdown/cooldown conditions, the ESWS cooling tower water storage basins will be supplied with non-safety-related makeup water pumped from {the Aquifer} at an average rate of {1,882 gpm (7,124 lpm)}. The makeup water serves to replenish water losses due to cooling tower evaporation and drift at a rate of {940 gpm (3,558 lpm) and 2 gpm (8 lpm)}, respectively. The remaining water is released to the {Missouri River} as ESWS cooling tower blowdown at an average rate of {940 gpm (3,558 lpm)}. For normal operation, {Aquifer water} consumption for the ESWS System will average approximately { $8.24 \text{ E}+07$ gallons per month ($3.12 \text{ E}+08$ liters per month)}. During normal plant shutdown/cooldown, when all four trains of the ESWS are operating and assuming a maximum makeup flow rate of 941 gpm (3,562 lpm) for each ESWS cooling tower, the peak water demand will be 3,764 gpm (14,248 lpm). The maximum water flow will be provided by the {Aquifer} and from water stored

in the ESWS cooling tower storage basins. Peak water demand will only be for a short period of time.

The ESWS cooling towers are connected to the remainder of the ESWS through intake and discharge paths. The ESWS takes suction from the ESWS cooling tower basins and cools the Component Cooling Water System (CCWS) heat exchangers. The CCWS is a closed-loop cooling water system that in conjunction with the ESWS provides a means to cool the reactor core, removing heat generated from plant essential and non-essential components connected to the CCWS.

During a design basis accident, {the Essential Service Water Emergency Makeup System (ESWEMS) Pond} will provide safety-related makeup water for the ESWS cooling tower, for the UHS functions, at a consumptive rate of up to {300 gpm (1,136 lpm)} for each ESWS cooling tower (minimum of two) operating during an accident. However, since the consumptive rate for accidents is not associated with normal modes of plant operation, this rate is not shown on the water use diagram, [Figure 3.3-1](#) or on [Table 3.3-1](#).

Sustained {water} demand for power plant makeup is {103 gpm (390 lpm)} and includes water supplies for the Demineralized Water Distribution System, the Potable and Sanitary Water Distribution System and the Fire Water Distribution System. The Demineralized Water Distribution System produces and delivers demineralized water to the power plant for systems that need high quality, non-safety makeup water. Except for containment isolation, the Demineralized Water Distribution System interfaces are non-safety-related. Under normal system operation, water consumption by the Demineralized Water Distribution System is 80 gpm (303 lpm). During normal shutdown/cooldown conditions, water consumption is also anticipated to be approximately 80 gpm (303 lpm). During normal plant operation, the Potable and Sanitary Water Distribution System supplies consumers with pre-treated water (i.e., Drinking Water Supply) at an average rate of {20 gpm (76 lpm)}. Due to potential surges in demand, water consumption during normal shutdown/cooldown conditions is anticipated to be 36 gpm (136 lpm)}. The system provides water for human consumption and sanitary cleaning purposes, and can be used by other systems as a water source. The Potable and Sanitary Water Distribution System is not connected with any radioactive source or other system which may contain substances harmful to the health of personnel. Failures in the Potable and Sanitary Water Distribution System will have no consequences on plant operation or safety functions. Similarly, the Fire Water Distribution System is classified as a non-safety system. It is required to remain functional following a plant accident, to provide water to hose stations in areas containing safe shutdown equipment. Water consumed by the Fire Water Distribution System during normal conditions is required to maintain system availability. The maximum consumptive rate accounts for system actuation. During normal operation, water consumed by the Fire Water Distribution System is due to system leakage and periodic testing. The maximum consumptive rate is based on meeting the National Fire Protection Association (NFPA)'s requirements for replenishing fire protection water storage.

Miscellaneous low volume waste generated by {Callaway Plant Unit 2 is discharged at an average rate of 55 gpm (208 lpm)}. This equates to an average consumptive rate of {48 gpm (182 lpm)} for power plant makeup, or {2.10 E+06 gallons per month (7.96 E+06 liters per month)}. Water consumption should not vary during drought conditions. Maximum water flow required for power plant makeup during normal shutdown/cooldown conditions is {116 gpm (439 lpm)}.

{CWS cooling tower blowdown, ESWS cooling tower blowdown, miscellaneous treated low volume waste, and treated liquid radwaste are directed to the Missouri River. During plant

startup, startup flushes and chemical cleaning, wastes will first collect in temporary tanks or bladders and sampled and treated as necessary, and then be discharged to the Missouri River.

Total water demand for the Aquifer during normal operations is 24,160 gpm (91,446 lpm). From this total, 5,607 gpm (21,224 lpm) is returned to the Missouri River from the ESWS and CWS systems. The remaining 18,553 gpm (70,222 lpm) is evaporated in the CWS and ESWS cooling towers and from the ESWEMS pond.}

Section 2.3.2 provides a discussion of permitted activities associated with plant water consumption. Section 4.2 provides a discussion of water consumption during construction activities.

3.3.2 WATER TREATMENT

Water treatment will be required for both influent and effluent water streams. {Considering that, when constructed, the cooling water source for Callaway Plant Unit 2 will become the common source for both units. Cooling water treatment methodologies will be similar for both units.} Table 3.3-2 lists the principal water treatment systems and treatment operating cycles. The types, quantities and points of chemical additives to be used for water treatment are also indicated.

The {Water Treatment System provides treated water for the CWS and ESWS} and consists of three phases: makeup treatment, internal circulating water treatment and blowdown treatment. Makeup treatment will consist of a biocide (i.e., sodium hypochlorite) injected into {the Aquifer} water influent during spring, summer and fall months to minimize {aquatic} organism growth and control fouling on heat exchanger surfaces. Treatment will improve makeup water quality. {All water drawn from the Aquifer for use at the Callaway Plant site will receive treatment to control dissolved minerals, primarily iron concentration. Treatment reduces the dissolved solids load and prevents scaling in the circulating water systems. Aeration in combination with feed of a strong oxidizing agent (i.e., sodium hypochlorite) is proposed for oxidizing the dissolved iron in the Aquifer water from a ferrous iron state (dissolved) to a ferric iron state. The ferric iron species precipitates from the Aquifer water and can be removed by traditional clarification processes at the Water Treatment Plant. Chemically precipitated solids are coagulated by the addition of polyelectrolytes to the Aquifer water. The coagulated solids are concentrated in the bottom of the clarifiers and are continuously drawn off as sludge for disposal in the on-site sludge lagoons. The supernatant from the sludge lagoons is returned to the head of the water treatment plant and therefore there is no discharge of clarification wastes to the river.

An environmental permit to operate this treatment system will be obtained from the State of Missouri.} For prevention of Legionella, treatment for internal circulating water components (i.e., piping between the CWS Makeup Water Intake Structure and condensers) may utilize existing power industry control techniques consisting of hyperchlorination (chlorine shock) in combination with continuous or intermittent chlorination at lower levels, biocide and scale inhibitor addition. Blowdown treatment will depend on water chemistry, but is anticipated to include application of a biocide (i.e., sodium hypochlorite), dechlorination (i.e., sodium bisulfite) and scale inhibitor (i.e., dispersant) to control bio-growth, reduce residual chlorine, and protect against scaling, respectively.

ESWS cooling tower water chemistry will be maintained by the Water Treatment System, which is a nonsafety-related system designed to treat {Aquifer water} for normal operating conditions and {for} normal shutdown/cooldown. Treatment of system blowdown will also control the

concentration of various chemicals in the ESWS cooling water. During design basis accident conditions, the Water Treatment System is assumed to be non-operational.

{Deep well water} will be treated by the Demineralized Water Treatment System, which provides demineralized water to the Demineralized Water Distribution System. During normal operation, demineralized water is delivered to power plant systems. Treatment techniques will meet makeup water treatment requirements set by the Electric Power Research Institute. The {Demineralized Water Treatment System for Callaway Plant Unit 2 is shared with Callaway Plant Unit 1.}

The Drinking Water Treatment System, which supplies water for the Potable and Sanitary Distribution System, will treat {deep well water} so that it meets the State of {Missouri's} potable (drinking) water program and standards by U.S. EPA for drinking water quality under the National Primary Drinking Water Regulation (NPDWA) and National Secondary Drinking Water Regulation (NSDWA). The system will be designed to function during normal operation and outages (i.e., shutdown). However, treatment of {the well} water for the Fire Water Distribution System is not anticipated.

Liquid wastes generated by the plant during all modes of operation will be managed by the Liquid Waste Storage System and the Liquid Waste Processing System. The Liquid Waste Storage System collects and segregates incoming waste streams between radioactive and non-radioactive sources, provides initial chemical treatment of those wastes, and delivers them to one or another of the processing systems. The Liquid Waste Processing System separates waste waters from radioactive and chemical contaminants. The treated water is returned to the Liquid Waste Storage System for monitoring and eventual release. Chemicals used to treat waste water for both systems include sulfuric acid for reducing pH, sodium hydroxide for raising pH and an anti-foaming agent, complexing agent and/or precipitant for promoting settling of precipitates.

{Sewage Lagoons will be used to treat sewage for Callaway Plant Unit 2. Sanitary Wastes are discharged to the sewage lagoons, pumped to the Wetlands lagoons and recirculated to the Water Treatment Plant. This treatment system processes raw sewage so that discharged effluent conforms to applicable local and state health and safety codes, and environmental regulations.}

Section 5.2 provides a discussion on effluent limitations and permit conditions.

Table 3.3-1—{Anticipated Water Use}

Water Streams	Average Flow gpm (lpm)	Maximum Flow gpm (lpm)	Shutdown/ Cooldown Flow gpm (lpm)
Essential Service Water System (ESWS)/Essential Service Water Emergency Makeup System (ESWEMS) ⁽³⁾	1,887 (7,145)		3,764 (14,247)
ESWS Cooling Tower Evaporation	940 (3,558)		1,880 (7,116)
ESWS Cooling Tower Drift	2 (8)		4 (16)
ESWS Cooling Tower Blowdown	940 (3,558)		1,880 (7,116)
ESWEMS Pond	5 (21)	45 (170)	
Power Plant Makeup	103 (390)	116 (439)	
Deminerlized Water Distribution System	80 (303)	80 (303)	
Potable and Sanitary Water Distribution System ⁽⁴⁾	20 (76)	36 (136)	
Plant Users	20 (76)	36 (136)	
Non-Plant Users	0 (0)	0 (0)	
Fire Water Distribution System ⁽⁵⁾	3 (11)	625 (2,366)	
Missouri River Alluvial Aquifer Water Demand	24,160 (91,446)	34,305 (129,844)	
Wetlands return to Water Treatment System	10 (38)	15 (57)	
Circulating Water Supply System (CWS)	22,283 (84,340)	29,886 (113,118)	
CWS Cooling Tower Evaporation	17,607 (66,644)	19,924 (75,412)	
CWS Cooling Tower Drift ⁽⁶⁾	8 (31)	8 (31)	
CWS Cooling Tower Blowdown	4,667 (17,666)	9,954 (37,676)	
Effluent Discharge to Missouri River	5,673 (21,474)	11,929 (45,151)	
Miscellaneous Low Volume Waste	55 (208)	55 (208)	
Startup Temporary Storage Discharge ⁽⁷⁾	—	—	
Trash Screen Cleaning Water Discharge ⁽⁷⁾	—	—	
Treated Liquid Radwaste	11 (42)	40 (151)	

gpm – gallons per minute

lpm – liters per minute

Notes:

- (1) Average flow represents the expected water consumptive rates and returns for plant design operating conditions (4.8 cycles of concentration).
- (2) Maximum flow represents water consumptive rates and returns for plant operation at 3 cycles of concentration.
- (3) Two trains will be operating under normal conditions and four trains during shutdown/cooldown.
- (4) The average flow for potable water demand is based on projected staffing during normal plant operation. Non-plant water users include potable and sanitary needs for administrative buildings and warehouses, and water required for landscape maintenance. Non-plant water users are not included in the estimated demand; however, water stored in the raw water storage tank(s) should accommodate other station water users since it will be designed for peak load provisions.
- (5) During normal operating conditions, water consumed by the Fire Water Distribution System is attributed to system leakage and periodic testing. The maximum consumptive rate is based on meeting the National Fire Protection Association's requirement for replenishing fire protection water storage.
- (6) The average and maximum cooling tower drift losses are considered equivalent and are less than 0.0010% of the CWS flow rate.
- (7) Startup effluents occur during plant startup; the effluents will be stored within tanks or bladders, which will be removed once startup is complete. Makeup flows associated with startup and trash screen cleaning are anticipated to be minimal. Similarly, discharges associated with startup effluents and trash screen cleaning effluents, are also anticipated to be minimal.

Table 3.3-2—{Water Treatment Systems}

System	Point(s) of Addition	Chemical Additives	Estimated Quantity	Operating Cycle(s)
Water Treatment Plant System	Water Treatment Plant	Aeration	240 ft ³ /min (6.8 m ³ /min)	Normal Operating Conditions and Normal Shutdown/Cooldown
	Water Treatment Plant	Oxidizer/Biocide	278,511 gal/yr (1,054,162 l/yr)	
Circulating Water Treatment System ⁽¹⁾	CWS Piping	Dispersant	436,334 lb/yr (197,921 kg/yr)	Normal Operating Conditions and Normal Shutdown/Cooldown
	Cooling Tower Basin	Sulfuric Acid	22,087,792 lb/yr (10,019,023 kg/yr)	
	CWS Piping	Inhibitor	256,667 gal/yr (971,485 l/yr)	
Essential Service Water System (ESWS) Cooling Tower Water Treatment System ⁽²⁾	ESWS Piping	Dispersant	9,695 gal/yr (36,697 l/yr)	Normal Operating Conditions and Normal Shutdown/Cooldown
	ESWS Piping	Sulfuric Acid	2,781,853 lb/yr (1,261,848 kg/yr)	
	ESWS Piping	Inhibitor	48,476 gal/yr (183,483 l/yr)	
Demineralized Water Treatment System ⁽³⁾	Demineralized Water Distribution System Makeup	Sulfuric Acid	2,650 gal/yr (10,031 l/yr)	Normal Operating Conditions and Normal Shutdown/Cooldown
		Sodium Hydroxide	2,400 gal/yr (9,085 l/yr)	
Drinking Water Treatment System ⁽⁴⁾	Potable and Sanitary Distribution System Makeup	Sodium Hypochlorite	200 gal/yr (757 l/yr)	Normal Operating Conditions and Normal Shutdown/Cooldown
Liquid Waste Storage System and Liquid Waste Processing System ^{(4) (5),e}	Influent Waste Water	Sulfuric Acid	22,900 gal/yr (86,686 l/yr)	Normal Operating Conditions and Normal Shutdown/Cooldown
		Sodium Hydroxide	2,400 gal/yr (9,085 l/yr)	

Key:

gal/yr – gallons per year

ft³/yr – cubic feet per year

lb/yr – pounds per year

l/yr – liters per year

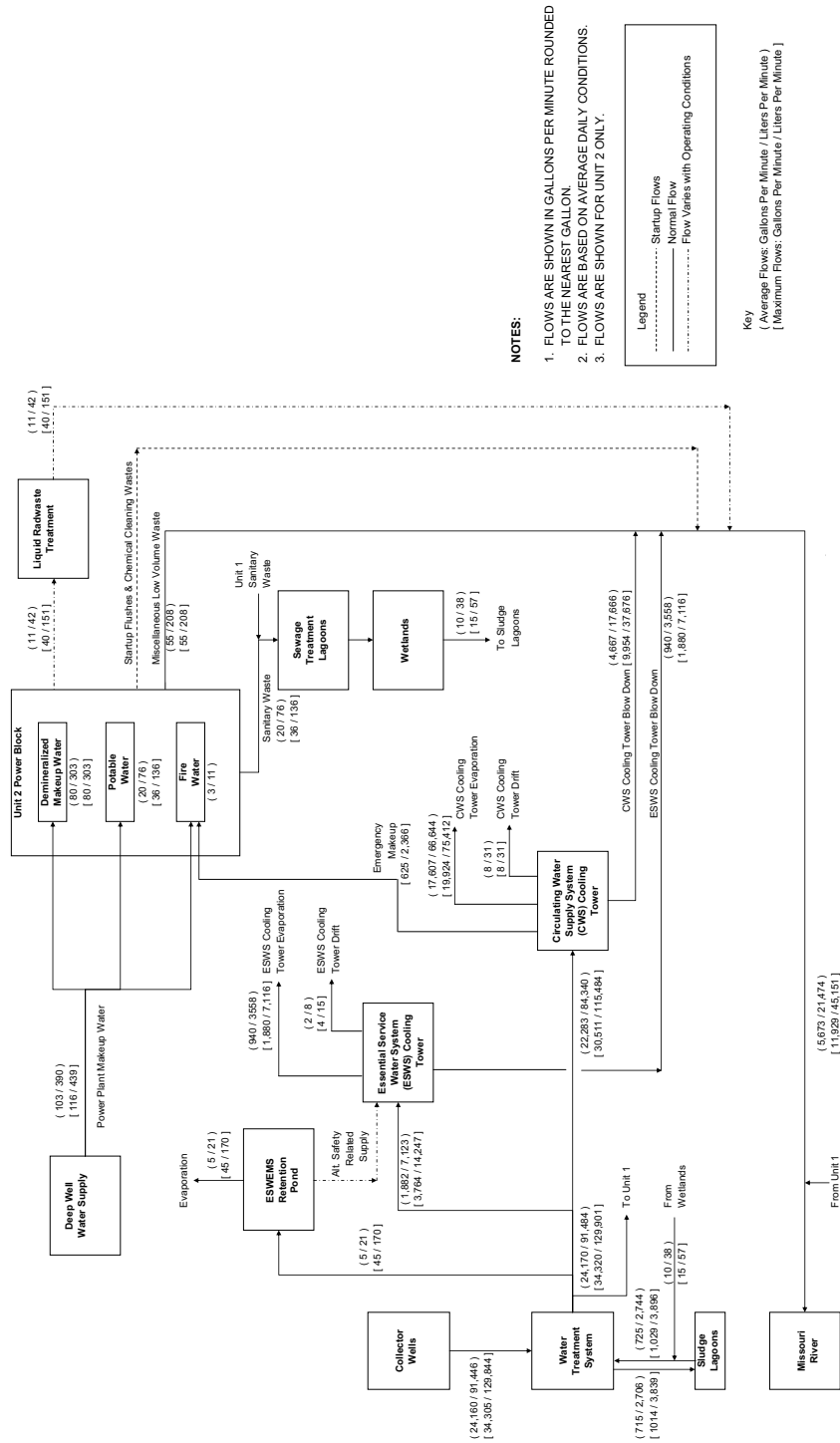
m³/yr – cubic meters per year

kg/yr – kilograms per year

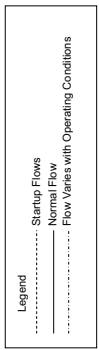
Notes:

- (1) The CWS has no safe shutdown or accident mitigation functions. Sodium hypochlorite will typically be added to makeup water. Sodium hypochlorite and dispersant may be added to piping. Chlorine may also be added to piping for prevention of Legionella. The estimated quantities of chemical additives are totals used throughout the Circulating Water Treatment System.
- (2) During a DBA, the ESWS Cooling Tower Water Treatment System is assumed to be non-operational. The estimated quantity of chemical additives is a combined total for both chemicals listed.
- (3) The estimated quantities of chemical additives are based on the existing Callaway Plant Unit 1 Demineralized Water Treatment System which uses the indicated chemicals for the regeneration of condensate demineralizers. The actual quantities of chemical additives will depend on how the demineralizer for Callaway Plant Unit 2 will be used (i.e., full-flow demineralizers use higher quantities).
- (4) Types and estimated quantities of chemical additives are based on the existing Callaway Plant Unit 1.
- (5) An anti-foaming agent, complexing agent and/or precipitant may also be used to promote settling of precipitates.

Figure 3.3-1—{Anticipated Water Use Diagram}



- NOTES:**
1. FLOWS ARE SHOWN IN GALLONS PER MINUTE ROUNDED TO THE NEAREST GALLON.
 2. FLOWS ARE BASED ON AVERAGE DAILY CONDITIONS.
 3. FLOWS ARE SHOWN FOR UNIT 2 ONLY.



Key

- () Average Flows: Gallons Per Minute / Liters Per Minute
- [] Maximum Flows: Gallons Per Minute / Liters Per Minute

3.4 COOLING SYSTEM

The {Callaway Plant Unit 2} cooling system design, operational modes, and component design parameters are determined from the U.S. EPR design documents, site characteristics, and engineering evaluations. The plant cooling systems and the anticipated cooling system operational modes are described in Chapter 9. Design data and performance characteristics for the cooling system components are {presented} in Chapter 9. These characteristics and parameters are used to assess and evaluate the {impacts on} the environment. The environmental interfaces occur at the intake and discharge structures and the cooling towers. There are two cooling systems that have intakes and cooling towers. These systems are the Circulating Water Supply System (CWS) and the Essential Service Water System (ESWS). [Figure 3.4-1](#) is a general flow diagram of the cooling water systems for {Callaway Plant Unit 2}.

3.4.1 DESCRIPTION AND OPERATIONAL MODES

3.4.1.1 Circulating Water Supply System/Auxiliary Cooling Water Systems

The U.S. EPR uses a Circulating Water Supply System (CWS) to dissipate heat. A closed-cycle, wet cooling system is used for {Callaway Plant Unit 2 similar to the existing Callaway Plant Unit 1 cooling system. The Callaway Plant Unit 2 system uses two non-plume abated natural draft cooling towers for heat dissipation. The CWS cooling towers will have the same basic structure and profile as the cooling tower for Callaway Plant Unit 1.} The CWS at {Callaway Plant Unit 2} dissipates up to $1.108 \text{ E}+10 \text{ Btu/hr}$ ($2.792 \text{ E}+09 \text{ Kcal/hr}$) of waste heat rejected from the main condenser and the Closed Cooling Water System (CLCWS) during normal plant operation at full station load. The exhausted steam from the low pressure steam turbine is directed to a surface condenser (i.e., main condenser), where the heat of vaporization is rejected to a loop of CWS cooling water. Cooling water from the CWS is also provided to the auxiliary cooling water system. Two 100% capacity auxiliary cooling water system pumps receive cooling water from the CWS and deliver the water to the CLCWS heat exchangers. Heat from the CLCWS is transferred to the auxiliary cooling water system and heated auxiliary cooling water is returned to the CWS. The heated CWS water is sent to the spray headers of the cooling tower, where the heat content of the water is transferred to the ambient air via evaporative cooling and conduction. After passing through the cooling tower, the cooled water is recirculated back to the main condenser and auxiliary cooling water system to complete the CWS loop. The CWS has nominal flow rate of {806,730 gpm (3,053,473 lpm)}.

Evaporation in the cooling towers increases the concentration of solids in the circulating water. To control solids, a portion of the recirculated water must be removed for blowdown and replaced with clean water. In addition to the blowdown and evaporative losses, a small percentage of water in the form of droplets (drift) will also be lost from the cooling tower. {Peak anticipated evaporative losses are approximately 19,924 gpm (75,412 lpm). Maximum blowdown is approximately 9,954 gpm (37,676 lpm). Maximum drift losses are about 8 gpm (31 lpm) based upon 0.001% of the CWS nominal flow rate. Makeup water from the Aquifer is required to replace the 29,886 gpm (113,118 lpm) losses from evaporation, blowdown and drift.}

{Makeup water for the CWS will be taken from the Aquifer by pumps at a maximum rate of approximately 34,305 gpm (129,844 lpm). This is based on maintaining the CWS and supplying the Essential Service Water System (ESWS) Cooling Tower with 3,764 gpm (14,247 lpm) and supplying the ESWEMS Pond with 5 gpm (21 lpm). The pumps will be installed in new collector wells located at the Missouri River approximately 1.5 river miles (2.4 km) upstream of the original Callaway Plant Unit 1 intake structure. The makeup water is pumped through a common header directly to the Water Treatment System and gravity flows to the cooling tower

basins. Blowdown from the cooling towers discharge to a common line and is discharged to the {Missouri River}. [Figure 3.4-2](#) shows the location of the {Callaway Plant} collector wells and discharge.}

The CWS water is treated as required to minimize fouling inhibit scaling on the heat exchange surfaces, to control growth of bacteria, particularly Legionella bacteria, and to inhibit corrosion of piping materials. Water treatment is discussed in Section 3.6.

3.4.1.2 Essential Service Water System/Ultimate Heat Sink

The U.S. EPR design has a safety-related ESWS to provide cooling water to the Component Cooling Water System (CCWS) heat exchangers located in the Safeguards Building and to the cooling jackets of the emergency diesel generators located in the Emergency Power Generating Buildings. The ESWS is used for normal operations, refueling, shutdown/cooldown, anticipated operational events, design basis accidents and severe accidents. The ESWS is a closed-loop system with four safety-related trains and one non-safety-related dedicated (severe accident) train to dissipate design heat loads. The non-safety-related train is associated with one safety-related train.

Safety-related two-cell mechanical draft cooling towers with water storage basins comprise the Ultimate Heat Sink (UHS) which functions to dissipate heat rejected from the ESWS. The two cells of an ESWS cooling tower share a single basin. The ESWS cooling tower basins are sized to provide sufficient water to permit the ESWS to perform its safety-related heat removal function for up to 72 hours post-accident under worst anticipated environmental conditions without replenishment. After 72 hours have elapsed post-accident, if required, the safety-related Essential Service Water Emergency Makeup System (ESWEMS) pumps may be operated to provide {fresh} water from the {ESWEMS Pond} to the ESWS cooling tower basins to maintain water inventory for the 30 day post-accident period as stipulated in Regulatory Guide 1.27 (NRC, 1976).

Each of the four ESWS cooling towers has a dedicated CCWS heat exchanger to maintain separation of the safety-related trains. Each ESWS safety-related train uses a dedicated mechanical draft cooling towers to dissipate heat during normal conditions, shutdown/cooldown, or design basis accident conditions. The non-safety-related train uses its associated safety-related train ESWS cooling tower to dissipate heat under severe accident conditions.

Heated ESWS water returns through piping to the spray distribution header of the ESWS cooling tower. Water exits the spray distribution header through spray nozzles and {falls} through the tower fill. Two fans provide upward air flow to remove latent and sensible heat from the water droplets as they fall through the tower fill. The heated air will exit the tower and mix with ambient air, completing the heat rejection process. The cooled water is collected in the tower basin for return to the pump suction for recirculation through the system. Each ESWS cooling tower has a dedicated ESWS pump with an additional pump to supply the severe accident train. [Table 3.4-1](#) provides nominal flow rates and heat loads in different operating modes for the ESWS.

The water loss from the {ESWS} is expected to be 1,882 gpm (7,124 lpm) based on 940 gpm (3,558 lpm) from evaporation, 940 gpm (3,558 lpm) from blowdown, and drift loss of 2 gpm (8 lpm) during normal conditions based on two trains operating. The water loss under shutdown/cooldown conditions will be approximately 3,764 gpm (14,248 lpm) based on 1,880 gpm (7,117 lpm) from evaporation, 1,880 gpm (7,117 lpm) from blowdown; and drift loss of 4

gpm (15 lpm) with all four ESWS cooling towers in operation. The blowdown from the four ESWS cooling towers is {pumped to the Missouri River.}

Makeup water to the ESWS is normally supplied from the Water Treatment System. The Water Treatment System gets water from {the Aquifer.} Under {post-accident conditions lasting longer than 72 hours, the makeup water will be supplied from the safety-related ESWEMS water system. The safety-related ESWEMS pumps are housed in a safety-related intake structure at the ESWEMS Pond.

The ESWS makeup water under DBA conditions will be provided at a minimum flow rate of approximately 600 gpm (2271 lpm) to accommodate for the maximum evaporation rate, drift and system losses with no anticipated blowdown for two ESWS cooling towers.}

Maximum ESWS blowdown and makeup rates are based on maintaining two cycles of concentration and evaporation at 81°F wet-bulb and 20% relative humidity.

The ESWS water is treated as required to minimize fouling, inhibit scaling on heat exchange surfaces, control growth of bacteria (particularly Legionella bacteria) and inhibit the corrosion of piping materials. Pumps, valves and other system component materials will be designed for use in {a fresh} water application.

3.4.1.3 Common Operational Factors

3.4.1.3.1 Station Load Factor

The U.S. EPR is designed to operate with a capacity factor of 95% (annualized), considering scheduled outages and other plant maintenance. For the site, on a long-term basis, an average heat load of 1.053 E+10 BTU/hr (2.652 E+9 Kcal/hr) (i.e., 95% of the maximum rated heat load of 1.108 E+10 BTU/hr (2.792 E+09 Kcal/hr)) will be dissipated to the atmosphere.

3.4.1.3.2 {Missouri River} Water Temperature

{Water temperatures measured from 1997 through 2007 ranged between 32°F (0°C) and 89°F (32°C). Because the station will be using collector wells to provide cooling water from the Aquifer, river icing will not be a problem. The river has large enough flow volume that it has never completely iced over.

Water temperature is important to water infiltration through the riverbank into the Aquifer. At colder temperatures, the water viscosity increases and the rate at which water can move through the riverbed into the aquifer decreases. This results in greater drawdown (lower water pumping levels); however, the Aquifer has sufficient saturated thickness to account for minor increases in drawdown despite the impact of temperature, allowing the required flow to the wells to be maintained.}

3.4.1.3.3 {Missouri River} Water Level

{Callaway Plant Unit 2 will rely on collector wells to supply groundwater and water induced from the Missouri River for normal makeup water to the CWS cooling towers, ESWS cooling towers, and the ESWEMS Pond. Callaway Plant Unit 2 will not rely on the collector wells for safe shutdown since the ESWEMS Pond contains sufficient storage volume for shutdown loads. The collector wells can produce sufficient volume of water even during periods of low river levels and will be designed to operate during a 100-year drought elevation. The low river levels associated with the drought will increase drawdown in the collector well; however, the Aquifer is sufficiently thick that the well can still produce its design capacity.

The 200-year maximum flood level at the intake location is Elevation 539 ft (164 m) msl. Critical facilities will be constructed at an elevation 2 ft higher than the 200-year flood level. This is also higher than the 500-year maximum flood level.}

3.4.1.3.4 Anti-Fouling Treatment

Bio-fouling is controlled using chlorination or other treatment methods in the CWS cooling tower basin. The chemical addition to the cooling tower ensures that the fill in the cooling tower remains free of biofilms and other organic deposits. Additional pre-treatment of the cooling tower makeup is provided, if required, based on periodic water chemistry sampling. Corrosion inhibitors may also be introduced at these injection points, as required, based on the system piping materials and water chemistry.

Bio-fouling is controlled using chlorination or other treatment methods in the {ESWS} cooling tower basins. {ESWS} cooling tower makeup water is normally supplied by fresh water from the {Aquifer. Under post-accident conditions lasting longer than three days, however, the makeup water will be fresh water from the ESWEMS pond. In either case,} makeup water will be subjected to appropriate filtration and treatment as required, based on periodic water chemistry sampling. Corrosion inhibitors may also be introduced at these injection points, as required, based on the system piping materials and water chemistry.

3.4.2 COMPONENT DESCRIPTIONS

The design data of the cooling system components and their performance characteristics during the anticipated system operation modes are described in this section. Site-specific estimates are used as the basis for discussion.

3.4.2.1 {Missouri River} Collector Wells

{The collector well system consists of multiple collector wells spaced approximately 1,500 ft (460 m) apart and situated on the north bank of the Missouri River. The general site location of the new collector well system is shown in [Figure 3.4-2](#), [Figure 3.4-3](#) and [Figure 3.4-4](#) show a typical collector well in more detail.

The collector wells consist of a concrete caisson extending down to bedrock at a depth of approximately 90 to 100 ft (27 to 30 m). Laterals with stainless steel screens extend outward from the caisson a maximum distance of approximately 200 ft (61 m). The laterals are located just above bedrock, and are placed radially around the caisson as shown in [Figure 3.4-4](#).

Each collector well is equipped with multiple vertical turbine pumps, including one redundant pump. A pump house placed on top of each caisson contains pumping equipment, valves and piping. The electrical controls are contained in a separate room.

Collector well yields will be monitored for potential declines in production that would indicate possible pump wear or the accumulation of deposited minerals or bacterial growth on the lateral screens. Pump or well maintenance will be performed as appropriate.

The combined pumping flow rate from the Aquifer for Callaway Plant Unit 2 will be a maximum of approximately 34,305 gpm (129,844 lpm)}.

3.4.2.2 {Final} Plant Discharge

{The final discharge consists of cooling tower blowdown from the CWS cooling tower, the ESWS cooling towers and site wastewater streams, including the circulation water treatment systems.

All biocides or chemical additives in the discharge will be among those approved by the U.S. EPA and the State of Missouri as safe for humans and the environment, and the volume and concentration of each constituent discharged to the environment will meet requirements established in the National Pollutant Discharge Elimination System (NPDES) permit. The types and quantities of chemicals used are discussed in Section 3.3.

The Callaway Plant Unit 2 discharge will combine with the discharge from Callaway Plant Unit 1 and flow to the Missouri River through the existing 36 inch (91 cm) diameter blowdown pipe. Note that treated liquid radioactive waste will discharge directly to the Missouri River through the same blowdown pipe. The existing discharge pipe is equipped with a Tideflex check valve to prevent backflow and promote mixing as the discharge flows into the river. The normal discharge flow from Callaway Plant Unit 2 will be approximately 5,673 gpm (21,474 lpm) and the maximum discharge flow will be approximately 11,929 gpm (45,151 lpm). This includes the nominal discharge flow from the CWS cooling tower of approximately 4,667 gpm (47,666 lpm).

The discharge is designed to provide an acceptable mixing zone for the thermal plume per the State of Missouri regulations for thermal discharges. Figure 3.4-5 shows details of the discharge system. The discharge point is on the north bank of the Missouri River approximately 440 ft (134 m) downstream of the existing intake structure for Callaway Plant Unit 1 and extends about 100 ft (30 m) into the river through a buried nominal 36 inches (91 cm) discharge pipe with a check valve/diffuser at the end of the line. The centerline elevation of the discharge is 493.5 ft (150.4 m) above msl. The angle of discharge is approximately 45 degrees from the river bank. Riprap has been placed around the existing discharge point to resist potential erosion due to the discharge jet from the blowdown pipe. Fish screens are not required on the discharge pipe since there is a check valve bolted to the end of the pipe. The length of the area of active mixing downstream of the discharge location, also known as the near field, is approximately 313 ft (95 m) when both Callaway Plant Unit 1 and Callaway Plant Unit 2 are discharging}.

3.4.2.3 Heat Dissipation System

{The two CWS cooling towers are used as the normal heat sink. The CWS cooling towers are natural draft cooling towers that each having a concrete shell rising to a height of approximately 550 ft (168 m). Internal construction materials include fiberglass-reinforced plastic (FRP) or polyvinyl chloride (PVC) for piping laterals, polypropylene for spray nozzles, and PVC for fill material. Natural draft towers use air conduction across sprayed water to reject latent and sensible heat from the sprayed water to the atmosphere. The CWS cooling towers will dissipate a maximum waste heat load of up to 1.108 E+10 BTU/hr (2.792 E+09 Kcal/hr) from the unit, operate with a 11°F (6.1 °C) approach temperature, and maintain a maximum 90°F (32°C) return temperature at design ambient conditions. Table 3.4-2 provides specifications of the CWS cooling towers. The cooling towers occupy an area of approximately 13 acres (5.3 hectares). Figure 2.1-1 shows the location of the CWS cooling towers while Figure 3.4-6 provides a schematic elevation view of the main cooling system for Callaway Plant Unit 2}.

{Each} ESWS cooling tower is a rectilinear mechanical draft structure. Each of the four ESWS cooling towers are a counterflow, induced draft tower and are divided into two cells. Each cell uses one fan, located in the top portion of the cell, to draw air upward through the fill, counter to the downward flow of water. One operating ESWS pump supplies flow to both cells of an operating ESWS cooling tower during normal plant operation. Table 3.4-1 provides system flow rates and the expected heat duty for various operating modes of the ESWS cooling towers. The ESWS cooling towers are designed to maintain a maximum 92°F (33°C) return temperature to the ESWS heat exchangers during normal operation, 95°F (35°C) during both design basis accident and severe accident conditions, and 90°F (32°C) during Shutdown/Cooldown}.

Temperature rise through the ESWS heat exchangers will be approximately 17°F (9°C) during normal operation and 19°F (11°C) during cooldown operation based on the heat transfer rates defined in [Table 3.4-1](#). Blowdown from the ESWS cooling towers is mixed with CWS blowdown. The ESWS cooling towers are located on either side of the power block (two ESWS cooling towers per side), to provide spatial separation, with each ESWS cooling tower occupying an area of approximately 0.19 acres (0.077 hectares). The noise levels generated by the ESWS cooling towers is approximately 65 dBA or less at the distance of approximately 1,300 ft (396 m) from the cooling towers. [Table 3.4-3](#) provides specifications of the ESWS cooling towers. Locations of the ESWS cooling towers are shown on [Figure 2.1-1](#) while [Figure 3.4-7](#) provides a {schematic flow diagram for the ESWS cooling towers.}

3.4.3 REFERENCES

{**NRC, 1976.** Ultimate Heat Sink for Nuclear Power Plants, Regulatory Guide 1.27, Revision 2, Nuclear Regulatory Commission, January 1976.}

Table 3.4-1—Minimal and Nominal Essential Service Water System Flows and Heat Loads at Different Operation Modes Per Train

	Minimum Flow (gpm / lpm)⁽¹⁾	Nominal Flow (gpm / lpm)⁽¹⁾	Heat Transferred (BTU/hr / Kcal/hr)	Anticipated Number of Trains Operating
Normal Operation (Full Load)	17,340 / 65,639	19,075 / 72,206	165 E+06 / 416 E+05	2
Cooldown	17,340 / 65,639	19,075 / 72,206	182 E+06 / 459 E+05	4
Design Basis Accident	17,340 / 65,639	19,075 / 72,206	313 E+06 / 789 E+05	2
Severe Accident	2,420 / 9,160	2,665 / 10,088	55 E+06 / 139 E+05	1

Note:

- (1) Based on a mass flow rate (lbm/hr) converted to gpm using water properties at 14.7 psia (101.4 kPa) and 60°F (15.56 °C)

Table 3.4-2—{Circulating Water System Cooling Tower Design Specifications}

Design Conditions	Natural Draft Cooling Towers
Number of Towers	2
Heat Load	1.108 E+10 BTU/hr (2.792 E+09 Kcal/hr)
Circulating and Service Water	806,730 gpm (3,053,805 lpm)
Cycles of Concentration—Normal	4.8
Approximate Dimensions	
Height	550 ft(168 m)
Overall diameter	414 ft(126 m) (at the base)
Design Dry Bulb Temperature	95°F (35°C)(summer) ⁽¹⁾
Design Wet Bulb Temperature	79°F (26.1°C)(summer)
Design Range	28°F (15.6°C)
Design Approach	11°F (6.1°C)
Air Flow Rate (at ambient design point)	69,064,000 cfm (1,955,675 m ³ per min)
Drift Rate	<0.001%

Note:

- (1) Based on tower design at 50% relative humidity

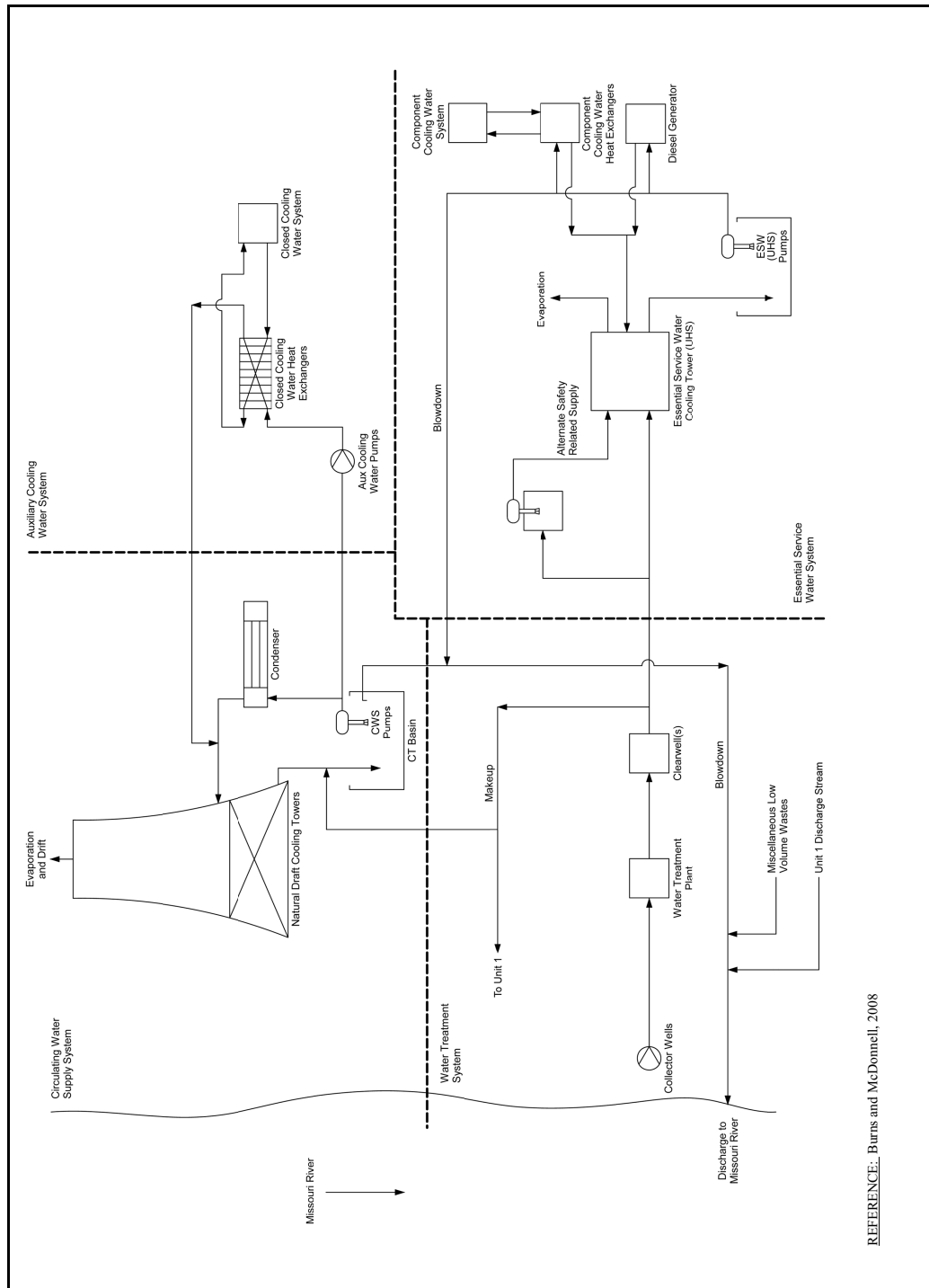
Table 3.4-3—{Essential Service Water System Cooling Tower Design Specifications}

Design Conditions	Mechanical Draft Cooling Tower
Number of Towers	4
Heat Load	See Table 3.4-1
Essential Service Water	See Table 3.4-1
Cycles of Concentration—normal	2
Approximate Dimensions	
Height	96 ft (29 m)
Overall length	60 ft (18.29 m)
Overall width	60 ft (18.29 m)
Design Dry Bulb Temperature (summer)	98.55°F (37°C)
(winter) ⁽¹⁾	25°F (-3.9°C)
Design Wet Bulb Temperature (summer)	81°F (27.2°C)
(winter) ⁽²⁾	24.3°F (-4.3°C)
Design Range	18.4°F (10.2°C)
Design Approach	7°F (3.9°C)
Air Flow Rate (at ambient design point)	1,213,000 cfm (3,438 m ³ /min)
Drift Rate	<0.005%

Notes:

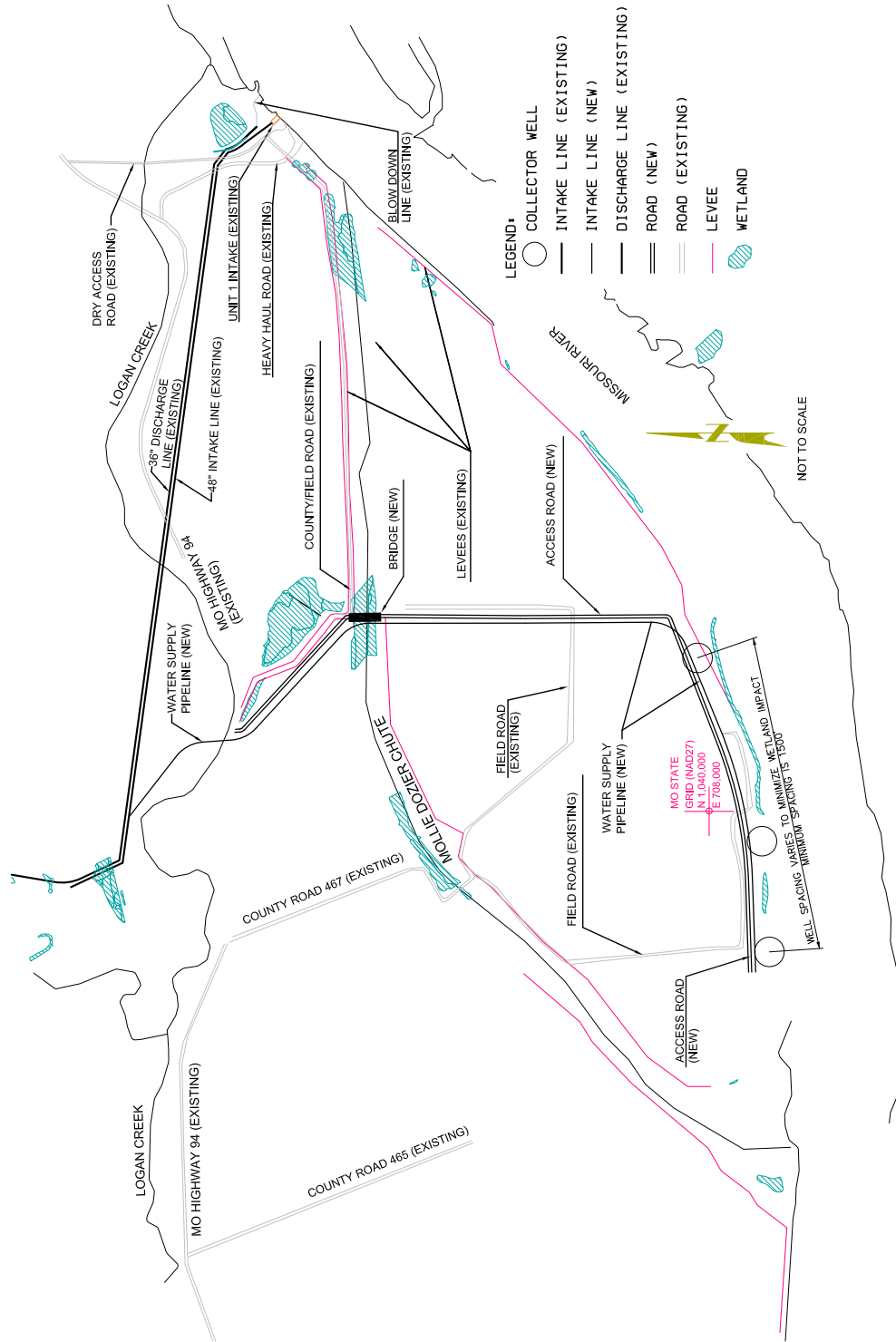
- (1) Based on tower design at 50% relative humidity
- (2) Includes 1°F (0.56°C) for recirculation

Figure 3.4-1—{General Cooling System Flow Diagram for Callaway Plant Unit 2}



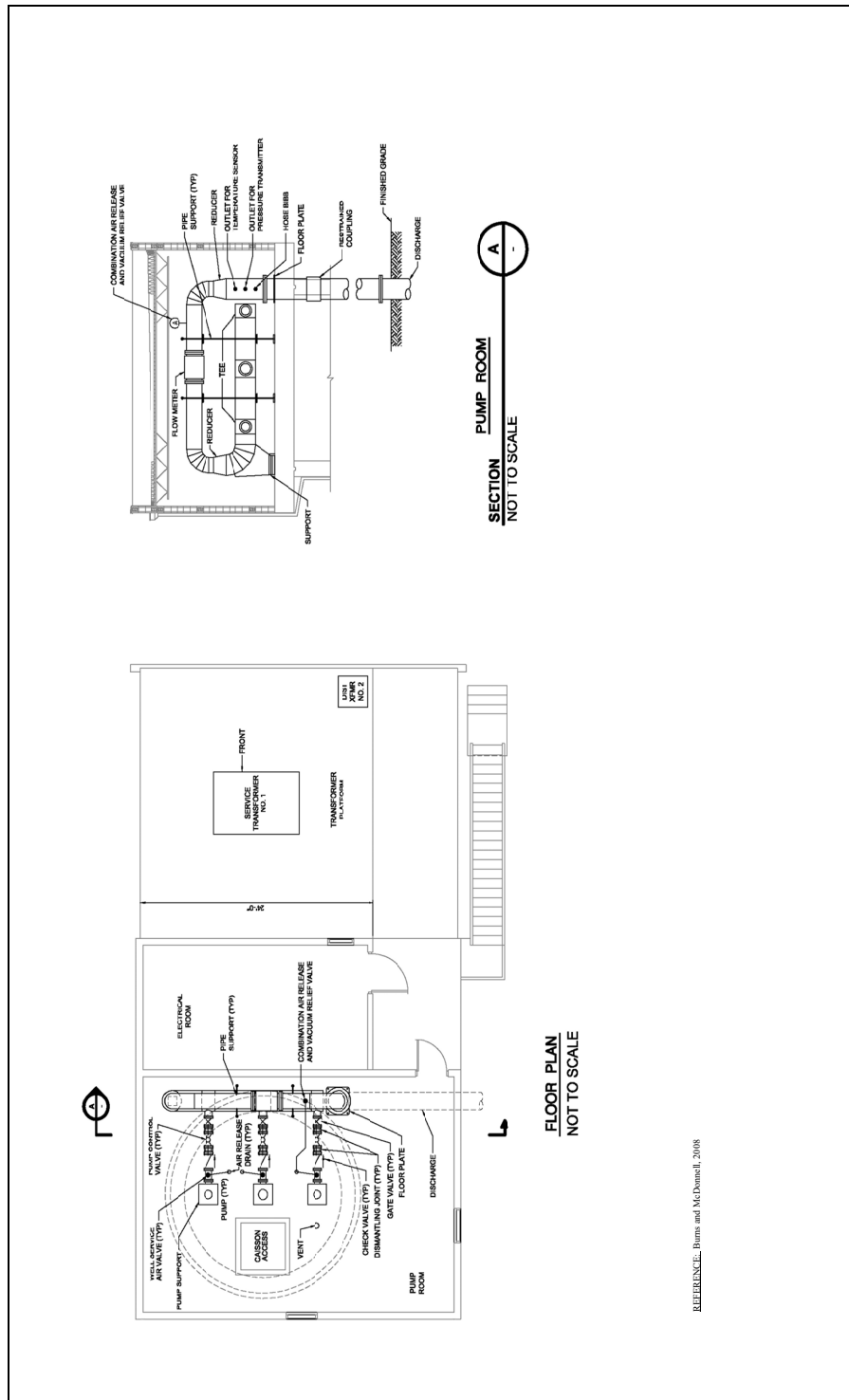
REFERENCE: Burns and McDonnell, 2008

Figure 3.4-2—{CWS Intake/Discharge Location Plan}



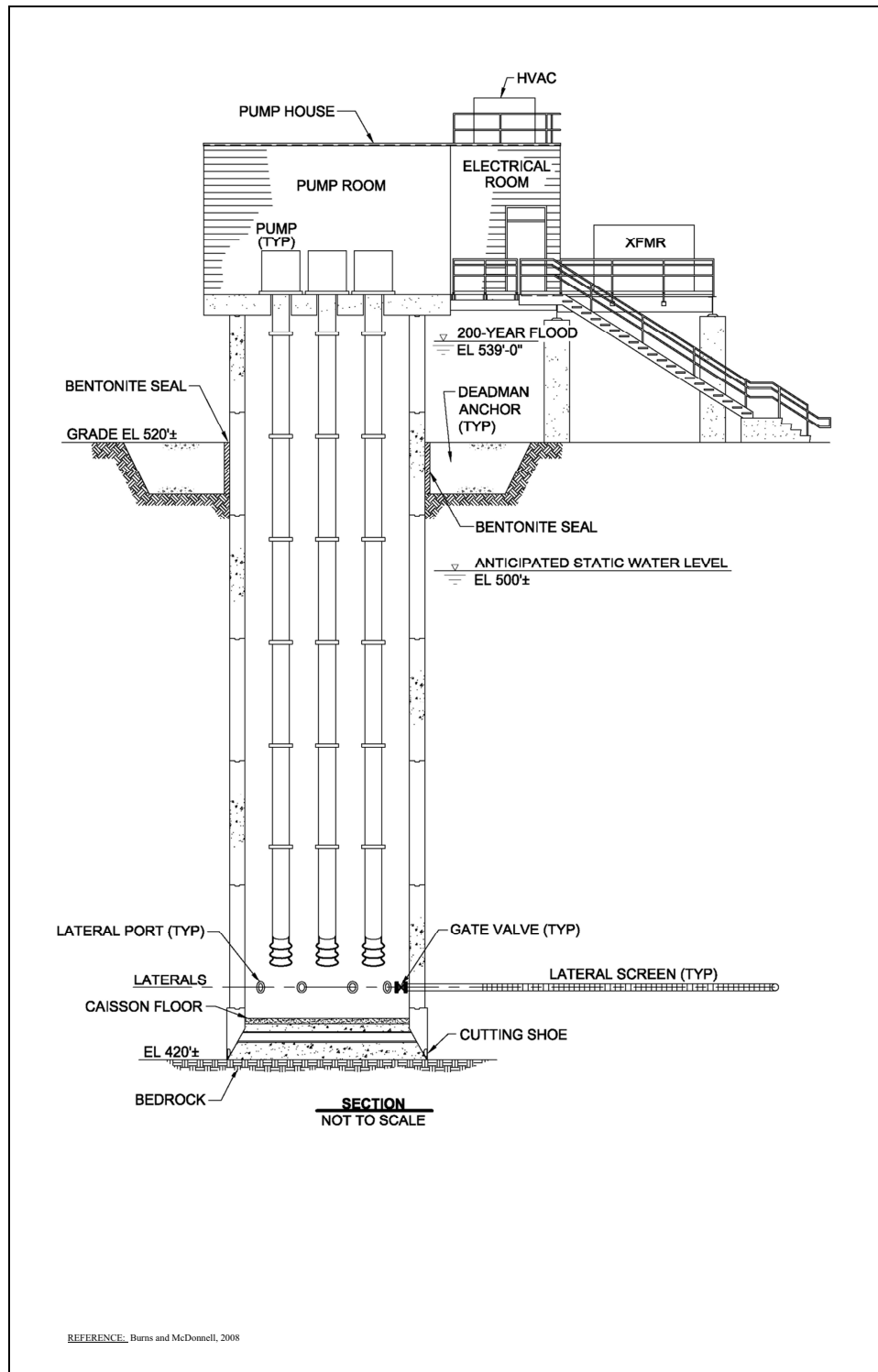
ER Chapter 3.0

Figure 3.4-3—{Typical Collector Well Plan for Callaway Plant Unit 2}



REFERENCE: Burns and McDonnell, 2008

Figure 3.4-4—{Typical Collector Well Section for Callaway Plant Unit 2}



ER Chapter 3.0

Figure 3.4-5—{View of Unit 1 & 2 Discharge Outfall}

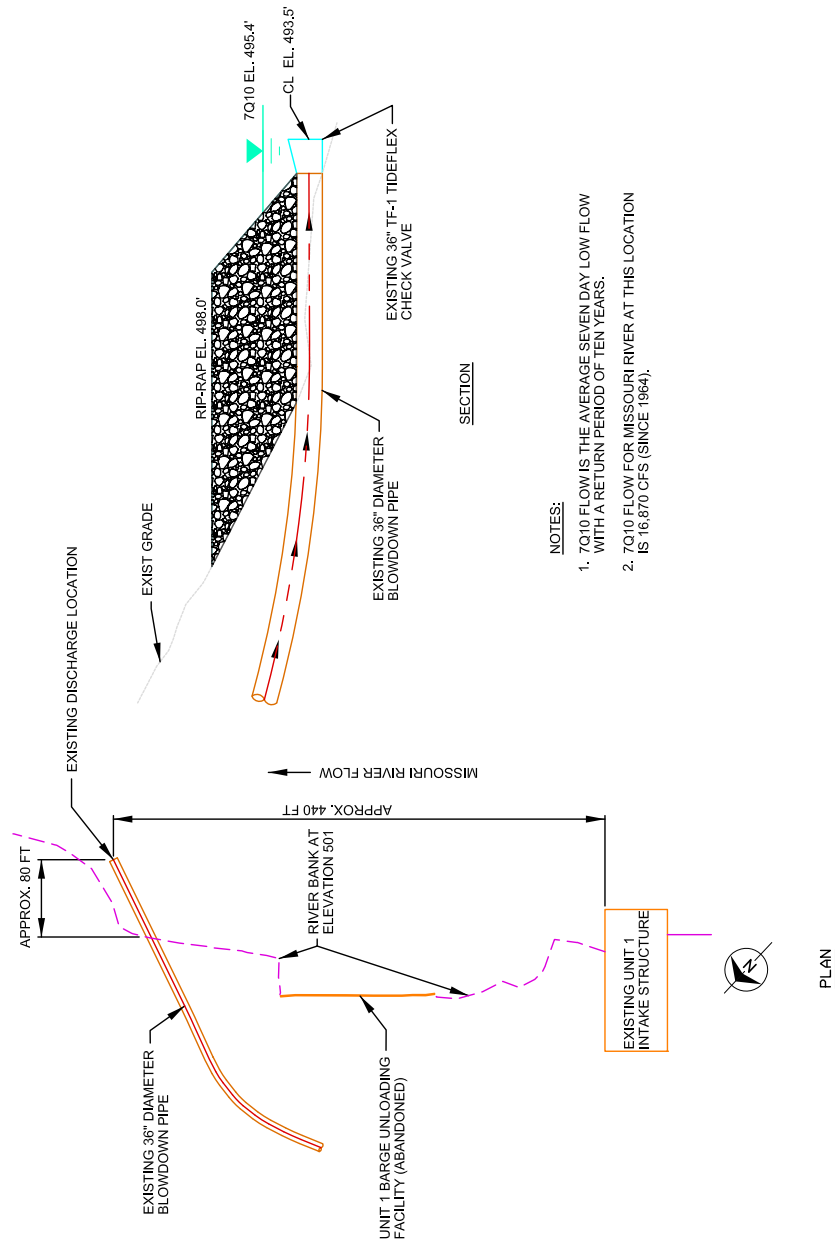


Figure 3.4-6—{Main Cooling Tower Schematic Elevation View}

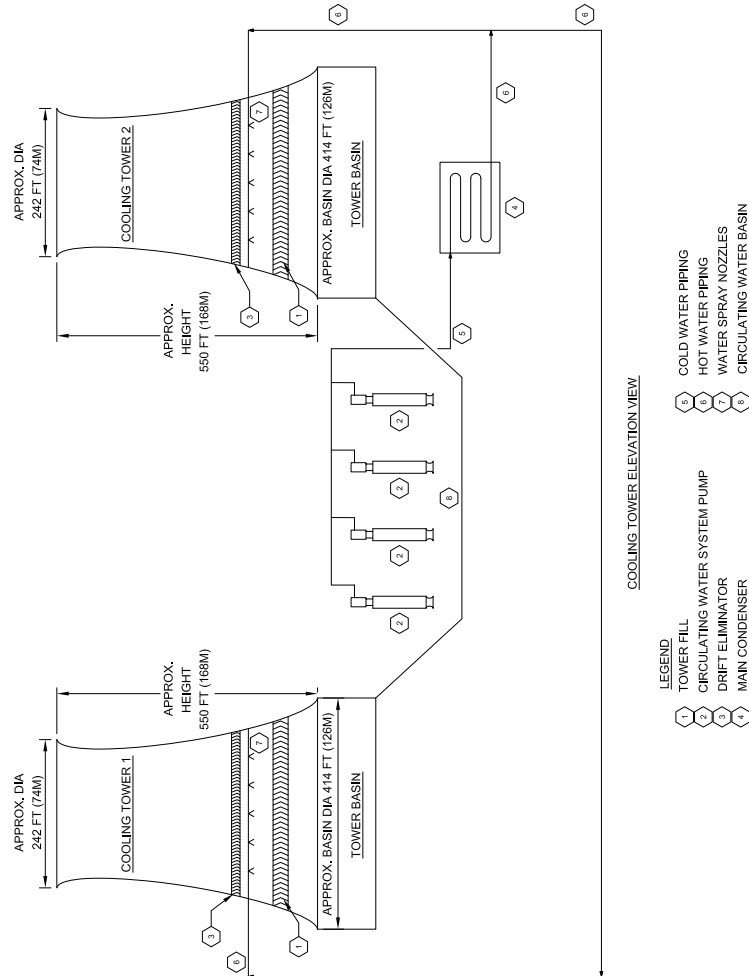
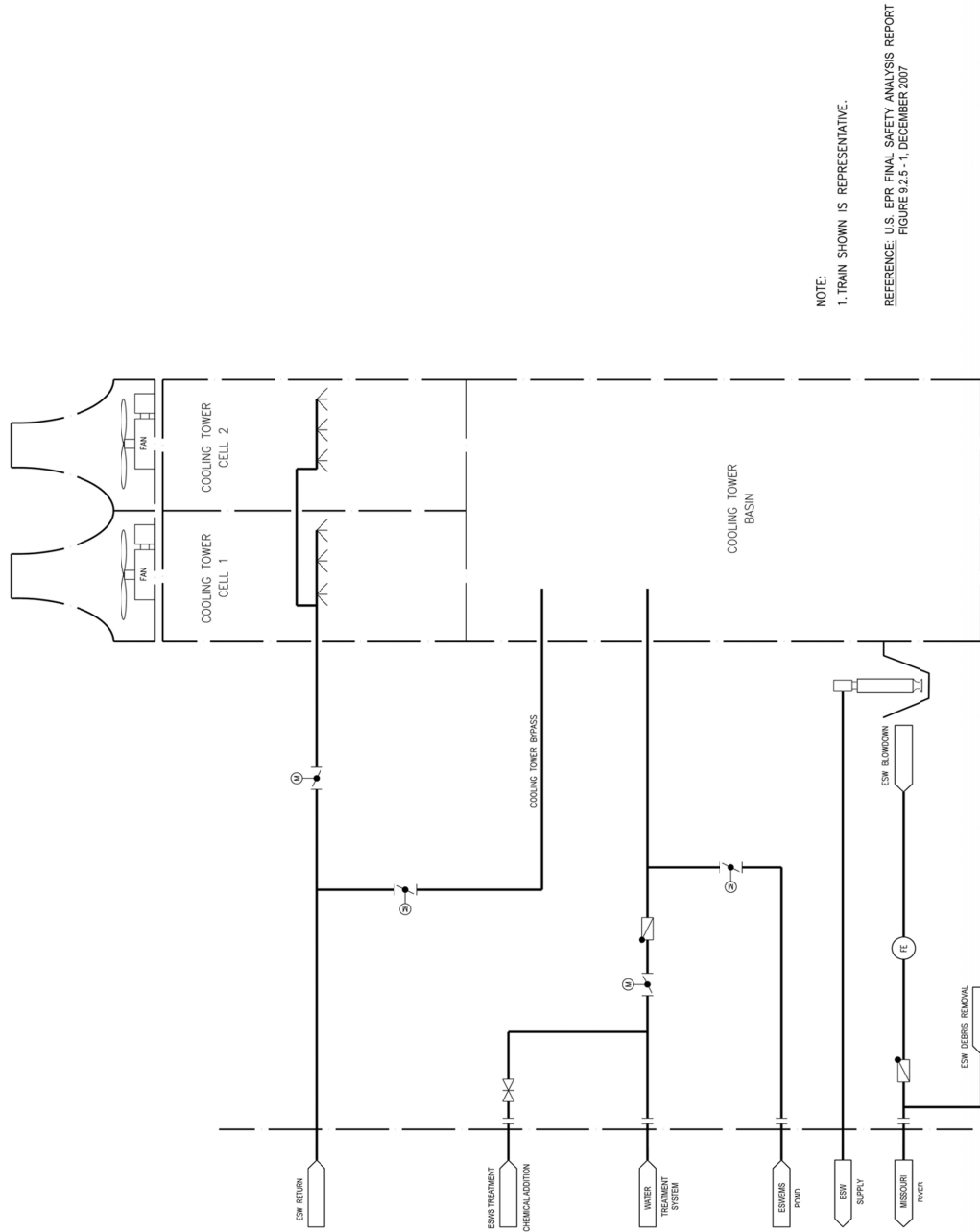


Figure 3.4-7—{ESWS Cooling Tower System Schematic}



NOTE:
1. TRAIN SHOWN IS REPRESENTATIVE.

REFERENCE: U.S. EPR FINAL SAFETY ANALYSIS REPORT
FIGURE 9.2.5 - 1, DECEMBER 2007

3.5 RADWASTE SYSTEMS AND SOURCE TERM

The generation of power within the reactor results in the presence of radioactive materials in various forms and quantities within the reactor core, reactor coolant system and associated systems and components. The vast majority of the radioactivity produced (fission products) is completely contained within the clad fuel rods and is therefore not available for release to fluid systems or to the environment. However, if imperfections in the cladding are present a small fraction of these fission products escapes from the affected fuel rods to the reactor coolant. The other main source of radioactivity to the reactor coolant is the corrosion of primary system surfaces and irradiation of the corrosion products within the reactor core.

Fission and activated corrosion product radionuclides within the reactor coolant system constitute the source of radioactivity to associated systems and components. This radioactivity appears in letdown and leakage from these systems and components which, in turn, forms the source of radioactivity in liquid and gaseous discharges from the plant site and in solid waste materials generated within the plant. System effluents are collected, processed, monitored and directed for either reuse or release to the environment by the radioactive waste treatment systems. Solid radioactive wastes are collected and packaged for temporary storage, shipment and offsite disposal.

The design and operational objectives of the {Callaway Plant Unit 2} radioactive waste treatment systems are to maintain, during normal operation, the radioactivity content of liquid and gaseous effluents from the site such that the dose guidelines expressed in Appendix I to 10 CFR Part 50 (10 CFR 50.34a) (CFR, 2007a), 40 CFR Part 190 (CFR, 2007b), and 10 CFR 20.1301(d) (CFR, 2007c) are met. The following descriptions of the design and operation of the radioactive waste treatment systems and presentations of the estimated radioactivity content of plant effluents serve to quantify the magnitudes and characteristics of the releases. These releases are then used as the sources for the radiological environmental impact analyses during normal operation, which are presented in Section 5.4 and demonstrate that the radioactive waste treatment systems are designed to keep doses to the public as low as reasonably achievable (ALARA). The dose to the public from radwaste systems during plant operation will meet the dose limits for individual members of the public as specified in 10 CFR 20.1301 (CFR, 2007c).

3.5.1 SOURCE TERMS

Source terms used in the evaluation of radwaste systems and effluent releases are discussed in this section. A power level of 4,612 MW(t) is used to calculate source terms based on the guaranteed core thermal output of 4,590 MW(t) plus a 22 MW(t) (approximately 0.5%) uncertainty allowance for heat balance measurements.

3.5.1.1 Primary Coolant Source Term

Two sets of source terms (reactor coolant radionuclide concentrations) have been determined. The first is a conservative design basis used for waste system performance calculations. This source term is based on the assumption that the primary coolant radionuclide concentrations are made up of a combination of proposed technical specification limits for halogens (1 $\mu\text{Ci/gm}$ Dose Equivalent (DE)-I-131 in primary coolant) and noble gases (210 $\mu\text{Ci/gm}$ DE-Xe133). Activation products and tritium are derived from the ANSI/ANS 18.1-1999 standard (ANS, 1999). Since the activated corrosion products are independent of failed fuel fraction, design basis and realistic basis concentrations are assumed to be the same. Design basis values for the remaining fission product radionuclides are calculated based on a 1.0% failed fuel fraction. The mathematical model used is described in Section 11.1 of the U.S. EPR FSAR. [Table 3.5-1](#) lists key design basis parameters used in the source term calculation for the primary coolant. [Table 3.5-2](#) summarizes the design basis reactor coolant concentration results. Design basis secondary coolant concentrations are

based on an assumed primary to secondary leak rate totaling 600 gpd (2,271 L/d) from all four steam generators. [Table 3.5-3](#) summarizes the secondary coolant liquid and steam phase radioactivity concentrations for design basis conditions.

The second source term is based on a realistic model in which the reactor coolant radionuclide concentrations are based on observed industry experience. The model used is described in Regulatory Guide 1.112 (NRC, 1976a), with the source term calculated using NUREG-0017, Revision 1 (NRC, 1985), which contains the Nuclear Regulatory Commission Pressurized Water Reactor (PWR) Gale Code, revised 1985. Specific parameters used in the calculation are provided in [Table 3.5-4](#).

The resulting radioactivity concentrations in the reactor coolant are listed in [Table 3.5-2](#). The inventories calculated in this manner represent “expected basis” activities and are used for the evaluation of environmental impact during routine operation, including anticipated operational occurrences. The data presented in [Table 3.5-2](#) do not include a shutdown iodine spike. Design basis accident analyses include iodine spikes and are discussed in Section 7.1.

Tritium is produced in the reactor mainly through the interaction of neutrons with soluble boron in the coolant. Additional contributions come from the ternary fissions and from the interaction of neutrons with burnable poison rods, lithium and deuterium. Some of the tritium formed within fuel materials will be present in the reactor coolant due to diffusion and leakage through the fuel cladding. For the U.S. EPR design, the expected tritium production rate in the Reactor Coolant System is 1,840 Ci/yr (6.81E+13 Bq/yr). The concentration of tritium in the reactor coolant is provided in [Table 3.5-2](#).

Radioactivity enters the spent fuel pool due to contamination by reactor coolant during refueling operations and possible fission product releases from spent fuel during the storage period. These radionuclides are continuously removed through the spent fuel pool purification train and the building ventilation filtration system. Therefore, the radioactivity in the spent fuel pool area is not a major source of environmental releases (except for tritium and noble gases). Activity concentrations in the fuel pool and atmosphere are listed in [Table 3.5-5](#).

3.5.1.2 Transported Source Terms

The radioactivity in the reactor is transported to various locations in the plant through plant fluid systems and leakages. A schematic diagram of the radwaste effluent flow paths is provided on [Figure 3.5-1](#).

Normal plant operation is anticipated to result in a certain degree of radioactivity within the secondary coolant systems through primary-to-secondary steam generator tube leakage. With steam generator tube defects present, radioactivity will be released to the environment through steam leakage, condensate leakage, and main condenser off gases. The concentrations of radionuclides in the secondary coolant system are based on ANSI/ANS-18.1-1999 (ANS, 1999) for the reference PWR with U-tube steam generators. The results are shown in [Table 3.5-3](#). The radioactivity present in the reactor coolant and secondary coolant are further transported through various radwaste systems and become source terms for environmental releases.

Liquid Source Terms

The following sources are considered in calculating the release of radioactive materials in liquid effluents from normal operations;

- a. Processed water generated from the boron recovery system to maintain plant water balance and for tritium control,

- b. Processed liquid waste from the containment building sump, floor drains from the auxiliary building, spent fuel building and radwaste building, laboratory drains, sampling drains, and other controlled area drains, and miscellaneous waste,
- c. Unprocessed liquid waste from the turbine building floor drain sumps.

The radioactivity input to the liquid radwaste treatment system is based on the flow rates of the liquid waste streams and their radioactivity levels expressed as a fraction of the primary coolant activity. [Table 3.5-6](#) shows the liquid waste flow rate and activity level. The table indicates radioactivity in each stream as a fraction of primary coolant activity prior to treatment and the decontamination factors applied to waste processing and effective decay time while passing through treatment systems.

Isotopic distribution for various waste streams is shown in [Table 3.5-7](#) for the liquid waste system.

Gaseous Source Terms

The following sources are considered in calculating the releases of radioactive materials (noble gases, iodines and particulates) in gaseous effluents from normal operation:

Containment purges (continuous),

Non-condensable gases from the gaseous waste system,

Nuclear auxiliary building(s) ventilation,

Radwaste, Spent Fuel and Safeguard Buildings, Ventilation

Turbine building ventilation,

Main condenser evacuation exhaust.

Any leakage of primary coolant or the process stream either in the containment or in the auxiliary buildings are collected in the buildings and vented through filtration systems to the environment. Any steam/water leakages in the turbine building are directly vented to the environment. The non-condensable gases will also be discharged through the main condenser evacuation system exhaust to the plant stack.

The estimated releases, by isotope, from each source are shown in [Table 3.5-8](#) for normal operation. This table is based on the expected basis source term information presented above, assumptions and parameters in [Table 3.5-9](#).

Solid Source Terms

The following sources are considered in calculating the solid waste generated within the plant. Solidified radioactive waste results from the processing of materials from the following sources:

- a. Evaporator concentrates from:
 - ◆ Liquid waste evaporator.
 - ◆ Boron recovery evaporator.

- ◆ Liquid waste centrifuge.
- b. Spent resin from:
 - ◆ Spent fuel pool demineralizer.
 - ◆ Reactor coolant purification treatment ion exchangers.
 - ◆ Liquid waste system demineralizers.
 - ◆ Boron recycle system ion exchanger.
- c. Liquid from decontamination solutions
- d. Spent radioactive filter cartridges from various plant filtering systems and other solid non-compressible radioactive waste.

In addition to solid materials extracted from liquid processing systems, dry active waste (DAW) solids are also generated as the result of collecting low activity compressible waste such as paper, rags (cloth) and polyethylene bags from inside the radiation control area. Non-compressible DAW can include such materials as scrap metal, glass, wood, and soil. [Table 3.5-10](#) summarizes as bounding estimates the annual solid wastes generated.

3.5.2 RADIOACTIVE LIQUID PROCESSING SYSTEMS

The primary design functions of the Liquid Waste Storage System and the Liquid Waste Processing System are to receive radioactive liquid wastes collected from the various systems and buildings in which they were generated, to process those liquid wastes in a manner that reduces the activity present in the aggregate liquid wastes such that discharges to the environment can be controlled to stay below 10 CFR 20, Appendix B, Table 2 concentration limits (CFR, 2007d), and the ALARA design dose objectives of 10 CFR 50, Appendix I (CFR, 2007a) for members of the public. Discharges to the environment must also meet state and federal limits specified in discharge permits.

Normal plant operation also has the potential to result in a certain degree of radioactivity within the secondary coolant systems due to primary-to-secondary steam generator tube leakage. Blowdown and leakage of secondary coolant then constitutes radioactive liquid sources, the radioactivity contents of which are reduced and/ or accounted for by the steam generator blowdown processing system and the condensate leakage collection system.

[Figure 3.5-2](#) provides a simplified drawing of the Liquid Waste Storage System and the Liquid Waste Processing System. The discussions that follow describe the design and operation of each of these systems with greater details found in Section 11.2 of the U.S. EPR FSAR. [Figure 3.5-3](#) provides a simplified drawing of the liquid waste treatment system showing the evaporator and centrifuge. [Figure 3.5-4](#) provides a simplified drawing of the Liquid Waste Treatment System showing the vendor supplied demineralizer system.

3.5.2.1 Liquid Waste System

The U.S. EPR Liquid Waste Storage System and Liquid Waste Processing System are used to manage liquid wastes generated by the plant during all modes of operation. The Liquid Waste Storage System collects and segregates incoming waste streams, provides initial chemical treatment of those wastes and delivers them to one or more of the processing systems. The

Liquid Waste Processing System uses evaporation, centrifugal separation, or demineralization and filtration to separate the waste water from the radioactive and chemical contaminants, and to concentrate those contaminants. The cleaned water is returned to one of two waste monitoring tanks in the liquid waste storage system where it is isolated and recirculated to ensure representative samples can be taken and analyzed prior to release to the environment. Once the monitoring tank contents are deemed suitable to be released, the processed liquid is discharged from the monitoring tank to the {Missouri River} via a discharge line with a radiation monitor that will stop the release if unexpected or high radioactivity is detected. {The radwaste discharge line for Callaway Plant Unit 2 connects to the cooling tower blowdown line before release to the Missouri River.} The concentrates are returned to the Liquid Waste Storage System for further concentration and eventual transfer to the radioactive concentrates processing system.

The Liquid Waste Storage System collects liquid wastes from the plant, segregates the wastes based on their expected radioactivity and chemical composition, and stores them in the liquid waste storage tanks accordingly.

Group I wastes are those liquid wastes expected to contain radioactivity and boron, but little or no organic and inorganic substances or solids. Sources of Group I liquid wastes include:

- ◆ water from the Fuel Pool Cooling System and Fuel Pool Purification System transferred through the floor drains of the Nuclear Auxiliary Building,
- ◆ liquid waste from Decontamination Systems,
- ◆ waste water from sampling and from process drains and sumps collected in the Nuclear Auxiliary Building,
- ◆ waste water drained from the evaporator column in the Liquid Waste Processing System,
- ◆ waste water decanted from the concentrate tanks and waste water returned from the radioactive concentrates processing system, and
- ◆ waste water collected from the floor drains of the radioactive waste processing building.

Group I wastes are directed to the Group I liquid waste storage tanks.

Group II wastes are those liquid wastes expected to contain low levels of radioactivity, along with organic and inorganic substances and some solids. Sources of Group II liquid wastes include:

- ◆ waste water collected from floor drains and sumps of the Nuclear Auxiliary Building,
- ◆ waste water from the hot laboratory transferred through the sumps of the Nuclear Auxiliary Building,
- ◆ waste water from the showers and washrooms in the Nuclear Auxiliary Building,
- ◆ distillate from the Reactor Coolant Treatment System, and

- ◆ treated water returned from the centrifugal separator in the Liquid Waste Processing System.

Provisions exist for collection of Group II wastes from the Steam Generator Blowdown Demineralizing System flushing water. Group II wastes are directed to the Group II liquid waste storage tanks.

Group III wastes are those liquid wastes expected to contain no radioactivity, but some organic or inorganic chemicals, under normal plant operating conditions. Group III waste collection headers are shared with some of the Group II collection headers; the wastes carried in these headers normally are directed to the Group III liquid waste storage tank provided that there are no indications that the waste water contains radioactivity. The “shared” sources are the wastes from the Steam Generator Blowdown Demineralizing System flushing water, and treated water returned from the centrifugal separator in the Liquid Waste Processing System. Provisions also exist for the collection of wastes from some of the floor drains in the radioactive waste processing building.

Since Group III waste liquids normally will have very little or no radioactivity, several of the Group III waste water streams may be routed directly to the monitor tanks in the liquid waste storage system. The Steam Generator Blowdown Demineralizing System flushing water wastes, and the treated water returned from the centrifuge separator in the Liquid Waste Processing System each can be routed directly to the monitor tanks instead of the Group III liquid waste storage tank.

The Liquid Waste Storage System and the Liquid Waste Processing System operate independently of the operating modes of the plant. The systems provide sufficient storage and treatment capacity to process the daily inputs produced during all plant startup, normal operation, plant shutdown, maintenance, and refueling periods. The systems are operated on an as-needed basis throughout the plant operating cycle. From operating experience, the peak volume demand occurs during plant outages, when increased volumes of waste water, in particular the Group II waste water streams, are generated by increased maintenance activities.

The liquid waste storage system includes liquid waste storage tanks, concentrates tanks, and monitoring tanks which temporarily store the liquid wastes at various stages of treatment. It also includes recirculation pumps, a sludge pump, a concentrates pump, and recirculation/discharge pumps to move the liquid waste between the various tanks. Chemical tanks and chemical proportioning pumps are included to permit the precise mixing and injection of chemicals to treat the liquid waste. Piping and control valves route the liquid wastes between the different tanks and pumps, as well as to several interfaces with the liquid waste processing system.

The liquid waste processing system consists of three separate sections. The evaporator section employs a vapor-compressor type evaporator with a separate evaporator column. The evaporator section also includes evaporator feed pumps, a forced recirculation pump, and a distillate pump to move liquid waste through the evaporation process, several heat exchangers to condition the liquid waste at various stages of the process, and a distillate tank to collect the treated waste water for return to the Liquid Waste Storage System.

The centrifuge section employs both a decanter and a centrifugal separator to separate organic and inorganic contaminants from the waste water. The contaminant ‘sludge’ is collected in a sludge tank, then pumped to a waste drum for collection and processing as solid waste. The treated water is returned to the Liquid Waste Storage System. The demineralizer and filtration

section includes a demineralizer and an ultra-filtration unit. Piping and control valves allow liquid wastes to be passed through either unit or through both units consecutively; contaminants are retained and the cleaned waste water is returned to the Liquid Waste Storage System. The capacity of the Liquid Waste Processing System is sufficient to process the average quantity of liquid wastes produced weekly in less than half that period of time. The Liquid Waste Processing System consists of three different subsystems, each of which applies a unique process to concentrate and remove radioactive material from liquid wastes. The processes used are evaporation, centrifugal separation, and demineralization/filtration. Because they contain little or no organics and solids, the Group I wastes are processed by evaporation. The evaporator design provides for a flow that is sufficient to allow processing of 1,050 g/h (3,975 l/hr). This is sufficient capacity to process the entire weekly Group I liquid waste volume in slightly more than 25 hours. Because they contain organics and solids, but little or no activity, the Group II and III waste streams are processed by centrifugal separator. The separator is capable of processing approximately 1,300 g/h (4,921 l/hr). This is sufficient capacity to process the entire weekly Group II and Group III liquid waste volume in 63 hours. The demineralizer is capable of processing approximately 2,400 g/h (9,085 l/hr) of liquid waste. This is sufficient to process the combined weekly volume of the Group I, II, and III waste streams in about 40 hours.

Both the Liquid Waste Storage System and the Liquid Waste Processing System are located entirely within the radioactive waste processing building. Interfacing system piping delivers influent liquid wastes that originate in the plant drains with potential to contain liquid radioactive waste. [Table 3.5-11](#) lists the storage capacity for each of the liquid waste collection and process tanks. [Table 3.5-12](#) provides expected process rates for components in the waste processing system. [Table 3.5-13](#) provides the flow rates and activity for each main grouping of liquid radioactive waste.

Coolant Treatment System

Normal operating modes of the Coolant Treatment System purify and recycle reactor coolant and separate boron for reuse. However, the control of tritium levels in the Reactor Coolant System necessitates the periodic discharge of reactor coolant letdown after processing by the Coolant Treatment System for the removal of boron and the degasification of noble gas activity. The volume of processed reactor coolant to be discharged from the plant is administratively controlled to maintain tritium concentrations in the coolant system within a selected range. This processed liquid is discharged to the Liquid Waste Storage System and the Liquid Waste Processing System before being released to the environment instead of being recycled. This treatment option is performed in order to maintain reactor coolant tritium levels such that personnel exposures during containment entry during both power operation and refueling shutdowns is not unduly limited.

Steam Generator Blowdown Processing System

Control of the steam generator secondary side liquid chemistry is achieved by blowdown and demineralized water makeup. The radioactivity content of this blowdown is dependent on reactor coolant radioactivity levels and the primary-to-secondary leakage rate. The estimated average primary-to-secondary steam generator tube leakage reflected in the GALE source term estimates is 75 lb per day (34 kg per day) (NRC, 1985). The steam generator secondary side blowdown rate associated with this leakage level is 218,400 lbm/hr (99,065 kgm/hr) total for all four steam generators.

The blowdown liquid is routed to the blowdown flash tank. As a result of pressure reduction, approximately 29% of the liquid mass flashes to steam. The steam-water mixture is separated in the flash tank. The overhead steam is directed to the deaerator (also called the feedwater tank). The remaining 71% of the flash tank inlet mass (liquid condensate) is routed through two

stages of letdown cooling before being processed by the Steam Generator Blowdown Demineralizer System located in the Nuclear Auxiliary Building for cleanup and return to the turbine condenser.

Secondary System Condensate Leakage Collection and Discharge

With radioactivity present in the secondary sides of the steam generators, moisture carryover brings some radioactivity to the remainder of the secondary coolant system. Consequently, leakage of secondary system condensate forms a potential radioactive liquid release source. The amount of radioactivity reaching condensate leakage points is minimized by the high quality of the steam exiting the steam generators so that no processing of condensate leakage before discharge is required. The estimated average volumetric generation rate of this liquid is 5 gpm (19 lpm) at main steam activity. This liquid is discharged from the plant unprocessed, which results in an estimated annual release of 0.00033 curies/yr (1.2E+7 Bq/yr), not including tritium. A central collection point within turbine building is provided to allow sampling and analysis for radioactivity content. The liquid is released from the plant via a monitored pathway (with alarm and trip function on detected high radioactivity) {to the Missouri River}.

It is assumed, per the GALE code, that the turbine building floor drains will collect leakage of 7,200 gpd (27,255 lpd) at main steam activity (NRC, 1985). The leakage collected in the floor drain sump is directly discharged to the environment without treatment. Should monitors detect excess radiation in the sump, the sump is isolated for evaluation.

3.5.2.2 Liquid Release to the Environment

The radioactivity inputs to the liquid waste system release calculations are provided in [Table 3.5-14](#). The expected annual liquid release source terms based on the GALE code model of the U.S. EPR are summarized in [Table 3.5-7](#).

Releases from Anticipated Operational Occurrences

Annual average radioactivity releases through liquid effluents are summarized in [Table 3.5-15](#). The additional unplanned liquid release due to anticipated operational occurrences is estimated to be 0.16 Ci/year for the U.S. EPR design based on reactor operating data presented in NUREG 0017 (NRC, 1985). These releases were evaluated to determine the frequency and extent of unplanned liquid release and are assumed to have the same isotopic distributions for the calculated source term of the liquid wastes. The total releases from the anticipated operational occurrences are shown in [Table 3.5-16](#) and are included as part of the “total liquid release source term.”

Summary of Radioactive Liquid Release from Normal Operations

Discharge concentrations are listed in [Table 3.5-16](#) and are calculated using a {5,673 gpm (21,475 lpm)} discharge flow rate which will be the average flow rate. The above discharge concentrations are compared with effluent concentration limits given in Table 2, column 2 of 10CFR20, Appendix B (CFR, 2007d).

Due to the impracticality of removing tritium on the scale necessary, some tritium present in the reactor coolant system will be released to the environment during plant life time. The quantity of tritium released through the liquid pathway is based on the calculated volume of liquid released, excluding secondary system waste, with a primary coolant tritium concentration of 1 $\mu\text{Ci/ml}$ up to a maximum of 0.9 of the total quantity of tritium calculated to be available for release. It is assumed that the remainder of tritium produced is released as a gas from building ventilation exhaust systems. Hence, 1,660 curies (6.14E13 Bq) of tritium are expected to be released to the environment via liquid effluents from the U.S. EPR each year.

3.5.2.3 Liquid Waste System Cost-Benefit Analysis

In addition to meeting the numerical As Low As Reasonably Achievable (ALARA) design objective dose values for effluents released from a light water reactor as stipulated in 10CFR50, Appendix I (CFR, 2007a), the regulation also requires that plant designs include all items of reasonably demonstrated cleanup technology that when added to the liquid waste processing system sequentially and in order of diminishing cost-benefit return, can, at a favorable cost-benefit ratio, effect reductions in dose to the population reasonably expected to be within 50 miles (80 km) of the reactor.

{In assessing if there are any demonstrated technologies that could be added to the plant design at a favorable cost-benefit ratio, a bounding assessment was performed which demonstrates that there is insufficient collective dose available to be saved that would warrant additional equipment cost. Since the Missouri River is not used as a source of water to a municipal water system within 25 miles (40 km) upstream and 50 miles (80 km) downstream of the site and is not used for agricultural irrigation, the liquid exposure pathway is not significant and further analysis is unnecessary.}

It should be noted that even though not warranted on a population dose savings basis, a vendor supplied waste demineralizer subsystem skid has been added to the plant design to provide plant operators greater flexibility to process waste liquids by different processes to best match waste stream characteristics, such as chemical form, with the waste process treatment method that best handles the waste from an economics standpoint.

3.5.3 RADIOACTIVE GASEOUS TREATMENT SYSTEMS

Radioactive gases (such as xenon, krypton and iodine) created as fission products during reactor operation can be released to the reactor coolant through fuel cladding defects along with hydrogen and oxygen that is generated by radiolytic decomposition of the reactor coolant. Since these gases are dissolved in the reactor coolant, they are transported to various systems in the plant by process fluid interchanges. Subsequent reactor coolant leakage releases a portion of these gases and any entrained particulate radioactivity to the ambient building atmosphere.

Fission product and radiolytic decomposition gases released from reactor coolant within the various process systems are handled by the Gaseous Waste Processing System. Radioactive gases or airborne particulates released to the ambient atmosphere in one of the buildings due to system leakage from the process system piping is managed by the combined operation of the Containment Ventilation System, Safeguards Building Controlled Area Ventilation System, Fuel Building Ventilation System, Nuclear Auxiliary Building Ventilation System, and Sampling Activity Monitoring Systems.

3.5.3.1 System Description and Operations

The Gaseous Waste Processing System and sources are provided in [Figure 3.5-5](#). The Gaseous Waste Processing System combines a quasi-closed loop purge section with a discharge path provided through a carbon bed delay section. The purge section recycles the majority of purge gas after it has been processed. This limits the system demand for makeup purge gas, and also limits the amount of gas that must be discharged through the delay section to the environment.

The purge section includes waste gas compressors, purge gas pre-driers, several purge gas reducing stations, purge gas supply piping to tanks in a number of interfacing systems, purge gas return piping from those tanks, purge gas driers, recombiners, and gas coolers. The purge

section also includes a gas supply subsystem, gas measuring subsystems, and compressor sealing subsystems. The purge gas stream consists of nitrogen with small quantities of hydrogen and oxygen, and trace quantities of noble gas fission products.

The carbon bed delay section includes a gel drier, delay beds, a gas filter, and a discharge gas reducing station. The delay section discharges processed gaseous waste to the Nuclear Auxiliary Building Ventilation System for release to the environment via the ventilation exhaust stack.

All the components of the Gaseous Waste Processing System and the majority of the components of connected systems are located in the Nuclear Auxiliary Building. However, there are some connected components that are continually swept by gaseous waste processing purge gas flow that are located in other buildings. The volume control tank and two of seven nuclear island drain and vent systems primary effluent tanks are located in the Fuel Building. Four more nuclear island drain and vent systems primary effluent tanks are located in the four safeguard buildings. The pressurizer relief tank and the reactor coolant drain tank are located in the Reactor Building. Gaseous Waste Processing System piping is routed among the buildings.

The Gaseous Waste Processing System is designed to operate continuously during normal plant operation. For the majority of this time, with the plant operating at full power, the Gaseous Waste Processing System will operate in a steady state mode, with a constant flow rate (0.19 lbm/sec) (86 gm/sec) for two compressors running), through the purge section, and a small (0.00015 lbm/sec (0.068 gm/sec)), constant discharge rate from the delay section. [Figure 3.5-6](#) depicts the Gaseous Waste Treatment System. The U.S. EPR FSAR Section 11.3 describes the individual components and design details of the Gaseous Waste Management System.

Normal Operation – Purge Section

The circulation of purge gas is maintained by the operation of one or both waste gas compressors. The Gaseous Waste Processing System operates at positive pressures from the waste gas compressors to the reducing stations and the volume control tank, and at sub-atmospheric pressure downstream of the reducing stations through the various connected tanks and the gaseous radwaste processing equipment that returns the purge flow to the suction of the waste gas compressor.

Radioactive fission product gases are collected from the pressurizer relief tank, the reactor coolant drain tank, and the volume control tank. The primary influent source is expected to be the Coolant Degasification System, which extracts both hydrogen and fission product gases from the reactor coolant on a continuous basis. The other major source of influent to the Gaseous Waste Processing System is the reactor coolant drain tank.

Gaseous Waste Processing System purge gas drawn from the connected components is routed through the gaseous radwaste processing equipment. First, the gas drier treats the returning purge gas. The gas drier uses a cooling process to reduce the moisture content in the purge gas.

The recombiner uses a catalytic process at elevated temperature to recombine the free hydrogen and oxygen entrained in the purge gas stream.

The gas cooler cools the purge gas stream at the recombiner outlet. A filter assures that no particulates are carried forward to the waste gas compressor.

The waste gas compressor compresses the incoming purge gas flow, and discharges to the sealing liquid tank.

The sealing liquid tank separates the gaseous and liquid phases from each other. The purge gas leaving the sealing liquid tank is routed to the pre-drier. The pre-drier cools the purge gas to reduce its moisture content by condensation.

The Gaseous Waste Processing System piping branches downstream of the pre-drier, dividing the purge gas flow. One branch supplies purge gas to the pressurizer relief tank and the reactor coolant drain tank. A second branch supplies purge gas flow to the volume control tank. The third branch connects to the delay section.

The purge gas flow in the third branch is joined by the purge gas discharged from the volume control tank, and is then distributed to the four parallel branches. These four paths purge radioactive fission product gases from the coolant supply and storage system tanks, the reactor boron and water makeup system, the coolant purification system, the coolant treatment system, the coolant degasification system, the various nuclear island vent and drain system primary effluent tanks (in the Safeguards Buildings, the Fuel Building, and the Nuclear Auxiliary Building), and the Nuclear Sampling System active liquid samples subsystem.

Normal Operation – Delay Section

Only a small quantity of purge flow is sent to the delay beds under normal operating conditions. The remaining quantity is recycled.

The delay beds retain the radioactive fission product gases that enter the delay section. These gases (e.g. Xenon and Krypton) are dynamically adsorbed by the activated charcoal media in the delay beds, which provides the residence times required for natural decay. For normal operations, the Xenon and Krypton dynamic adsorption coefficient are $70 \text{ cm}^3/\text{gm}$ ($1,940 \text{ in}^3/\text{lb}$) for Krypton and $1,160 \text{ cm}^3/\text{gm}$ ($32,110 \text{ in}^3/\text{lb}$) for Xenon, respectively. This equates to an estimated holdup time for Xenon and Krypton of 27.7 days for Xenon and 40 hours for Krypton, respectively.

The delay beds consist of three vertical pressure vessels connected in series which are maintained at a constant positive pressure to improve the adsorption of waste gases in the activated charcoal media. Two moisture sensors are configured in parallel upstream of the delay beds to provide warning and protective interlock signals if the moisture content of waste gas entering the delay beds exceeds acceptable levels. A radiation sensor is also located upstream of the delay beds to monitor influent activity levels. Two pressure sensors monitor pressure upstream of the delay beds to provide warning signals for high or low operating pressure conditions, and to provide protective interlock signals.

Surge Gas Operation

Operations that transfer large quantities of primary coolant in the systems purged by the Gaseous Waste Processing System automatically place the system into surge gas operation mode. The Gaseous Waste Processing System operates in surge gas mode primarily during plant startup or shutdown.

Surge Gas Operation – Purge Section

Operation of the Gaseous Waste Processing System purge section is not significantly altered by plant operating mode. Purge flow through the components connected to the Gaseous Waste Processing System continues as in normal operating conditions.

Surge Gas Operation – Delay Section

During conditions of excess gas generation, the flow volume to the delay section automatically increases. This increased flow volume is automatically sensed and shifts the system to surge gas operation mode. Surge gas operation mode automatically stops waste gas releases from the Gaseous Waste Processing System via the Nuclear Auxiliary Building Ventilation System until the system is manually reset.

The capacity of the delay section adapts to the increased flow rate during surge gas operation mode because surge gas mode elevates delay section pressure. Higher pressure increases the storage capacity of the delay section and improves the adsorption capabilities of the activated charcoal.

The delay section maintains the required residence time for natural decay of the fission product gases during surge gas operation mode by virtue of the increased capacity arising from the elevated operating pressure.

Surge gas operation continues for a predetermined period of time sufficient to achieve the required residence times for the fission product gases. When this time period expires, delay section pressure reduction is manually initiated and gradually reduces the pressure in the delay section.

Steam Generator Blowdown Flash Tank Venting

During normal operations, the blowdown liquid is routed to the blowdown flash tank. As a result of pressure reduction, a portion of the liquid mass flashes to steam. The steam-water mixture is separated in the flash tank, with the overhead steam directed to the deaerator (also called the feedwater tank). Non-condensable gases from the deaerator are sent to the main turbine condensers and are removed by the main condenser evacuation system for release to the plant stack.

Radiation sensors on the Steam Generator Blowdown Sampling System continually monitor blowdown activity for indications of a steam generator tube leaks or rupture. If indications of tube rupture are detected, the affected steam generator is automatically isolated from the blowdown flash tank in the Steam Generator Blowdown System. Eventually, after a controlled plant shutdown and cooldown has been completed, the affected steam generator may be drained to the nuclear island vents and drains system, which is one of four normal destinations for steam generator draining (plant drains, clean drains, the condenser and the nuclear island vents and drains).

Main Condenser Evacuation System

The Main Condenser Evacuation System is designed to establish and maintain a vacuum in the condenser during startup, cooldown and normal operation by the use of mechanical vacuum pumps. Vacuum pumps remove air and non-condensable gases from the condenser and connected steam side systems and pass the steam and the air mixture through moisture separators. As a result of compression, the steam component condenses while the extracted air is vented through the vent system into the ventilation system of the Nuclear Auxiliary Building Ventilation System and released to the environment via the plant stack. The activity of the exhausted air is monitored.

Ventilation Filter Systems

Effluent discharged from the delay section of the Gaseous Waste Processing System is directed to the filtration section of the Nuclear Auxiliary Building Ventilation System. Exhaust air from

the containment purge “full flow purge” (used only during plant outage periods), along with exhaust air from the Safeguards Building Controlled Area Ventilation, Fuel Pool Building Ventilation, and Nuclear Auxiliary Building Ventilation Systems, is also processed by the filtration section of the Nuclear Auxiliary Building Ventilation System before release from the stack. The ventilation flow paths (including containment “low flow purge” and “full flow purge”) continuously exhaust to the Nuclear Auxiliary Building Ventilation System. Each exhaust flow path has a pre-filter and a HEPA filter. The filtered air is sent to the common exhaust plenum and removed via the stack. If radiation sensors in any of the rooms within the Nuclear Auxiliary Building, Reactor Building, Fuel Building, Safeguards Buildings, or the stack detect elevated radioactivity levels in exhaust gases, the associated flow paths are redirected to iodine-adsorbent activated charcoal delay beds and the filtered air is sent to the stack. The charcoal beds each have a downstream HEPA filter to remove potentially radioactive charcoal dust and particulates. The ventilation systems are shown in [Figure 3.5-6](#) through [Figure 3.5-12](#).

3.5.3.2 Gaseous Release to the Environment

All gaseous effluents are released at the top of the plant stack. The stack height is approximately 197 ft (60 m) above plant grade, or about 6.56 ft (2 m) above the height of the adjacent Reactor Building. The normal stack flow rate is conservatively estimated at 260,000 cfm (7,362 m³/min) with no credit for thermal buoyancy of the exit gas assumed (ambient temperature) and the low flow purge system assumed to not be operating. The stack diameter is 12.5 ft (3.8 m). The releases of radioactive effluent to the plant stack include contributions from:

- ◆ Gaseous Waste Processing System discharges via the carbon delay beds for noble gas holdup and decay.
- ◆ Containment purge ventilation discharges.
- ◆ Ventilation discharges from (1) the four Safeguards and Access Building controlled areas, (2) the Fuel Pool Building, (3) the Radwaste Building and (4) the Nuclear Auxiliary Building.
- ◆ Main Condenser air evacuation exhaust.

The annual average airborne releases of radionuclides from the plant were determined using the PWR GALE code (NRC, 1985). The GALE code models releases using realistic source terms derived from the experiences of many operating reactors, field and laboratory tests, and plant-specific design considerations incorporated to reduce the quantity of radioactive materials that may be released to the environment during normal operation, including anticipated operational occurrences. The code input values used in the analysis are provided in Section 3.5.1. The expected annual releases from the plant are presented in [Table 3.5-8](#) and [Table 3.5-17](#).

3.5.3.3 Gaseous Waste System Cost-Benefit Analysis

As with the liquid waste processing systems, the ALARA design objective dose values for effluents released from a light water reactor as stipulated in 10 CFR 50, Appendix I (CFR, 2007a), the regulation also requires that plant designs include all items of reasonably demonstrated cleanup technology that when added to the gaseous waste processing system sequentially and in order of diminishing cost-benefit return, can, at a favorable cost-benefit ratio, effect reductions in dose to the population reasonably expected to be within 50 miles (80 km) of the reactor. Values of \$2,000 per person-rem and \$2,000 per person-thyroid-rem are used as a

favorable cost benefit threshold based on NRC NUREG-1530 (NRC, 1995). The source term for each equipment configuration option was generated using the same GALE code as described in Section 3.5.1 along with the same plant specific parameters modified only to accommodate the changes in waste stream decontamination factor afforded by the design options simulated.

For the U.S. EPR, the dose reduction effects for the sequential addition of the next logical gaseous waste processing component (i.e., addition of an additional charcoal delay bed to the waste gas holdup subsystem) results in a reduction in the 50 miles (80 km) population total body exposure of 0.03 person-rem (0.0003 person-sievert). Section 5.4 describes the population dose calculation for both the base case augmented charcoal delay bed holdup system for processing gaseous waste. Table 3.5-18 illustrates the relative population dose associated with both base equipment configuration and that associated with the augmented holdup system. Table 3.5-19 compares the estimated total body and thyroid dose reduction or savings achieved for the addition of the extra delay bed along with a conservative estimated cost for the purchase. Operating and maintenance cost associated with this passive subsystem is negligible. The cost basis for the equipment option is taken from Regulatory Guide 1.110 (NRC, 1976a) and reported in 1975 non-escalated dollars which provides a conservatively low estimate of the equipment cost to today's dollars. The site area population within 50 miles (80 km) is based a projected population in 2080, over 60 years from the estimated start of plant operations. Using the population at the end of plant life is conservative in that it maximizes the collective dose from plant effluents.

For both the total body and thyroid dose reduction, Table 3.5-19 illustrates that the favorable benefit in reduced dose associated with the additional charcoal delay bed had a dollar equivalent benefit value of {\$240.00.} However, the estimated cost to purchase this equipment was approximately \$67,000, thereby resulting in a total body effective benefit to cost ratio of less than 1.0 (not justified on an ALARA basis of dose savings to the public).

The total gas release from the plant is made up of several sources, of which the charcoal delay bed subsystem provides treatment for the process gas from primary side reactor system components only. As a consequent, assuming that the process gas stream release has a zero value does not result in a zero dose to the population. Ventilation system exhaust from the reactor building and other controlled area buildings, along with any secondary side process gas releases if primary to secondary leaks occur also contribute to the total release. Because these sources are distributed throughout the plant, no single system can be added that effectively reduces all sources of gas releases. However, beyond the waste gas processing that is accomplished by the charcoal delay beds, the existing controlled area ventilation systems already provide for HEPA filtration, and as needed charcoal filtration, to the major sources of gas released to the environment. As a result, no other treatment options not in use are available that could treat a significant fraction of the total release at a favorable cost to that shown for the charcoal delay bed.

3.5.4 SOLID RADIOACTIVE WASTE SYSTEM

The Solid Waste Management System serves to collect, treat and store the solid radioactive wastes produced throughout the plant. There are several types of wet solid waste produced in the plant. These include spent resins, filter and centrifuge sludge's, sludge from the storage tank bottoms, and evaporator concentrates. There are also dry wastes such as paper, cloth, wood, plastic, rubber, glass and metal components that are contaminated.

The solid system consists of three parts; the radioactive concentrates processing system, the solid waste processing system and the solid waste storage system. Figure 3.5-13 provides a flow diagram of the inputs and processes associated with the Solid Waste System.

The radioactive concentrates processing system serves to process radioactive concentrates into a monolithic salt block by drying liquid radioactive waste from different systems. The liquid waste treated includes the concentrates left after the liquid waste has been treated in the evaporator of the Liquid Waste Processing System. It also treats the radioactive sludge from the liquid waste storage tanks of the Liquid Waste Storage System. The spent ion exchange resins from the Coolant Purification System or liquid waste processing demineralizer package are also sent to the concentrates processing system, after they have been stored for a period of time, to be processed with the other radioactive concentrates.

The Dry Solid Waste Processing System serves to collect and process the solid or DAW produced throughout the plant. This waste can include materials such as plastics, paper, clothing, glass, rubber, wood and metal. The waste is separated and processed separately depending upon size, activity and physical/chemical conditions. In-plant capability to separate, shred and compact DAW waste materials into disposable containers is provided. Alternately, DAW may be shipped in the "as collected" form to an offsite licensed processor for volume reduction treatment and final packaging and shipment to a disposal facility.

The Solid Waste Storage System serves to store the solid waste mentioned above both before and after processing. The untreated solid waste is stored near its producing area until it is ready to be processed. Wet solid waste shall be stored separately from DAW to avoid cross contamination. Once treated, the solid waste, along with the treated concentrates, is stored in one of two areas. One area is a tubular shaft storage area for the high activity drums and the other is a temporary storage area for low to medium activity drums. Once the activity has reduced to a low enough level, the drums are transported to an offsite repository for final disposal.

3.5.4.1 Radioactive Concentrates Processing System

The Radioactive Concentrates Processing System is used to produce a monolithic salt block inside a disposal drum by drying high solids content liquids from different systems.

Evaporator concentrates from the concentrate tanks and contaminated sludge from the liquid waste storage tanks of the liquid waste storage system are transferred to the concentrate buffer tank. These wastes are mixed, sampled and analyzed for proper pretreatment before leaving the concentrate buffer tank.

Spent resins are stored in the resin waste tanks of the coolant purification system for an extended length of time to allow short lived activity to decay away. When processed, these resins are transferred into the resin proportioning tank. Depending upon activity levels in the resin, a portion of the resins is transferred into the concentrate buffer tank with liquid waste where it is mixed to control the overall waste radioactivity concentration. Spent resin from the Liquid Waste Storage and Processing System demineralizer/ultra filtration skid may be sent directly to high integrity containers (HICs) for in-container dewatering or transferred to the concentrate buffer tank. This demineralizer system produces spent resins as well as a small amount of solid waste from the back flush of the ultra filtration system.

From the concentrate buffer tank, the liquid waste can be transferred into a storage drum in one of three drum drying stations where the water content is evaporated off. Alternately, resin slurries can be transferred to HICs to be dewatered and sent to disposal. In the drum drying station, a seal is established on the drum and a vacuum established. Then the heaters are energized to evaporate the water from the drum. The vacuum in the drum allows a lower required heating temperature to boil off the water. The water vapor is condensed and collected and volume counted before it is drained to the condensate collection tank. The air and

non-condensable gases are routed to the Radioactive Waste Building Ventilation System for processing. After most of the liquid has been evaporated out of the drum, it is refilled with more waste from the concentrate buffer tank and the drying process is re-initiated. This filling and evaporation process is repeated until the drum is filled with a solid precipitated dry activity waste product. The solid drum drying process reduces the moisture content of the solid block to the level required for disposal at an offsite repository.

Once the residual moisture has been reached, the shell and bottom heaters are turned off and disengaged from the drum. After a set time, the vacuum unit is shut down and the drum drying station is directly vented to the Radioactive Waste Building Ventilation System. While the drum is still connected to the Radioactive Waste Building Ventilation System, the product is allowed to cool to a less than 212°F (100°C).

The whole drying process is performed automatically which means that the system can operate 24 hours a day unattended. Only during the drum exchange process does an operator have to be at the control panel to perform the different drum exchange steps. This process is done remotely.

Once the product cools down, the drum is lowered and transferred to the pickup position outside the filling station. In this position the drum is picked up by the drum handling device, lowered to the pickup position conveyor (part of the drum transfer device (DTD)). The DTD transfers the filled uncapped drum to the sampling position for dried waste for taking samples from the content of the drum as far as defined in a semi-automatic mode (the sample is taken automatically while insertion and removal of the shielded drill is performed manually). In the next step the drum is routed to the drum capping device for capping the filled drum.

The drum capping device operates automatically. After the drum reaches the capping position, a start button is pressed and a lid is automatically placed on the drum. The drum is automatically capped and once complete, a release signal allows the further transport of the drum to the drum input/output position. From the input/output position, the drum is moved by the drum store crane to the drum measuring device.

In the drum measuring device, the weight, the dose rate and the main radionuclides of the drum content are measured. A gamma spectroscopy measurement with a Ge-detector is used to determine radionuclides and their activity. The drum is arranged on a turntable which is slowly rotated during the measurement process. The drum measuring device operates automatically. Once the measuring process is complete, the drum is picked up by the drum store crane and moved to the drum store for storage.

3.5.4.2 Solid Waste Processing System

DAW is collected in suitable containers such as plastic bags, drums or bins that are placed in various locations throughout the controlled areas of the plant at such points as step-off pads at exits from contaminated areas. Once full, these collection bags or bins are sent to the solid waste processing area for sorting, compaction if suitable, and final packaging for temporary storage in-plant or shipment to a licensed disposal facility offsite or licensed waste processor for additional processing before final disposal.

The in-plant treatment facilities include a sorting box for sorting waste into compressible and non-compressible fractions, a drying box for drying of wet materials that might have greater than incidental moisture before further treatment, a shredder for treating large bulky combustible and compressible waste before being compacted, and a compactor for in-drum

compaction of compressible waste. Filter cartridges are loaded in high integrity containers with other wastes for disposal.

3.5.4.3 Solid Waste Storage System

The different properties, sizes, materials and activity of the solid radioactive waste are considered while collecting the waste in different containers so as to simplify both handling and storage of the waste in the plant and its transport.

Various storage areas are provided in the Radioactive Waste Building for the different types of solid waste and contaminated components.

The system is able to handle and store the waste generated in the different controlled areas of the plant independent from the plant operating conditions. Storage space is provided for collected untreated waste waiting for treatment. Additional space is provided for treated and packaged low activity waste, such as DAW, as well as higher activity waste in a tubular shaft storage arrangement that provides shielding for operating staff. The tubular shaft store is part of the permanent building structure formed in the shape of tubes. The higher activity waste includes items such as the radioactive concentrates treated in the radioactive concentrates processing system and the spent filter cartridges. The drum store is located in the Radioactive Waste Processing Building in an area used for temporary storage of low level radioactive waste treated by the solid waste processing system. The drums can be stacked a maximum of five drums high to optimize the available storage space. The drums are stored for a sufficient time to allow the short lived radionuclides to decay thereby reducing the radiation levels to keep radiation exposures ALARA.

The drums containing the spent filter cartridges are placed in a shielded cask and brought to the drum transfer station. Once at the drum transfer station, the vehicle entrance crane lifts the lid off the cask and the drum store crane takes the drum to the tubular shaft store for storage. The lid is then placed back on the cask and the cask is returned to the Nuclear Auxiliary Building to the filter changing area.

The drums containing the medium activity waste such as spent filter cartridges, spent resins and the concentrates wastes from the radioactive concentrates processing system are transported to a final repository after being temporarily stored in the tubular shaft storage area. This is done by using the drum storage crane to remove the drums from the tubular shaft and place them in the drum transfer position. They are placed in a shielding cask and lifted to the vehicle entrance area by the vehicle entrance crane. Once in the vehicle entrance area, each drum is removed from the cask and placed into an approved shipping container to be moved to the offsite facility.

3.5.4.4 Expected Volumes

The volume of solid radioactive waste estimated to be generated by the U.S. EPR is approximately 7,933 ft³ (224.6 m³) per year (including compressible waste). [Table 3.5-10](#) delineates the expected annual volume by waste type. For liquid waste streams, the maximum volume reduction is achieved by converting liquid waste concentrates to dried salt deposits in the waste drum drying subsystem. Final drum drying is expected to achieve a volume reduction (VR) factor of about five over the concentrate stream. After dewatering, spent demineralizer resins are assumed to have the same volume as the initial resin volume used (i.e., no VR). [Table 3.5-10](#) presents the final volume of processed concentrates ready for storage or shipment to a disposal facility. For DAW, [Table 3.5-10](#) indicates the “as collected” volumes and assumes that no onsite volume reduction to these waste are applied. These materials are

expected to be sent to an off site licensed waste processor for sorting and treatment for volume reduction before shipment to a disposal facility. If onsite compaction of compressible DAW is performed, a VR factor of 5 or more is expected assuming:

- a. Each non-regenerable ion-exchanger is changed annually; and
- b. Approximately 15 spent filter cartridges from all process systems combined are generated annually with a package volume of approximately 120 ft³ (3.40 m³) (one filter element per disposal drum).

Curie content associated with this waste volume is also delineated in [Table 3.5-10](#). The radioactive concentrations vary considerably depending upon plant operating conditions. However, radiation monitoring (and related interlocks) within the solidification system ensure that all shipments will comply with federal and state regulations (i.e., radiation levels and gross weight of shipping vehicle).

3.5.4.5 Solid Release to the Environment

Solid wastes will be shipped from the site for burial at a NRC licensed burial site. The containers used for solid waste shipments will meet the requirements of 49 CFR Parts 170 through 189 (Department of Transportation Radioactivity Material Regulations) (CFR, 2007e), and 10 CFR Part 71 (Packaging of Radioactive Materials for Transport) (CFR, 2007f). [Table 3.5-10](#) summarizes the annual total solid radioactive waste generated at {Callaway Plant Unit 2}.

3.5.4.6 {Independent Spent Fuel Storage Installation

AmerenUE does not have an on-site interim dry spent fuel storage facility (often referred to as an ISFSI) at the Callaway Plant site at this time and has no immediate plans to install such a facility. The need for additional storage capacity for spent fuel being generated by the operations of Callaway Plant Units 1 and 2 is not anticipated until after 2020. An Independent Spent Fuel Storage Installation will be constructed at an appropriate location on the Callaway site in advance of the need for dry storage of spent fuel.}

3.5.5 PROCESS AND EFFLUENT MONITORING

For routine operations, the process and effluent radiological monitoring and sampling systems monitor, record and (for certain subsystems) control the release of radioactive materials that may be generated during normal operation, including anticipated operational occurrences.

The process and effluent radiological monitoring systems consist of radiation detectors connected to local microprocessors. Each microprocessor processes the detector signal in digital form, computes average radioactivity levels, stores data, performs alarm or control functions, and transmits the digital signal to one of the control room information and control systems. Monitoring systems alarm when setpoint limits are exceeded and if the system becomes inoperable. Alarms are indicated both locally and in the control room.

For gaseous waste, all compartment ventilation exhaust air from controlled areas (i.e., Reactor Building, Fuel Building, Safeguard Buildings, Waste Building and Nuclear Auxiliary Building) and the gaseous waste system exhaust air is discharged to and monitored in the plant vent stack. Effluent sampling systems also monitor the Reactor Building, Fuel Building, the Nuclear Auxiliary Building and the mechanical area of the Safeguard Buildings, as well as the vent stack. Samples are also taken and monitored from the exhaust air of the Access Building and the Waste Building. These two buildings are not part of the controlled area and do not vent to the vent stack. Sampling of these two buildings provides assurance that an inadvertent release of

radioactivity to the environment will be monitored. Gaseous effluent monitoring systems utilized in the U.S. EPR are discussed in the following sections.

The liquid radioactive waste effluent monitoring system measures the concentration of radioactive materials in liquids released to the environment to ensure that radionuclide concentration limits specified in 10 CFR 20 are complied with. Process line monitors provide operating personnel indication of system performance and the existence of leaks from contaminated systems to clean systems or subsystems of lower expected radioactivity.

The process and effluent monitors are discussed below by the plant system that is being monitored. [Table 3.5-20](#) has been arranged by the radioisotopes monitored to make it more convenient to compare monitors that perform a similar function. The monitors in [Table 3.5-20](#) are grouped by categories for noble gas effluent, gaseous iodine and aerosol (halogen and particulate) effluent, process monitoring (area radiation levels, personnel and equipment contamination, system leakage from the primary side to nuclear island buildings or secondary systems), liquid effluent, and airborne radiation levels.

3.5.5.1 Vent Stack

Vent stack gaseous effluent monitoring is accomplished by the use of continuously operating measurement devices for noble gas, aerosol, and iodine. Samples are also collected that may be utilized for laboratory determination of tritium. Two independent systems provide system redundancy and permits maintenance on one train while continually monitoring effluents with the other train. Each sampling system consists of a sampling nozzle array designed to provide a representative sample, two 100% capacity rotary sampling pumps, and specially designed interconnecting tubing running between a sampling nozzle array, sampling pumps, and radiation monitoring instrumentation. Gaseous samples exiting the monitoring instrumentation are returned to the vent stack. The vent stack effluent monitoring system has the following general characteristics:

- ◆ Noble gas activity is monitored with beta-sensitive detectors. The gross output of the monitor is periodically normalized to the radionuclide composition by performing a gamma-spectroscopic analysis on a representative grab sample.
- ◆ Aerosol activity is monitored with the use of a particulate filter through which sample flow is continuously maintained. Aerosol particles are removed by the filter, which is continuously monitored by a gamma-sensitive detector.
- ◆ Iodine activity is monitored with the use of a dual filter for organic and inorganic iodine. Each filter is continuously monitored by a gamma-sensitive detector.

For both particulate and iodine monitoring, the gross outputs of the monitors are normalized by laboratory analysis of a duplicate set of filters installed in parallel with the primary ones. The vent stack gaseous effluent monitoring system does not perform any automatic actions. The system monitors, records, and alarms in the control room in the event that monitored radiation levels increase beyond specified setpoints. Measurement ranges of noble gas, aerosol, and iodine monitors are shown in [Table 3.5-20](#).

3.5.5.2 Gaseous Waste Carbon Delay Beds

The gaseous waste delay bed process stream is continuously monitored prior to waste flow being directed to the plant vent stack. One gamma-sensitive radiation detector is located up-stream of the delay beds and one beta-sensitive radiation detector downstream of the beds

outlet. The upstream detector provides plant personnel with an indication of the amount of radioactivity entering the system. The downstream detector is a beta-sensitive instrument, as Krypton-85 generally forms the main constituent (about 95%) of the normal radioactive noble gas waste stream, and provides personnel a means to compare the reduction in radioactivity afforded by the delay bed system. The gaseous waste monitoring system provides control room and local indication and an alarm in the main control room terminates release to the plant vent stack by closing the discharge valve. Measuring ranges of the gaseous waste disposal radiation monitoring system are shown in [Table 3.5-20](#).

3.5.5.3 Condenser Air Removal Monitor

Non-condensable gases (air and noble gases) in the secondary system are continuously removed during operation by the condenser air removal system. These gases are exhausted to the vent stack. The function of the condenser air removal radiation monitor is to provide local and control room alarm in the event that noble gas radioactivity is detected in the secondary system. This would be an indication of a breach of fuel cladding, primary coolant boundary, or containment leak. Measuring ranges of the condenser air removal radiation monitoring system are shown in [Table 3.5-20](#). No automatic actions are initiated by this system.

3.5.5.4 Main Steam Radiation Monitoring System

Radioactivity releases from the reactor coolant system to the main steam system (nitrogen-16 (N-16), noble gases) can occur as a result of steam generator tube leakage. Radioactivity in the main steam system is monitored over a wide power range by four redundant measuring arrangements per steam line (16 total for the system). The gamma sensitive detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments. At low power levels, radioactivity will be detected in the main steam due to noble gas. At high power levels, the detectors detect the strong gamma from N-16. Shielding of detectors ensures that detectors on other main steam lines do not erroneously respond. The redundant measurement signals are processed, and provide alarm in the control room upon detection of radioactivity. The main steam radiation monitoring system is utilized in conjunction with the condenser air removal and steam generator blowdown radiation monitoring systems to identify a defective steam generator. The main steam radiation monitoring system does not initiate any automatic actions. Isolation of a defective steam generator is performed by manual operator actions. Measuring ranges of the main steam radiation monitoring system are shown in [Table 3.5-20](#).

3.5.5.5 Reactor Coolant Radiation Monitor and Sampling System

The noble gas radioactivity concentration of the primary coolant is monitored by monitoring the noble gas activity concentration in the gaseous volume flow prior to discharge to the Nuclear Sampling System degasifier. Monitoring is accomplished with a beta-sensitive measuring arrangement located immediately adjacent to the sampling line. This measuring point allows early detection of fuel element failures. The measurement range for this radiation monitoring system is shown in [Table 3.5-20](#).

3.5.5.6 Containment Atmosphere Radiation Monitor

The containment atmosphere radiation monitor measures the radioactive gaseous concentrations in the containment atmosphere. The containment atmosphere radiation monitor is a part of a reactor coolant pressure boundary leak detection system. The presence of gaseous radioactivity in the containment atmosphere is an indication of reactor coolant pressure boundary leakage. The measurement range for this radiation monitoring system is shown in [Table 3.5-20](#).

3.5.5.7 Containment Ventilation System Radiation Monitor

The containment ventilation system air filtration exhaust radiation monitor measures the concentration of radioactive materials in the containment purge exhaust air. The monitor provides an alarm in the main control room when the concentration of radioactive gases in the exhaust exceeds a predetermined setpoint.

The containment ventilation system air filtration exhaust radiation monitor is to be an inline monitor that uses a beta-sensitive scintillation detector. The measurement range for this radiation monitoring system is shown in [Table 3.5-20](#).

3.5.5.8 Liquid Waste Tank Monitors

The liquid radioactive waste monitoring system measures the concentration of radioactive materials in liquids released to the environment to ensure that radionuclide concentration limits specified in 10 CFR 20 and dose requirements specified in 10 CFR 50 are complied with. Liquid radioactive waste is discharged in batches. Prior to release of a liquid radioactive waste tank, a representative sample is taken and radiochemically analyzed. Results of this analysis are utilized in conjunction with dilution factor data to determine a release setpoint for the liquid waste monitoring system. Two continuously operating radiation sensors monitor the release line from the tanks. Release is automatically terminated if a set limit is exceeded or if the monitoring system is inoperable. Measurement ranges of the liquid radioactive waste monitoring system are shown in [Table 3.5-20](#).

3.5.5.9 Primary Component Cooling Liquid Monitors

The component cooling water system consists of a closed loop used to transfer heat from nuclear components to essential service water by the use of coolers (heat exchangers). The closed nature of this system constitutes a barrier against the release of radioactivity to the essential service water and thus to the environment in the event of leaks in the associated coolers.

The Component Cooling Water Radiation Monitoring System consists of two subsystems. The general component cooling water monitoring system utilizes gamma-sensitive radiation detectors in the four separate safety-related trains of the Component Cooling Water System to monitor the fluid for any escape of radioactivity from the various radioactivity containing systems that make up the nuclear components served by the component cooling circuits. This subsystem provides local and control alarm in the event that component cooling water gamma radiation levels exceed the monitor setpoint. No automatic actions are initiated by this subsystem.

The second subsystem consists of two gamma-sensitive radiation detectors upstream and two gamma-sensitive radiation detectors downstream on the component cooling water lines feeding/exiting the two high-pressure (HP) coolers of the Volume Control System. In the event of a leak in a HP cooler with high-activity primary coolant leaking into the component cooling water system, the radiation detector downstream of the defective cooler indicates the entry of radioactivity from this HP cooler into the component cooling loop that is running at the time. If the radioactivity exceeds a pre-determined limit, the defective HP cooler is automatically isolated, with associated control room alarm, on the primary side. This automatic action is suppressed if the limit value of the radiation detector at the inlet of the cooler has already triggered a high activity signal and during in-service inspection of the measuring points.

The component cooling water radiation monitoring system utilizes lead-shielded gamma-sensitive detectors installed adjacent to the piping. Measuring ranges of the Component Cooling Water Radiation Monitoring System are shown in [Table 3.5-20](#).

3.5.5.10 Steam Generator Blowdown Sample Monitors

The evaporation process within the steam generator results in the concentration of contaminants in the liquid phase. These contaminants include any non-gaseous radioactive substances that have entered the secondary system from the reactor coolant system as a result of tube leakage in a steam generator.

Sampling lines extract blowdown water from the individual blowdown lines for chemical analysis. These lines are located ahead of the primary isolation valve within the reactor containment. Flow is continuously extracted from each of these lines and fed to gamma activity measurement equipment. This allows each steam generator to be monitored separately and continuously for radioactivity carryover to the secondary side. These monitors enable the identification or verification of a steam generator tube leak. Measuring ranges of the Steam Generator Radiation Monitoring System are shown in [Table 3.5-20](#).

3.5.5.11 Turbine Building Drains Effluent Monitor

Turbine Building waste liquid is released from the plant via a monitored pathway. The effluent monitor provides alarm and trip function on the discharge flow if unexpected levels of radioactivity are detected in the release. Measuring ranges of the turbine building drains effluent monitor is shown in [Table 3.5-20](#).

3.5.6 REFERENCES

ANS, 1999. American National Standard – Radioactive Source Term for Normal Operations of Light Water Reactors, ANSI/ANS 18.1-1999, American Nuclear Society, 1999.

CFR, 2007a. Title 10, Code of Federal Regulations, Part 50.34a, Design Objectives for Equipment to Control Releases of Radioactive Material in Effluents – Nuclear Power Reactors, and Appendix I, Numerical Guides for Design Objectives and Limiting Conditions for Operation to Meet the Criterion “As Low as is Reasonably Achievable” for Radioactive Material in Light-Water-Cooled Nuclear Power Reactor Effluents, 2007.

CFR, 2007b. Title 40, Code of Federal Regulations, Part 190, Radiation Protection Programs, 2007.

CFR, 2007c. Title 10, Code of Federal Regulations, Part 20.1301, Dose Limits for Individual Members of the Public, Code of Federal Regulations, 2007.

CFR, 2007d. Title 10, Code of Federal Regulations, Part 20, Appendix B, Table 2, Radionuclides, Annual Limits on Intake (ALIs) and Derived Air Concentrations (DACs) of Radionuclides for Occupational Exposure; Effluent Concentrations; Concentrations for Release to Sewage, 2007.

NRC, 1985. Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Pressurized Water Reactors, PWR-GALE Code, NUREG-0017, Revision 1, Nuclear Regulatory Commission, April 1985.

NRC, 1995. Reassessment of NRC’s Dollar Per Person-Rem Conversion Factor Policy, NUREG-1530, Nuclear Regulatory Commission, 1995.

NRC, 1976a. Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Light-water Reactors, Regulatory Guide 1.112, Nuclear Regulatory Commission, 1976.

NRC, 1976b. Cost-Benefit Analysis for Radwaste Systems for Light Water-Cooled Nuclear Power Reactors, Regulatory Guide 1.110 (For Comment), Nuclear Regulatory Commission, March 1976.}

Table 3.5-1—Parameters Used in the Calculation of Fission Product Activity in Reactor (Design Basis)

	Parameter	Value
1.	Total core thermal power, including measurement uncertainty [MWt]	4,612
2.	Clad defects, as a percent of rated core thermal power being generated by rods with clad defects [%]	1.0
3.	Volume of reactor coolant system [ft ³] (m ³)	15,009 (425)
4.	Reactor coolant full power average temperature [°F](C)	594. (312.2)
5.	Purification flow rate (normal) [lbm/hr] (kg/hr)	79,400 (36,000)
6.	Effective cation demineralizer flow [gpm]	NA
7.	Fission product escape rate coefficients:	
	a. Noble gas isotopes [sec ⁻¹]	6.5E-08
	b. Br, Rb, I and Cs isotopes [sec ⁻¹]	1.3E-08
	c. Te isotopes [sec ⁻¹]	1.0E-09
	d. Mo isotopes [sec ⁻¹]	2.0E-09
	e. Sr and Ba isotopes [sec ⁻¹]	1.0E-11
	f. Y, Zr, Nb, Ru, Rh, La, Ce, Pr, Nd and Pm isotopes [sec ⁻¹]	1.6E-12
8.	Purification mixed bed demineralizer decontamination factors (fractions removed):	
	a. Noble gases and N-16, H-3	0.0
	b. Cs, Rb	0.5
	c. Anion / others	0.99 / 0.98
9.	Cation bed demineralizer decontamination factor	NA
10.	Degasifier noble gas stripping fractions	1.0

Table 3.5-2—Reactor Coolant Radionuclide Concentrations

(Page 1 of 3)

Radionuclide	Design Basis		Realistic Source Term (GALE)	
	μCi/gm	Bq/gm	μCi/gm	Bq/gm
Noble Gases⁽¹⁾				
Kr-83m	1.3E-01	4.7E+03		
Kr-85m*	5.7E-01	2.1E+04	1.8E-02	6.7E+02
Kr-85*	5.3E+00	2.0E+05	5.8E-01	2.1E+04
Kr-87*	3.3E-01	1.2E+04	2.0E-02	7.4E+02
Kr-88*	1.0E+00	3.8E+04	2.1E-02	7.8E+02
Kr-89	2.4E-02	9.0E+02		
Xe-131m*	1.1E+00	4.0E+04	8.8E-01	3.3E+04
Xe-133m*	1.4E+00	5.0E+04	8.2E-02	3.0E+03
Xe-133*	9.5E+01	3.5E+06	3.4E-02	1.3E+03
Xe-135m*	2.0E-01	7.2E+03	1.5E-01	5.6E+03
Xe-135*	3.4E+00	1.3E+05	7.7E-02	2.8E+03
Xe-137	4.6E-02	1.7E+03	3.9E-02	1.4E+03
Xe-138*	1.6E-01	6.1E+03	7.0E-02	2.6E+03
Halogens⁽²⁾				
Br-83	3.2E-02	1.2E+03		
Br-84	1.7E-02	6.2E+02	1.8E-02	6.7E+02
Br-85	2.0E-03	7.4E+01		
I-129	4.6E-08	1.7E-03		
I-130	5.0E-02	1.8E+03		
I-131*	7.4E-01	2.7E+04	1.3E-03	4.8E+01
I-132*	3.7E-01	1.4E+04	6.0E-02	2.2E+03
I-133*	1.3E+00	4.6E+04	1.9E-02	7.0E+02
I-134*	2.4E-01	8.9E+03	1.1E-01	4.1E+03
I-135*	7.9E-01	2.9E+04	4.8E-02	1.8E+03
Rubidium, Cesium⁽³⁾				
Rb-86m	1.2E-06	4.4E-02		
Rb-86	7.7E-03	2.8E+02		
Rb-88	4.1E+00	1.5E+05	2.2E-01	8.1E+03
Rb-89	1.9E-01	7.0E+03		
Cs-134	6.8E-01	2.5E+04	2.6E-05	9.6E-01
Cs-136	2.1E-01	7.8E+03	6.2E-04	2.3E+01
Cs-137	4.3E-01	1.6E+04	3.7E-05	1.4E+00
Cs-138	8.8E-01	3.3E+04		
Miscellaneous Nuclides⁽³⁾				
Sr-89	2.5E-03	9.4E+01	8.9E-05	3.3E+00
Sr-90	1.3E-04	4.8E+00	7.6E-06	2.8E-01
Sr-91	4.1E-03	1.5E+02	8.0E-04	3.0E+01
Sr-92	6.9E-04	2.6E+01		
Y-90	3.1E-05	1.1E+00		
Y-91M	2.1E-03	7.7E+01	5.0E-04	1.9E+01
Y-91	3.2E-04	1.2E+01	3.3E-06	1.2E-01
Y-92	5.6E-04	2.1E+01		
Y-93	2.6E-04	9.6E+00	3.5E-03	1.3E+02
Zr-95	3.7E-04	1.4E+01	2.5E-04	9.3E+00
Zr-97	2.7E-04	9.9E+00		
Nb-95	3.7E-04	1.4E+01	1.8E-04	6.7E+00

Table 3.5-2—Reactor Coolant Radionuclide Concentrations

(Page 2 of 3)

Radionuclide	Design Basis		Realistic Source Term (GALE)	
	μCi/gm	Bq/gm	μCi/gm	Bq/gm
Mo-99	4.3E-01	1.6E+04	4.3E-03	1.6E+02
Tc-99M	1.9E-01	6.9E+03	4.2E-03	1.6E+02
Ru-103	3.1E-04	1.1E+01	4.8E-03	1.8E+02
Ru-105	3.8E-04	1.4E+01		
Ru-106	1.1E-04	4.0E+00	5.7E-02	2.1E+03
Rh-103M	2.7E-04	1.0E+01		
Rh-105	1.8E-04	6.5E+00		
Rh-106	1.1E-04	4.0E+00	5.7E-02	2.1E+03
Ag-110M	7.9E-07	2.9E-02	8.2E-04	3.0E+01
Ag-110	4.4E-08	1.6E-03		
Sb-125	3.2E-06	1.2E-01		
Sb-127	2.0E-05	7.4E-01		
Sb-129	2.7E-05	1.0E+00		
Te-127M	1.8E-03	6.5E+01		
Te-127	8.7E-03	3.2E+02		
Te-129M	5.8E-03	2.1E+02	1.2E-04	4.4E+00
Te-129	9.6E-03	3.6E+02	2.6E-02	9.6E+02
Te-131M	1.5E-02	5.5E+02	1.1E-03	4.1E+01
Te-131	1.0E-02	3.8E+02	8.6E-03	3.2E+02
Te-132	1.6E-01	6.0E+03	1.1E-03	4.1E+01
Te-134	2.7E-02	9.9E+02		
Ba-137M	4.0E-01	1.5E+04	3.5E-05	1.3E+00
Ba-139	8.6E-02	3.2E+03		
Ba-140	2.5E-03	9.2E+01	8.4E-03	3.1E+02
La-140	6.4E-04	2.4E+01	1.7E-02	6.3E+02
La-141	2.1E-04	7.8E+00		
La-142	1.3E-04	4.6E+00		
Ce-141	3.5E-04	1.3E+01	9.6E-05	3.6E+00
Ce-143	3.0E-04	1.1E+01	2.0E-03	7.4E+01
Ce-144	2.8E-04	1.0E+01	2.5E-03	9.3E+01
Pr-143	3.5E-04	1.3E+01		
Pr-144	2.8E-04	1.0E+01	2.5E-03	9.3E+01
Nd-147	1.4E-04	5.1E+00		
Np-239	3.5E-03	1.3E+02	1.5E-03	5.6E+01
Pu-238	8.0E-07	2.9E-02		
Pu-239	8.1E-08	3.0E-03		
Pu-240	1.1E-07	4.1E-03		
Pu-241	2.8E-05	1.0E+00		
Am-241	3.1E-08	1.2E-03		
Cm-242	7.5E-06	2.8E-01		
Cm-244	4.1E-07	1.5E-02		
Activation Products⁽⁴⁾				
Na-24	3.7E-02	1.4E+03	3.7E-02	1.4E+03
Cr-51	2.0E-03	7.4E+01	2.0E-03	7.4E+01
Mn-54	1.0E-03	3.7E+01	1.0E-03	3.7E+01
Fe-55	7.6E-04	2.8E+01	7.6E-04	2.8E+01
Fe-59	1.9E-04	7.0E+00	1.9E-04	7.0E+00

Table 3.5-2—Reactor Coolant Radionuclide Concentrations

(Page 3 of 3)

Radionuclide	Design Basis		Realistic Source Term (GALE)	
	μCi/gm	Bq/gm	μCi/gm	Bq/gm
Co-58	2.9E-03	1.1E+02	2.9E-03	1.1E+02
Co-60	3.4E-04	1.3E+01	3.4E-04	1.3E+01
Zn-65	3.2E-04	1.2E+01	3.2E-04	1.2E+01
W-187	1.8E-03	6.7E+01	1.8E-03	6.7E+01
Tritium				
H-3	4.0	1.5E+05	1.0E+00	3.7E+04
Nitrogen				
N-16			4.0E+01	1.5E+06

Notes:

For Design Basis concentrations, the following conditions apply;

- (1) The noble gas concentrations are at the U.S. EPR Standard Technical Specification limit of 210 μCi/gm DE-Xe-133
 - (2) The halogen concentrations are at the U.S. EPR proposed Standard Technical Specification limit of 1 μCi/gm DE-I-131
 - (3) The concentrations for this group are based on 1.0% failed fuel fraction.
 - (4) The concentration of activation products based on ANSI/ANS-18.1-1999.
- * Radionuclide concentration controlled by proposed Technical Specifications

Table 3.5-3—Secondary Coolant Radionuclide Concentrations

(Page 1 of 3)

Radionuclide	Design Basis Liquid		Design Basis Steam		Realistic Source Term- Liquid ^(e)		Realistic Source Term- Steam ^(e)	
	μCi/gm	Bq/gm	μCi/gm	Bq/gm	μCi/gm	Bq/gm	μCi/gm	Bq/gm
Noble Gases^(a)								
Kr-83m	N/A	N/A	2.1E-05	7.9E-01				
Kr-85m	N/A	N/A	5.8E-06	2.1E-01	0.0E+00	0.0E+00	3.1E-09	1.1E-04
Kr-85	N/A	N/A	5.4E-05	2.0E+00	0.0E+00	0.0E+00	9.5E-08	3.5E-03
Kr-87	N/A	N/A	3.3E-06	1.2E-01	0.0E+00	0.0E+00	9.1E-09	3.4E-04
Kr-88	N/A	N/A	1.0E-05	3.8E-01	0.0E+00	0.0E+00	3.5E-09	1.3E-04
Kr-89	N/A	N/A	2.4E-07	9.0E-03				
Xe-131m	N/A	N/A	1.1E-05	4.0E-01	0.0E+00	0.0E+00	1.4E-07	5.3E-03
Xe-133m	N/A	N/A	1.5E-05	5.6E-01	0.0E+00	0.0E+00	1.4E-08	5.1E-04
Xe-133	N/A	N/A	9.7E-04	3.6E+01	0.0E+00	0.0E+00	5.6E-09	2.1E-04
Xe-135m	N/A	N/A	8.2E-04	3.0E+01	0.0E+00	0.0E+00	2.5E-08	9.1E-04
Xe-135	N/A	N/A	1.6E-04	6.0E+00	0.0E+00	0.0E+00	1.3E-08	4.7E-04
Xe-137	N/A	N/A	4.6E-07	1.7E-02	0.0E+00	0.0E+00	6.5E-09	2.4E-04
Xe-138	N/A	N/A	1.7E-06	6.1E-02	0.0E+00	0.0E+00	1.2E-08	4.4E-04
Halogens^(b)								
Br-83	1.6E-03	6.0E+01	1.6E-05	6.0E-01				
Br-84	3.1E-04	1.1E+01	3.1E-06	1.1E-01	5.8E-08	2.2E-03	5.8E-10	2.1E-05
Br-85	3.9E-06	1.5E-01	3.9E-08	1.5E-03				
I-129	4.8E-09	1.8E-04	4.8E-11	1.8E-06				
I-130	4.3E-03	1.6E+02	4.3E-05	1.6E+00				
I-131	7.7E-02	2.8E+03	7.7E-04	2.8E+01	4.1E-08	1.5E-03	4.1E-10	1.5E-05
I-132	2.3E-02	8.4E+02	2.3E-04	8.4E+00	6.5E-07	2.4E-02	6.5E-09	2.4E-04
I-133	1.2E-01	4.4E+03	1.2E-03	4.3E+01	5.2E-07	1.9E-02	5.2E-09	1.9E-04
I-134	6.7E-03	2.5E+02	6.7E-05	2.5E+00	5.5E-07	2.0E-02	5.5E-09	2.0E-04
I-135	6.0E-02	2.2E+03	6.0E-04	2.2E+01	9.2E-07	3.4E-02	9.2E-09	3.4E-04
Rubidium, Cesium^(c)								
Rb-86M	9.0E-12	3.3E-07	4.5E-14	1.7E-09				
Rb-86	1.5E-05	5.7E-01	7.7E-08	2.8E-03				
Rb-88	5.0E-04	1.9E+01	2.5E-06	9.3E-02	4.2E-07	1.6E-02	2.1E-09	7.7E-05
Rb-89	2.0E-05	7.4E-01	1.0E-07	3.7E-03				
Cs-134	1.4E-03	5.1E+01	6.8E-06	2.5E-01	9.3E-10	3.4E-05	4.9E-12	1.8E-07
Cs-136	4.2E-04	1.6E+01	2.1E-06	7.8E-02	2.2E-08	8.3E-04	1.1E-10	4.1E-06
Cs-137	8.7E-04	3.2E+01	4.3E-06	1.6E-01	1.4E-09	5.1E-05	6.6E-12	2.4E-07
Cs-138	1.9E-04	6.9E+00	9.4E-07	3.5E-02				
Miscellaneous Nuclides^(c)								
Sr-89	2.9E-06	1.1E-01	1.4E-08	5.3E-04	2.9E-09	1.1E-04	1.5E-11	5.4E-07
Sr-90	1.4E-07	5.4E-03	7.2E-10	2.7E-05	2.5E-10	9.1E-06	1.2E-12	4.4E-08
Sr-91	3.6E-06	1.3E-01	1.8E-08	6.7E-04	1.8E-08	6.5E-04	8.8E-11	3.3E-06
Sr-92	4.0E-07	1.5E-02	2.0E-09	7.4E-05				
Y-90	3.8E-08	1.4E-03	1.9E-10	7.1E-06				
Y-91M	2.2E-06	8.0E-02	1.1E-08	4.0E-04	2.5E-09	9.1E-05	1.2E-11	4.6E-07
Y-91	3.7E-07	1.4E-02	1.8E-09	6.8E-05	1.1E-10	3.9E-06	5.5E-13	2.0E-08
Y-92	5.3E-07	2.0E-02	2.7E-09	9.9E-05				
Y-93	2.3E-07	8.6E-03	1.2E-09	4.3E-05	7.5E-08	2.8E-03	3.8E-10	1.4E-05
Zr-95	4.2E-07	1.5E-02	2.1E-09	7.7E-05	8.0E-09	3.0E-04	4.0E-11	1.5E-06
Zr-97	2.6E-07	9.6E-03	1.3E-09	4.8E-05				
Nb-95	4.2E-07	1.5E-02	2.1E-09	7.7E-05	5.5E-09	2.0E-04	2.9E-11	1.1E-06

Table 3.5-3—Secondary Coolant Radionuclide Concentrations

(Page 2 of 3)

Radionuclide	Design Basis Liquid		Design Basis Steam		Realistic Source Term- Liquid ^(e)		Realistic Source Term- Steam ^(e)	
	μCi/gm	Bq/gm	μCi/gm	Bq/gm	μCi/gm	Bq/gm	μCi/gm	Bq/gm
Mo-99	4.6E-04	1.7E+01	2.3E-06	8.5E-02	1.3E-07	4.9E-03	6.3E-10	2.3E-05
Tc-99M	2.6E-04	9.8E+00	1.3E-06	4.9E-02	7.3E-08	2.7E-03	3.8E-10	1.4E-05
Ru-103	3.4E-07	1.3E-02	1.7E-09	6.4E-05	1.6E-07	5.8E-03	8.0E-10	3.0E-05
Ru-105	2.8E-07	1.0E-02	1.4E-09	5.2E-05				
Ru-106	1.2E-07	4.4E-03	6.0E-10	2.2E-05	1.9E-06	6.8E-02	9.0E-09	3.3E-04
Rh-103M	3.1E-07	1.1E-02	1.5E-09	5.7E-05				
Rh-105	2.0E-07	7.4E-03	1.0E-09	3.7E-05				
Rh-106	1.2E-07	4.4E-03	6.0E-10	2.2E-05				
Ag-110M	8.8E-10	3.3E-05	4.4E-12	1.6E-07	2.7E-08	9.8 E-04	1.4E-10	5.0E-06
Ag-110	1.2E-11	4.4E-07	5.9E-14	2.2E-09				
Sb-125	3.5E-09	1.3E-04	1.8E-11	6.6E-07				
Sb-127	2.2E-08	8.1E-04	1.1E-10	4.0E-06				
Sb-129	1.9E-08	7.1E-04	9.6E-11	3.6E-06				
Te-127M	2.0E-06	7.3E-02	9.8E-09	3.6E-04				
Te-127	8.1E-06	3.0E-01	4.0E-08	1.5E-03				
Te-129M	6.4E-06	2.4E-01	3.2E-08	1.2E-03	3.9E-09	1.5E-04	2.0E-11	7.3E-07
Te-129	6.3E-06	2.3E-01	3.1E-08	1.2E-03	1.7E-07	6.2E-03	8.3E-10	3.1E-05
Te-131M	1.5E-05	5.7E-01	7.7E-08	2.8E-03	3.0E-08	1.1E-03	1.5E-10	5.6E-06
Te-131	4.6E-06	1.7E-01	2.3E-08	8.5E-04	2.3E-08	8.4E-04	1.2E-10	4.3E-06
Te-132	1.8E-04	6.5E+00	8.8E-07	3.3E-02	3.5E-08	1.3E-03	1.7E-10	6.4E-06
Te-134	6.4E-06	2.4E-01	3.2E-08	1.2E-03				
Ba-137M	8.1E-04	3.0E+01	4.1E-06	1.5E-01				
Ba-139	3.9E-05	1.4E+00	1.9E-07	7.2E-03				
Ba-140	2.7E-06	1.0E-01	1.4E-08	5.1E-04	2.6E-07	9.7E-03	1.3E-09	4.9E-05
La-140	8.4E-07	3.1E-02	4.2E-09	1.6E-04	5.1E-07	1.9E-02	2.5E-09	9.3E-05
La-141	1.5E-07	5.5E-03	7.4E-10	2.8E-05				
La-142	5.6E-08	2.1E-03	2.8E-10	1.0E-05				
Ce-141	3.9E-07	1.5E-02	2.0E-09	7.3E-05	3.1E-09	1.1E-04	1.6E-11	5.8E-07
Ce-143	3.1E-07	1.1E-02	1.6E-09	5.8E-05	5.5E-08	2.0E-03	2.8E-10	1.0E-05
Ce-144	3.1E-07	1.1E-02	1.5E-09	5.7E-05	8.0E-08	3.0E-03	4.1E-10	1.5E-05
Pr-143	3.9E-07	1.4E-02	2.0E-09	7.4E-05				
Pr-144	3.1E-07	1.1E-02	1.5E-09	5.6E-05				
Nd-147	1.5E-07	5.6E-03	7.5E-10	2.8E-05				
Np-239	3.7E-06	1.4E-01	1.9E-08	6.9E-04	4.5E-08	1.7E-03	2.2E-10	8.3E-06
Pu-238	8.9E-10	3.3E-05	4.4E-12	1.6E-07				
Pu-239	9.0E-11	3.3E-06	4.5E-13	1.7E-08				
Pu-240	1.2E-10	4.6E-06	6.2E-13	2.3E-08				
Pu-241	3.1E-08	1.1E-03	1.5E-10	5.7E-06				
Am-241	3.5E-11	1.3E-06	1.7E-13	6.4E-09				
Cm-242	8.3E-09	3.1E-04	4.2E-11	1.5E-06				
Cm-244	4.6E-10	1.7E-05	2.3E-12	8.4E-08				
Activation Products^(d)								
Na-24	3.5E-05	1.3E+00	1.8E-07	6.5E-03	8.9E-07	3.3E-02	4.5E-09	1.7E-04
Cr-51	2.2E-06	8.2E-02	1.1E-08	4.1E-04	6.5E-08	2.4E-03	3.2E-10	1.2E-05
Mn-54	1.1E-06	4.1E-02	5.6E-09	2.1E-04	3.3E-08	1.2E-03	1.7E-10	6.3E-06
Fe-55	8.5E-07	3.1E-02	4.2E-09	1.6E-04	2.5E-08	9.3E-04	1.3E-10	4.8E-06
Fe-59	2.1E-07	7.8E-03	1.1E-09	3.9E-05	6.0E-09	2.2E-04	3.1E-11	1.1E-06

Table 3.5-3—Secondary Coolant Radionuclide Concentrations

(Page 3 of 3)

Radionuclide	Design Basis Liquid		Design Basis Steam		Realistic Source Term- Liquid ^(e)		Realistic Source Term- Steam ^(e)	
	μCi/gm	Bq/gm	μCi/gm	Bq/gm	μCi/gm	Bq/gm	μCi/gm	Bq/gm
Co-58	3.2E-06	1.2E-01	1.6E-08	6.0E-04	9.5E-08	3.5E-03	4.7E-10	1.7E-05
Co-60	3.8E-07	1.4E-02	1.9E-09	7.0E-05	1.1E-08	4.1E-04	5.5E-11	2.0E-06
Zn-65	3.6E-07	1.3E-02	1.8E-09	6.6E-05	1.1E-08	3.9E-04	5.0E-11	1.9E-06
W1-87	1.8E-06	6.7E-02	9.1E-09	3.4E-04	4.9E-08	1.8E-03	2.5E-10	9.3E-06
Nitrogen								
N-16					6.9E-07	2.6E-02	6.9E-08	2.6E-03
Tritium^(d)								
H-3	4.0E-32	1.5E-27	4.0E+00	1.5E+05	1.0E-03	3.7E+01	1.0E-03	3.7E+01

Notes:

For design basis concentrations, the following conditions apply:

- (a) The noble gases are assumed to enter the steam phase instantly.
- (b) The halogen concentrations are at the US EPR Standard Technical Specification limit of 0.1 μCi/gm DE-I131.
- (c) The concentrations for this group are based on 1.0% failed fuel fraction.
- (d) The concentration of activation products conservatively assumed to be same concentration as in primary coolant.
- (e) Normal operation coolant concentrations for the ANSI/ANS-18.1-1999 reference PWR with U-tube steam generators

Table 3.5-4—Principal Parameters Used in Estimating Realistic Releases of Radioactive Materials in Effluents (GALE Code Input Parameters)

(Page 1 of 4)

Item	GALE Input Parameter	Value
0	Name and Type of Reactor	U.S. EPR PWR
1	Thermal Power Level (MWth) (4,590 MWth + 22 MWth measurement uncertainty)	4,612 MWth (4.612E9 J/sec)
2	Mass of Coolant in Primary System (RCS dry nominal volume - not including the pressurizer) (13,596 ft ³ /0.02290 ft ³ /lbm)	5.937E5 lbm (2.693E5 kg)
3	Primary System Letdown Rate (7.94E+04 lbm/h x 0.0229 ft ³ /lbm x 7.48 gal/ft ³ x 1 min/60 sec = 226.7 gpm)	226.7 gpm (858.2 l/min)
4	Letdown Cation Demineralizer Flow Rate (No purification system cation demineralizer)	0 gpm (0 l/min)
5	Number of steam generators	4
6	Total steam flow (Nominal 4 x 5.168E+06 = 20.67E+06 lbm/hr Increase by 1.05 to account for higher thermal power = 21.71E+06 lbm/hr)	2.171E7 lbm/hr (9.845E6 kg/hr)
7	Mass of liquid in secondary side of each steam generator (SG)	1.6977E5 lbm (7.7006E5 kg)
8	SG Blowdown rate (Nominal 4 x 0.052E+06 lbm/hr = 208E+03 lbm/hr Adjust by 1.05 to account for higher thermal power 208 x 1.05 = 218.4E+03)	2.184E5 lbm/hr (9.8901E4 kg/hr)
9	Blowdown Treatment Method (Full blowdown flow processed by Blowdown System and recycled to condensate system.)	0
10	Condensate Demineralizer Regeneration Time (days) (Regeneration not used)	0
11	Condensate Demineralizer Flow Fraction	0.33
12	Shim Bleed Flow Rate (gpd) (Shim bleed is letdown flow for boron control and the liquid is recycled. The nominal flow is: 500 lbm/hr x 0.0229 ft ³ /lbm x 7.48 gal/ft ³ x 24 hr/day = 2,056 gpd Adjusting by 1.05 to account for higher thermal power yields 2,158 gpd. The analysis will conservatively assume that 5% of the processed shim bleed flow 2,158 x 0.05 = 107.9 rounded to 110 gpd is liquid waste)	110 gpd (416 l/day)
13	Shim Bleed DF for Iodine (With Liquid Waste Storage and Processing System Demineralizer)	1.0E4
14	Shim Bleed DF for Cesium and Rubidium (With Liquid Waste Storage and Processing System Demineralizer)	1.0E7
15	Shim Bleed DF for Other Nuclides (With Liquid Waste Storage and Processing System Demineralizer)	1.0E7
16	Shim Bleed Collection Time (days) $\frac{18500 \text{ gal}}{(110 + 1728) \frac{\text{gal}}{\text{day}}} * 0.8 = 8.05 = 8.1 \text{ days}$ (The collection time is for one tank. The collection time includes 1,728 gpd (6,541 lpd) from equipment drains.)	8.1 days
17	Shim Bleed Processing and Discharge Times (days) $\frac{18500 \text{ gal}}{(1.1 \frac{\text{kg}}{\text{sec}})(0.001 \frac{\text{m}^3}{\text{kg}})(86400 \frac{\text{sec}}{\text{day}})} * 0.8 = 0.589 \text{ days}$	0.589 days
18	Shim Bleed Average Fraction of Waste to be Discharged (There is no recycling of liquid radioactive waste.)	1.0

Table 3.5-4—Principal Parameters Used in Estimating Realistic Releases of Radioactive Materials in Effluents (GALE Code Input Parameters)

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Item	GALE Input Parameter	Value
19	Equipment Drains Input (gpd) (Based on U.S. EPR Standard Technical Specification limit on unidentified leakage of 1 gpm (3.79 lpm). Assumes collected by floor drains. Twenty percent added for conservatism.)	1,728 gal/day 6,541 l/day
20	Equipment Drains Primary Coolant Activity (PCA)	1.0
21	Equipment Drains DF for Iodine (With Liquid Waste Storage and Processing System Demineralizer)	1.0E4
22	Equipment Drains DF for Cesium and Rubidium (With Liquid Waste Storage and Processing System Demineralizer)	1.0E7
23	Equipment Drains DF for Other Nuclides (With Liquid Waste Storage and Processing System Demineralizer)	1.0E7
24	Equipment Drains Collection Time (days) (Includes 110 gpd (416.4 lpd) from shim bleed.) $\frac{70 m^3}{\left((110 + 1728) \frac{gal}{day} \right) \left(\frac{1 m^3}{264.17 gal} \right)} * 0.8 = 8.1 \text{ days}$ (Includes 110 gpd (416.4 lpd) from shim bleed.)	8.1 days
25	Equipment Drains Processing and Discharge Times (days) $\frac{70 m^3}{\left(1.1 \frac{kg}{sec} \right) \left(0.001 \frac{m^3}{kg} \right) \left(86400 \frac{sec}{day} \right)} * 0.8 = 0.589 \text{ days}$	0.589 days
26	Equipment Drains Average Fraction of Waste to be Discharged (There is no recycling of liquid radioactive waste.)	1.0
27	Clean Waste Input (gpd) (Clean Waste included as Group II.) (Conservative – 66,000 gal/week / 7 day/week = 9,428 gallons per day)	9,428 gal/day 35,690 l/day
28	Clean Waste PCA	0.001
29	Clean Waste DF for Iodine (With Liquid Waste Storage and Processing System Demineralizer)	1.0E2
30	Clean Waste DF for Cesium and Rubidium (With Liquid Waste Storage and Processing System Demineralizer)	1.0E2
31	Clean Waste DF for Other Nuclides (With Liquid Waste Storage and Processing System Demineralizer)	1.0E2
32	Clean Waste Collection Time (days) $\frac{70 m^3}{\left(250 \frac{m^3}{week} \right) \left(\frac{1 week}{7 days} \right)} * 0.8 = 1.6 \text{ days}$	1.6 days
33	Clean Waste Processing and Discharge Times (days) $\frac{70 m^3}{\left(1.4 \frac{kg}{sec} \right) \left(0.001 \frac{m^3}{kg} \right) \left(86400 \frac{sec}{day} \right)} * 0.8 = 0.463 \text{ days}$	0.463 days
34	Clean Waste Average Fraction of Waste to be Discharged (There is no recycling of liquid radioactive waste.)	1.0
35	Dirty Waste Input (gpd) (Group III waste is normally not radioactive and it is neglected to maximize concentrations)	0 gal/day (0 l/day)
36	Dirty Waste PCA (N/A since input is 0 gallons per day)	N/A

Table 3.5-4—Principal Parameters Used in Estimating Realistic Releases of Radioactive Materials in Effluents (GALE Code Input Parameters)

(Page 3 of 4)

Item	GALE Input Parameter	Value
37	Dirty Waste DF for Iodine (N/A since input is 0 gallons per day)	N/A
38	Dirty Waste DF for Cesium and Rubidium (N/A since input is 0 gallons per day)	N/A
39	Dirty Waste DF for Other Nuclides (N/A since input is 0 gallons per day)	N/A
40	Dirty Waste Collection Time (days) (N/A since input is 0 gallons per day)	N/A
41	Dirty Waste Processing and Discharge Times (days) (N/A since input is 0 gallons per day)	N/A
42	Dirty Waste Average Fraction of Waste to be Discharged (There is no recycling of liquid radioactive waste.)	1.0
43	Blowdown Fraction Processed	1.0
44	Blowdown DF for Iodine (1 in the cation bed x 100 in the mixed bed = 100 overall)	1.0E+02
45	Blowdown DF for Cesium and Rubidium (10 in the cation bed x 10 in the mixed bed = 100 overall)	1.0E+02
46	Blowdown DF for Other Nuclides (10 in the cation bed x 100 in the mixed bed = 1000 overall)	1.0E+03
47	Blowdown Collection Time (days)	0 days
48	Blowdown Processing and Discharge Times (days)	0 days
49	Blowdown Average Fraction of Waste to be Discharged	0.0
50	Regenerant Flow Rate (gpd) (Regeneration not used)	0.0
51	Regenerant DF for Iodine	1.0
52	Regenerant DF for Cesium and Rubidium	1.0
53	Regenerant DF for Other Nuclides	1.0
54	Regenerant Collection Time (days)	0.0
55	Regenerant Processing and Discharge Times (days)	0.0
56	Regenerant Average Fraction of Waste to be Discharged	0.0
57	Is There Continuous Stripping of Full Letdown Flow? (The degasification is normally operated prior to refueling, prior to maintenance of the reactor coolant circuit or if required to decrease the concentration of gaseous reactivity. Value of 'Y' for card 30 is ratio of total amount of noble gases routed to gaseous radwaste from the purification system to total routed from the primary coolant system. Options are 0, 0.25, 1. This is a recycled loop during normal operations, and very little of the flow ends up in delay beds, the value of 0 best represents system.)	No
58	Holdup Time for Xenon (days)	27.7 days
59	Holdup Time for Krypton (days)	1.67 days
60	Fill Time of Decay Tanks for the Gas Stripper (Days) (Discharged directly to the stack.)	0 days
61	Waste Gas System Particulate Releases HEPA Efficiency (%)	99%
62	Fuel Handling Building Releases: Charcoal Efficiency (%) (HEPA and Charcoal efficiencies for non-ESF systems taken to be the same as Gaseous Waste Processing System)	90%
63	Fuel Handling Building Releases: HEPA Efficiency (%) (HEPA and Charcoal efficiencies for non-ESF systems taken to be the same as Gaseous Waste Processing System)	99%
64	Auxiliary Building Releases: Charcoal Efficiency (%) (HEPA and Charcoal efficiencies for non-ESF systems taken to be the same as Gaseous Waste Processing System)	90%

Table 3.5-4—Principal Parameters Used in Estimating Realistic Releases of Radioactive Materials in Effluents (GALE Code Input Parameters)

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Item	GALE Input Parameter	Value
65	Auxiliary Building Releases: HEPA Efficiency (%) (HEPA and Charcoal efficiencies for non-ESF systems taken to be the same as Gaseous Waste Processing System)	99%
66	Containment Free Volume.	2,800,000 ft ³ (79,000 m ³)
67	Containment Internal Cleanup System: Charcoal Efficiency (%) (HEPA and Charcoal efficiencies for non-ESF systems taken to be the same as Gaseous Waste Processing System)	90%
68	Containment Internal Cleanup System: HEPA Efficiency (%) (HEPA and Charcoal efficiencies for non-ESF systems taken to be the same as Gaseous Waste Processing System)	99%
69	Containment Internal Cleanup System: Flow Rate	4,100 cfm (116,100 liter/min)
70	Containment High Volume Purge: Charcoal Efficiency (%) (HEPA and Charcoal efficiencies for non-ESF systems taken to be the same as Gaseous Waste Processing System)	90%
71	Containment High Volume Purge: HEPA Efficiency (%) (HEPA and Charcoal efficiencies for non-ESF systems taken to be the same as Gaseous Waste Processing System)	99%
72	Containment High Volume Purge: Purges per Year	2 per yr.
73	Containment Low Volume Purge: Charcoal Efficiency (%)	90%
74	Containment Low Volume Purge: HEPA Efficiency (%)	99%
75	Containment Low Volume Purge: Flow Rate (cfm)	2,970 cfm (84,100 liter/min)
76	Percent of Iodine Released from Blowdown Tank Vent	0.0%
77	Percent of Iodine Removed from Air Ejector Release (No condenser air ejectors, mechanical vacuum pumps vent to stack without treatment)	0.0%
78	Detergent Waste PF (No onsite laundry)	0.0
79	SG blowdown flash tank gases vented via main condenser air ejector?	No
80	Condenser air ejector offgas released without treatment? (No condenser air ejectors, mechanical vacuum pumps vent to stack without treatment)	Yes
81	Condenser air ejector offgas processed via charcoal adsorbers prior to release? (No condenser air ejectors, mechanical vacuum pumps vent to stack without treatment)	No
82	Average flow rate of water used to dilute liquid waste discharged to the environment.	5,673 (21,472 lpm)
83	Number of Main Condenser Water Boxes	3
84	Main Condenser Water Box liquid volume (each) (nominal operating conditions) (ft ³) (m ³)	6,357 ft ³ (180 m ³)
85	Main Condenser Water Box temperature (nominal operating conditions) (°F) (°C)	69.4 °F (20.8 °C)
86	Main Condenser Water Box pressure (nominal operating conditions) (millibars)	24.7

Table 3.5-5—Average Radioactivity Concentrations in the Spent Fuel Pool (SFP) Area

(Page 1 of 2)

Nuclide	SFP Water Activity		SFP Airborne Activity	
	($\mu\text{Ci}/\text{cm}^3$)	(MBq/cm^3)	($\mu\text{Ci}/\text{cm}^3$)	(MBq/cm^3)
H-3	5.90E-01	2.18E-02	2.67E-06	9.88E-08
Na-24	1.13E-06	4.18E-08	5.10E-12	1.89E-13
Cr-51	6.01E-06	2.22E-07	2.72E-11	1.01E-12
Mn-54	3.40E-06	1.26E-07	1.54E-11	5.70E-13
Fe-55	2.57E-06	9.51E-08	1.17E-11	4.33E-13
Fe-59	6.06E-07	2.24E-08	2.75E-12	1.02E-13
Co-58	9.50E-06	3.52E-07	4.30E-11	1.59E-12
Co-60	1.14E-06	4.22E-08	5.15E-12	1.91E-13
Zn-65	1.08E-06	4.00E-08	4.91E-12	1.82E-13
Br-83	2.43E-17	8.99E-19	1.07E-22	3.96E-24
Kr-83M	4.87E-16	1.80E-17	2.13E-21	7.88E-23
Kr-85M	1.18E-10	4.37E-12	5.28E-16	1.95E-17
Kr-85	4.33E-03	1.60E-04	1.96E-08	7.25E-10
Kr-87	3.71E-26	1.37E-27	1.60E-31	5.92E-33
Kr-88	6.73E-14	2.49E-15	2.98E-19	1.10E-20
Rb-88	6.79E-14	2.51E-15	3.06E-19	1.13E-20
Sr-89	1.52E-07	5.62E-09	6.89E-13	2.55E-14
Sr-90	1.69E-08	6.25E-10	7.67E-14	2.84E-15
Sr-91	1.42E-10	5.25E-12	6.39E-16	2.36E-17
Sr-92	4.58E-18	1.69E-19	2.02E-23	7.47E-25
Y-90	4.69E-09	1.74E-10	2.13E-14	7.88E-16
Y-91M	7.84E-11	2.90E-12	3.56E-16	1.32E-17
Y-91	3.01E-08	1.11E-09	1.36E-13	5.03E-15
Y-92	8.42E-16	3.12E-17	3.75E-21	1.39E-22
Y-93	3.95E-11	1.46E-12	1.78E-16	6.59E-18
Zr-95	3.50E-08	1.30E-09	1.59E-13	5.88E-15
Nb-95	3.67E-08	1.36E-09	1.66E-13	6.14E-15
Mo-99	1.75E-05	6.48E-07	7.92E-11	2.93E-12
Tc-99M	9.60E-06	3.55E-07	4.38E-11	1.62E-12
Ru-103	3.59E-08	1.33E-09	1.63E-13	6.03E-15
Ru-106	2.27E-08	8.40E-10	1.03E-13	3.81E-15
Rh-103M	3.24E-08	1.20E-09	1.47E-13	5.44E-15
Rh-106	2.27E-08	8.40E-10	1.03E-13	3.81E-15
Ag-110M	3.83E-10	1.42E-11	1.74E-15	6.44E-17
Te-127M	2.41E-07	8.92E-09	1.09E-12	4.03E-14
Te-129M	6.52E-07	2.41E-08	2.95E-12	1.09E-13
Te-129	4.24E-07	1.57E-08	1.92E-12	7.10E-14
Te-131M	1.92E-07	7.10E-09	8.70E-13	3.22E-14
Te-131	4.33E-08	1.60E-09	1.96E-13	7.25E-15
Te-132	7.92E-06	2.93E-07	3.59E-11	1.33E-12
Te-134	2.09E-45	7.73E-47	8.64E-51	3.20E-52
I-129	1.08E-11	4.00E-13	4.87E-17	1.80E-18
I-130	6.09E-08	2.25E-09	2.74E-13	1.01E-14
I-131	1.26E-04	4.66E-06	5.72E-10	2.12E-11
I-132	2.87E-05	1.06E-06	1.27E-10	4.70E-12
I-133	1.29E-05	4.77E-07	5.83E-11	2.16E-12
I-134	1.67E-35	6.18E-37	7.03E-41	2.60E-42
I-135	1.11E-08	4.11E-10	4.96E-14	1.84E-15

Table 3.5-5—Average Radioactivity Concentrations in the Spent Fuel Pool (SFP) Area

(Page 2 of 2)

Nuclide	SFP Water Activity		SFP Airborne Activity	
	($\mu\text{Ci}/\text{cm}^3$)	(MBq/cm^3)	($\mu\text{Ci}/\text{cm}^3$)	(MBq/cm^3)
Xe-131M	2.81E-04	1.04E-05	1.27E-09	4.70E-11
Xe-133M	1.39E-04	5.14E-06	6.30E-10	2.33E-11
Xe-133	1.67E-02	6.18E-04	7.54E-08	2.79E-09
Xe-135M	3.39E-09	1.25E-10	1.37E-14	5.07E-16
Xe-135	4.68E-06	1.73E-07	2.11E-11	7.81E-13
Cs-134	1.64E-04	6.07E-06	7.44E-10	2.75E-11
Cs-136	3.16E-05	1.17E-06	1.43E-10	5.29E-12
Cs-137	6.29E-05	2.33E-06	2.85E-10	1.05E-11
Ba-137M	5.92E-05	2.19E-06	2.69E-10	9.95E-12
Ba-140	2.00E-07	7.40E-09	9.04E-13	3.34E-14
La-140	6.93E-08	2.56E-09	3.15E-13	1.17E-14
Ce-141	3.29E-08	1.22E-09	1.49E-13	5.51E-15
Ce-143	4.26E-09	1.58E-10	1.93E-14	7.14E-16
Ce-144	2.70E-08	9.99E-10	1.22E-13	4.51E-15
Pr-143	3.19E-08	1.18E-09	1.44E-13	5.33E-15
Pr-144	2.70E-08	9.99E-10	1.22E-13	4.51E-15
W-187	3.17E-07	1.17E-08	1.43E-12	5.29E-14
Np-239	1.73E-07	6.40E-09	7.81E-13	2.89E-14
Total (Excluding Tritium)	2.20E-02	8.14E-04	9.94E-08	3.68E-09
Iodines	1.68E-04	6.22E-06	7.58E-10	2.80E-11
Particulates	2.58E-05	9.55E-07	1.17E-10	4.33E-12
Noble Gases	2.14E-02	7.92E-04	9.70E-08	3.59E-09

Note:

$$\text{MBq}/\text{cm}^3 = 1.0\text{E}6 \text{ Bq}/\text{cm}^3$$

Table 3.5-6—Liquid Waste Release Source Term Inputs

Liquid Waste Inputs								
Stream	Flow Rate (gal/day) (L/day)	Fraction of PCA	Fraction Discharged	Collection Time (days)	Decay Time (days)	Decontamination Factors		
						I	Cs	Others
Shim Bleed Rate	1.10E+02 (4.16 E+02)	1.0	1.0	8.1	0.589	1.0E+04	1.0E+07	1.0E+07
Equipment Drains	1.73E+03 (6.55E+03)	1.0	1.0	8.1	0.589	1.0E+04	1.0E+07	1.0E+07
Clean Waste Input	9.43E+03 (3.57E+04)	0.001	1.0	1.6	0.463	1.0E+02	1.0E+02	1.0E+02
Dirty Wastes	0.00E+00 (0.00E+00)	0.1	1.0	0.0	0.0	1.0E+02	1.0E+03	1.0E+03
Blowdown	6.28E+05 (2.38E+06)		0.0	0.0	0.0	1.0E+02	1.0E+02	1.0E+03
Untreated Blowdown	0.00E+00 (0.00E+00)		1.0	0.0	0.0	1.0E+00	1.0E+00	1.0E+00
Regenerant Sols.	0.00E+00 (0.00E+00)		0.0	0.0	0.0	1.0E+00	1.0E+00	1.0E+00

Table 3.5-7—Annual Expected Liquid Waste Releases (in Curies)
(Page 1 of 2)

Nuclide	Half-Life (days)	Primary (mCi/ml)	Secondary (mCi/ml)	Boron Recovery System (Ci)	Misc Wastes (Ci)	Secondary (Ci)	Turbine Building (Ci)	Total Liquid Waste Sources (Ci)	Adjusted Total (Ci/yr)	Detergent Wastes (Ci/yr)	Total (Ci/yr)
Corrosion and Activation Products											
Na-24	6.25E+01	2.84E-02	3.40E-07	0.00000	0.00104	0.00000	0.00001	0.00105	0.00613	0.00000	0.00610
Cr-51	2.78E+01	1.39E-03	1.96E-08	0.00000	0.00018	0.00000	0.00000	0.00018	0.00103	0.00000	0.00100
Mn-54	3.03E+02	7.09E-04	9.66E-09	0.00000	0.00009	0.00000	0.00000	0.00009	0.00054	0.00000	0.00054
Fe-55	9.50E+02	5.32E-04	7.28E-09	0.00000	0.00007	0.00000	0.00000	0.00007	0.00041	0.00000	0.00041
Fe-59	4.50E+01	1.34E-04	1.80E-09	0.00000	0.00002	0.00000	0.00000	0.00002	0.00010	0.00000	0.00010
Co-58	7.13E+01	2.04E-03	2.84E-08	0.00000	0.00026	0.00000	0.00000	0.00027	0.00155	0.00000	0.00150
Co-60	1.92E+03	2.35E-04	3.27E-09	0.00000	0.00003	0.00000	0.00000	0.00003	0.00018	0.00000	0.00018
Zn-65	2.45E+02	2.26E-04	3.12E-09	0.00000	0.00003	0.00000	0.00000	0.00003	0.00017	0.00000	0.00017
W-187	9.96E+01	1.38E-03	1.73E-08	0.00000	0.00008	0.00000	0.00000	0.00008	0.00046	0.00000	0.00046
Np-239	2.35E+00	1.08E-03	1.44E-08	0.00000	0.00010	0.00000	0.00000	0.00010	0.00058	0.00000	0.00058
Fission Products											
Sr-89	5.20E+01	6.23E-05	8.52E-10	0.00000	0.00001	0.00000	0.00000	0.00001	0.00005	0.00000	0.00005
Sr-91	4.03E-01	6.41E-04	7.35E-09	0.00000	0.00001	0.00000	0.00000	0.00001	0.00008	0.00000	0.00008
Y-91M	3.47E-02	5.09E-04	2.01E-09	0.00000	0.00001	0.00000	0.00000	0.00001	0.00005	0.00000	0.00005
Y-93	4.25E-01	2.77E-03	3.09E-08	0.00000	0.00006	0.00000	0.00000	0.00006	0.00036	0.00000	0.00036
Zr-95	6.50E+01	1.73E-04	2.39E-09	0.00000	0.00002	0.00000	0.00000	0.00002	0.00013	0.00000	0.00013
Nb-95	3.50E+01	1.23E-04	1.65E-09	0.00000	0.00002	0.00000	0.00000	0.00002	0.00010	0.00000	0.00010
Mo-99	2.79E+00	3.11E-03	4.19E-08	0.00000	0.00030	0.00000	0.00000	0.00030	0.00175	0.00000	0.00180
Tc-99M	2.50E-01	3.54E-03	3.47E-08	0.00000	0.00029	0.00000	0.00000	0.00029	0.00170	0.00000	0.00170
Ru-103	3.96E+01	3.34E-03	4.65E-08	0.00000	0.00043	0.00000	0.00000	0.00043	0.00251	0.00000	0.00250
Rh-103M	3.96E-02	0.00E+00	0.00E+00	0.00000	0.00043	0.00000	0.00000	0.00043	0.00251	0.00000	0.00250
Ru-106	3.67E+02	3.99E-02	5.50E-07	0.00001	0.00518	0.00000	0.00003	0.00522	0.03050	0.00000	0.03100
Rh-106	3.47E-04	0.00E+00	0.00E+00	0.00001	0.00518	0.00000	0.00003	0.00522	0.03050	0.00000	0.03100
Ag-110M	2.53E+02	5.76E-04	7.88E-09	0.00000	0.00007	0.00000	0.00000	0.00008	0.00044	0.00000	0.00044
Ag-110	2.82E-04	0.00E+00	0.00E+00	0.00000	0.00001	0.00000	0.00000	0.00001	0.00006	0.00000	0.00006
Te-129M	3.40E+01	8.48E-05	1.17E-09	0.00000	0.00001	0.00000	0.00000	0.00001	0.00006	0.00000	0.00006
Te-129	4.79E-02	2.55E-02	1.28E-07	0.00000	0.00001	0.00000	0.00000	0.00001	0.00004	0.00000	0.00004
Te-131M	1.25E+00	7.98E-04	1.02E-08	0.00000	0.00005	0.00000	0.00000	0.00005	0.00031	0.00000	0.00031
Te-131	1.74E-02	9.04E-03	2.07E-08	0.00000	0.00001	0.00000	0.00000	0.00001	0.00006	0.00000	0.00006

Table 3.5-7—Annual Expected Liquid Waste Releases (in Curies)
(Page 2 of 2)

Nuclide	Half-Life (days)	Primary (mCi/ml)	Secondary (mCi/ml)	Boron Recovery System (Ci)	Misc Wastes (Ci)	Secondary (Ci)	Turbine Building (Ci)	Total Liquid Waste Sources (Ci)	Adjusted Total (Ci/yr)	Detergent Wastes (Ci/yr)	Total (Ci/yr)	
I-131	8.05E+00	2.07E-02	2.49E-07	0.00341	0.00243	0.00000	0.00002	0.00586	0.03424	0.00000	0.03400	
Te-132	3.25E+00	8.15E-04	1.09E-08	0.00000	0.00008	0.00000	0.00000	0.00008	0.00048	0.00000	0.00048	
I-132	9.58E-02	1.98E-01	1.34E-06	0.00001	0.00016	0.00000	0.00002	0.00020	0.00115	0.00000	0.00120	
I-133	8.75E-01	7.92E-02	8.87E-07	0.00185	0.00405	0.00000	0.00007	0.00597	0.03488	0.00000	0.03500	
Cs-134	7.49E+02	3.46E-03	4.87E-08	0.00000	0.00045	0.00000	0.00000	0.00045	0.00265	0.00000	0.00260	
I-135	2.79E-01	1.90E-01	1.81E-06	0.00052	0.00194	0.00000	0.00010	0.00256	0.01496	0.00000	0.01500	
Cs-136	1.30E+01	4.38E-04	6.16E-09	0.00000	0.00005	0.00000	0.00000	0.00005	0.00031	0.00000	0.00031	
Cs-137	1.10E+04	4.57E-03	6.49E-08	0.00000	0.00060	0.00000	0.00000	0.00060	0.00351	0.00000	0.00350	
Ba-137M	1.77E-03	0.00E+00	0.00E+00	0.00000	0.00056	0.00000	0.00000	0.00056	0.00328	0.00000	0.00330	
Ba-140	1.28E+01	5.88E-03	7.94E-08	0.00000	0.00072	0.00000	0.00000	0.00072	0.00421	0.00000	0.00420	
La-140	1.67E+00	1.28E-02	1.67E-07	0.00000	0.00130	0.00000	0.00001	0.00131	0.00763	0.00000	0.00760	
Ce-141	3.24E+01	6.70E-05	9.16E-10	0.00000	0.00001	0.00000	0.00000	0.00001	0.00005	0.00000	0.00005	
Ce-143	1.38E+00	1.47E-03	1.86E-08	0.00000	0.00010	0.00000	0.00000	0.00010	0.00061	0.00000	0.00061	
Pr-143	1.37E+01	0.00E+00	0.00E+00	0.00000	0.00001	0.00000	0.00000	0.00001	0.00005	0.00000	0.00005	
Ce-144	2.84E+02	1.73E-03	2.38E-08	0.00000	0.00022	0.00000	0.00000	0.00023	0.00132	0.00000	0.00130	
Pr-144	1.20E-02	0.00E+00	0.00E+00	0.00000	0.00022	0.00000	0.00000	0.00023	0.00132	0.00000	0.00130	
All Others		6.25E-01	1.89E-06	0.00000	0.00000	0.00000	0.00000	0.00000	0.00002	0.00000	0.00002	
Total (Except Tritium)		1.27E+00	7.93E-06	0.00582	0.02690	0.00000	0.00033	0.03304	0.19304	0.00000	0.19000	
Tritium Release		1.66E+03 Curies per year										

Note:

0.00000 indicates that the value is less than 1.0E-05.

Table 3.5-7—Annual Expected Liquid Waste Releases (in Becquerels)
(Page 1 of 2)

Nuclide	Half-Life (days)	Primary (Bq/ml)	Secondary (Bq/ml)	Boron Recovery System (Bq)	Misc Wastes (Bq)	Secondary (Bq)	Turbine Building (Bq)	Total Liquid Waste Sources (Bq)	Adjusted Total (Bq/yr)	Detergent Wastes (Bq/yr)	Total (Bq/yr)
Corrosion and Activation Products											
Na-24	6.25E-01	1.05E+03	1.26E-02	0.00E+00	3.85E+07	0.00E+00	3.70E+05	3.89E+07	2.27E+08	0.00E+00	2.26E+08
Cr-51	2.78E+01	5.14E+01	7.25E-04	0.00E+00	6.66E+06	0.00E+00	0.00E+00	6.66E+06	3.81E+07	0.00E+00	3.70E+07
Mn-54	3.03E+02	2.62E+01	3.57E-04	0.00E+00	3.33E+06	0.00E+00	0.00E+00	3.33E+06	2.00E+07	0.00E+00	2.00E+07
Fe-55	9.50E+02	1.97E+01	2.69E-04	0.00E+00	2.59E+06	0.00E+00	0.00E+00	2.59E+06	1.52E+07	0.00E+00	1.52E+07
Fe-59	4.50E+01	4.96E+00	6.66E-05	0.00E+00	7.40E+05	0.00E+00	0.00E+00	7.40E+05	3.70E+06	0.00E+00	3.70E+06
Co-58	7.13E+01	7.55E+01	1.05E-03	0.00E+00	9.62E+06	0.00E+00	0.00E+00	9.99E+06	5.74E+07	0.00E+00	5.55E+07
Co-60	1.92E+03	8.70E+00	1.21E-04	0.00E+00	1.11E+06	0.00E+00	0.00E+00	1.11E+06	6.66E+06	0.00E+00	6.66E+06
Zn-65	2.45E+02	8.36E+00	1.15E-04	0.00E+00	1.11E+06	0.00E+00	0.00E+00	1.11E+06	6.29E+06	0.00E+00	6.29E+06
W-187	9.96E-01	5.11E+01	6.40E-04	0.00E+00	2.96E+06	0.00E+00	0.00E+00	2.96E+06	1.70E+07	0.00E+00	1.70E+07
Np-239	2.35E+00	4.00E+01	5.33E-04	0.00E+00	3.70E+06	0.00E+00	0.00E+00	3.70E+06	2.15E+07	0.00E+00	2.15E+07
Fission Products											
Sr-89	5.20E+01	2.31E+00	3.15E-05	0.00E+00	3.70E+05	0.00E+00	0.00E+00	3.70E+05	1.85E+06	0.00E+00	1.85E+06
Sr-91	4.03E-01	2.37E+01	2.72E-04	0.00E+00	3.70E+05	0.00E+00	0.00E+00	3.70E+05	2.96E+06	0.00E+00	2.96E+06
Y-91M	3.47E-02	1.88E+01	7.44E-05	0.00E+00	3.70E+05	0.00E+00	0.00E+00	3.70E+05	1.85E+06	0.00E+00	1.85E+06
Y-93	4.25E-01	1.02E+02	1.14E-03	0.00E+00	2.22E+06	0.00E+00	0.00E+00	2.22E+06	1.33E+07	0.00E+00	1.33E+07
Zr-95	6.50E+01	6.40E+00	8.84E-05	0.00E+00	7.40E+05	0.00E+00	0.00E+00	7.40E+05	4.81E+06	0.00E+00	4.81E+06
Nb-95	3.50E+01	4.63E+00	6.11E-05	0.00E+00	7.40E+05	0.00E+00	0.00E+00	7.40E+05	3.70E+06	0.00E+00	3.70E+06
Mo-99	2.79E+00	1.15E+02	1.55E-03	0.00E+00	1.11E+07	0.00E+00	0.00E+00	1.11E+07	6.48E+07	0.00E+00	6.66E+07
Tc-99M	2.50E-01	1.31E+02	1.28E-03	0.00E+00	1.07E+07	0.00E+00	0.00E+00	1.07E+07	6.29E+07	0.00E+00	6.29E+07
Ru-103	3.96E+01	1.24E+02	1.72E-03	0.00E+00	1.59E+07	0.00E+00	0.00E+00	1.59E+07	9.29E+07	0.00E+00	9.25E+07
Rh-103M	3.96E-02	0.00E+00	0.00E+00	0.00E+00	1.59E+07	0.00E+00	0.00E+00	1.59E+07	9.29E+07	0.00E+00	9.25E+07
Ru-106	3.67E+02	1.48E+03	2.04E-02	3.70E+05	1.92E+08	0.00E+00	1.11E+06	1.93E+08	1.13E+09	0.00E+00	1.15E+09
Rh-106	3.47E-04	0.00E+00	0.00E+00	3.70E+05	1.92E+08	0.00E+00	1.11E+06	1.93E+08	1.13E+09	0.00E+00	1.15E+09
Ag-110M	2.53E+02	2.13E+01	2.92E-04	0.00E+00	2.59E+06	0.00E+00	0.00E+00	2.96E+06	1.63E+07	0.00E+00	1.63E+07
Ag-110	2.82E-04	0.00E+00	0.00E+00	0.00E+00	3.70E+05	0.00E+00	0.00E+00	3.70E+05	2.22E+06	0.00E+00	2.22E+06
Te-129M	3.40E+01	3.14E+00	4.33E-05	0.00E+00	3.70E+05	0.00E+00	0.00E+00	3.70E+05	2.22E+06	0.00E+00	2.22E+06
Te-129	4.79E-02	9.44E+02	4.74E-03	0.00E+00	3.70E+05	0.00E+00	0.00E+00	3.70E+05	1.48E+06	0.00E+00	1.48E+06
Te-131M	1.25E+00	2.95E+01	3.77E-04	0.00E+00	1.85E+06	0.00E+00	0.00E+00	1.85E+06	1.15E+07	0.00E+00	1.15E+07
Te-131	1.74E-02	3.34E+02	7.66E-04	0.00E+00	3.70E+05	0.00E+00	0.00E+00	3.70E+05	2.22E+06	0.00E+00	2.22E+06

Table 3.5-7—Annual Expected Liquid Waste Releases (in Becquerels)
(Page 2 of 2)

Nuclide	Half-Life (days)	Primary (Bq/ml)	Secondary (Bq/ml)	Boron Recovery System (Bq)	Misc Wastes (Bq)	Secondary (Bq)	Turbine Building (Bq)	Total Liquid Waste Sources (Bq)	Adjusted Total (Bq/yr)	Detergent Wastes (Bq/yr)	Total (Bq/yr)
I-131	8.05E+00	7.66E+02	9.21E-03	1.26E+08	8.99E+07	0.00E+00	7.40E+05	2.17E+08	1.27E+09	0.00E+00	1.26E+09
Te-132	3.25E+00	3.02E+01	4.03E-04	0.00E+00	2.96E+06	0.00E+00	0.00E+00	2.96E+06	1.78E+07	0.00E+00	1.78E+07
I-132	9.58E-02	7.33E+03	4.96E-02	3.70E+05	5.92E+06	0.00E+00	7.40E+05	7.40E+06	4.26E+07	0.00E+00	4.44E+07
I-133	8.75E-01	2.93E+03	3.28E-02	6.85E+07	1.50E+08	0.00E+00	2.59E+06	2.21E+08	1.29E+09	0.00E+00	1.30E+09
Cs-134	7.49E+02	1.28E+02	1.80E-03	0.00E+00	1.67E+07	0.00E+00	0.00E+00	1.67E+07	9.81E+07	0.00E+00	9.62E+07
I-135	2.79E-01	7.03E+03	6.70E-02	1.92E+07	7.18E+07	0.00E+00	3.70E+06	9.47E+07	5.54E+08	0.00E+00	5.55E+08
Cs-136	1.30E+01	1.62E+01	2.28E-04	0.00E+00	1.85E+06	0.00E+00	0.00E+00	1.85E+06	1.15E+07	0.00E+00	1.15E+07
Cs-137	1.10E+04	1.69E+02	2.40E-03	0.00E+00	2.22E+07	0.00E+00	0.00E+00	2.22E+07	1.30E+08	0.00E+00	1.30E+08
Ba-137M	1.77E-03	0.00E+00	0.00E+00	0.00E+00	2.07E+07	0.00E+00	0.00E+00	2.07E+07	1.21E+08	0.00E+00	1.22E+08
Ba-140	1.28E+01	2.18E+02	2.94E-03	0.00E+00	2.66E+07	0.00E+00	0.00E+00	2.66E+07	1.56E+08	0.00E+00	1.55E+08
La-140	1.67E+00	4.74E+02	6.18E-03	0.00E+00	4.81E+07	0.00E+00	3.70E+05	4.85E+07	2.82E+08	0.00E+00	2.81E+08
Ce-141	3.24E+01	2.48E+00	3.39E-05	0.00E+00	3.70E+05	0.00E+00	0.00E+00	3.70E+05	1.85E+06	0.00E+00	1.85E+06
Ce-143	1.38E+00	5.44E+01	6.88E-04	0.00E+00	3.70E+06	0.00E+00	0.00E+00	3.70E+06	2.26E+07	0.00E+00	2.26E+07
Pr-143	1.37E+01	0.00E+00	0.00E+00	0.00E+00	3.70E+05	0.00E+00	0.00E+00	3.70E+05	1.85E+06	0.00E+00	1.85E+06
Ce-144	2.84E+02	6.40E+01	8.81E-04	0.00E+00	8.14E+06	0.00E+00	0.00E+00	8.51E+06	4.88E+07	0.00E+00	4.81E+07
Pr-144	1.20E-02	0.00E+00	0.00E+00	0.00E+00	8.14E+06	0.00E+00	0.00E+00	8.51E+06	4.88E+07	0.00E+00	4.81E+07
All Others		2.31E+04	6.99E-02	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	7.40E+05	0.00E+00	7.40E+05
Total (Except Tritium)		4.70E+04	2.93E-01	2.15E+08	9.95E+08	0.00E+00	1.22E+07	1.22E+09	7.14E+09	0.00E+00	7.03E+09
Tritium Release		6.14E+13 Becquerel per year									

Note:

0.00000 indicates that the value is less than 1.0E-05.

Table 3.5-8—Annual Gaseous Effluent Releases
(Page 1 of 4)

Nuclide	Airborne Iodine Release Rate – Ci/yr									
	Primary Coolant (µCi/gm)	Secondary Coolant (µCi/gm)	Building Ventilation				Blowdown Vent		Main Condenser Removal (Ci/yr)	Total (Ci/yr)
			Fuel Handling (Ci/yr)	Reactor (Ci/yr)	Auxiliary (Ci/yr)	Turbine (Ci/yr)	Blowdown Vent Offgas (Ci/yr)	Offgas (Ci/yr)		
I-131	2.070E-02	2.510E-07	2.7E-04	1.9E-03	6.6E-03	0.0E+00	0.0E+00	0.0E+00	0.0E+00	8.8E-03
I-133	7.917E-02	8.930E-07	1.0E-03	5.8E-03	2.5E-02	0.0E+00	0.0E+00	0.0E+00	0.0E+00	3.2E-02

Total I-131 Released via Gaseous Pathway = 180 Ci/yr

C-14 Released via Gaseous Pathway = 7.3 Ci/yr

Ar-41 Released via Gaseous Pathway = 34 Ci/yr

Nuclide	Primary Coolant (µCi/gm)	Secondary Coolant (µCi/gm)	Airborne Noble Gas Release Rate – Ci/yr											
			Gas Stripping			Building Ventilation				Blowdown Vent			Main Condenser Removal (Ci/yr)	Total (Ci/yr)
			Shutdown (Ci/yr)	Continuous (Ci/yr)	Reactor (Ci/yr)	Auxiliary (Ci/yr)	Turbine (Ci/yr)	Blowdown Vent Offgas (Ci/yr)	Offgas (Ci/yr)	Offgas (Ci/yr)				
Kr-85m	2.021E-01	2.968E-08	0.0E+00	0.0E+00	1.4E+02	4.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	2.0E+00	1.5E+02	
Kr-85	6.836E+00	9.777E-07	3.7E+03	1.4E+04	1.6E+04	1.4E+02	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	6.8E+01	3.4E+04	
Kr-87	1.888E-01	2.609E-08	0.0E+00	0.0E+00	4.7E+01	4.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	2.0E+00	5.3E+01	
Kr-88	3.530E-01	5.140E-08	0.0E+00	0.0E+00	1.7E+02	7.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	4.0E+00	1.8E+02	
Xe-131m	1.222E+00	1.735E-07	1.3E+02	4.9E+02	2.8E+03	2.6E+01	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	1.2E+01	3.5E+03	
Xe-133m	9.368E-02	1.387E-08	0.0E+00	0.0E+00	1.8E+02	2.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	1.8E+02	
Xe-133	3.760E+00	5.396E-07	5.3E+01	2.0E+02	8.2E+03	8.0E+01	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	3.7E+01	8.6E+03	
Xe-135m	1.634E-01	2.345E-08	0.0E+00	0.0E+00	9.0E+00	3.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	2.0E+00	1.4E+01	
Xe-135	1.080E+00	1.580E-07	0.0E+00	0.0E+00	1.2E+03	2.3E+01	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	1.1E+01	1.2E+03	
Xe-137	4.273E-02	6.165E-09	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	
Xe-138	1.508E-01	2.171E-08	0.0E+00	0.0E+00	8.0E+00	3.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	1.0E+00	1.2E+01	
Total Noble Gases														4.8E+04

Note:

0.0E+00 appearing in the table indicates release is less than 1.0 Ci/yr for Noble Gases, 0.0001 Ci/yr for I.

Table 3.5-8—Annual Gaseous Effluent Releases

(Page 2 of 4)

Nuclide	Airborne Particulate Release Rate – Ci/yr				
	Waste Gas System (Ci/yr)	Building Ventilation			Total (Ci/yr)
		Reactor (Ci/yr)	Auxiliary (Ci/yr)	Fuel Handling (Ci/yr)	
Cr-51	1.4E-07	9.2E-05	3.2E-06	1.8E-06	9.7E-05
Mn-54	2.1E-08	5.3E-05	7.8E-07	3.0E-06	5.7E-05
Co-57	0.0E+00	8.2E-06	0.0E+00	0.0E+00	8.2E-06
Co-58	8.7E-08	2.5E-04	1.9E-05	2.1E-04	4.8E-04
Co-60	1.4E-07	2.6E-05	5.1E-06	8.2E-05	1.1E-04
Fe-59	1.8E-08	2.7E-05	5.0E-07	0.0E+00	2.8E-05
Sr-89	4.4E-07	1.3E-04	7.5E-06	2.1E-05	1.6E-04
Sr-90	1.7E-07	5.2E-05	2.9E-06	8.0E-06	6.3E-05
Zr-95	4.8E-08	0.0E+00	1.0E-05	3.6E-08	1.0E-05
Nb-95	3.7E-08	1.8E-05	3.0E-07	2.4E-05	4.2E-05
Ru-103	3.2E-08	1.6E-05	2.3E-07	3.8E-07	1.7E-05
Ru-106	2.7E-08	0.0E+00	6.0E-08	6.9E-07	7.8E-07
Sb-125	0.0E+00	0.0E+00	3.9E-08	5.7E-07	6.1E-07
Cs-134	3.3E-07	2.5E-05	5.4E-06	1.7E-05	4.8E-05
Cs-136	5.3E-08	3.2E-05	4.8E-07	0.0E+00	3.3E-05
Cs-137	7.7E-07	5.5E-05	7.2E-06	2.7E-05	9.0E-05
Ba-140	2.3E-07	0.0E+00	4.0E-06	0.0E+00	4.2E-06
Ce-141	2.2E-08	1.3E-05	2.6E-07	4.4E-09	1.3E-05

Table 3.5-8—Annual Gaseous Effluent Releases
(Page 3 of 4)

Airborne Iodine Release Rate – Bq/yr										
Nuclide	Primary Coolant (Bq/gm)	Secondary Coolant (Bq/gm)	Building Ventilation				Blowdown		Main Condenser Removal (Ci/yr)	Total (Bq/yr)
			Fuel Handling (Bq/yr)	Reactor (Bq/yr)	Auxiliary (Bq/yr)	Turbine (Bq/yr)	Vent Offgas (Bq/yr)			
I-131	7.659E+02	9.287E-03	1.0E+07	7.0E+07	2.4E+08	0.0E+00	0.0E+00	0.0E+00	0.0E+00	3.3E+08
I-133	2.929E+03	3.304E-02	3.7E+07	2.1E+08	9.3E+08	0.0E+00	0.0E+00	0.0E+00	0.0E+00	1.2E+09

Total I-131 Released via Gaseous Pathway = 6.7E+12 Bq/yr

C-14 Released via Gaseous Pathway = 2.7E+11 Bq/yr

Ar-41 Released via Gaseous Pathway = 1.3E+12 Bq /yr

Airborne Noble Gas Release Rate – Bq/yr												
Nuclide	Primary Coolant (Bq/gm)	Secondary Coolant (Bq/gm)	Gas Stripping			Building Ventilation			Blowdown		Main Condenser Removal (Ci/yr)	Total (Bq/yr)
			Shutdown (Bq/yr)	Continuous (Bq/yr)	Reactor (Bq/yr)	Auxiliary (Bq/yr)	Turbine (Bq/yr)	Vent Offgas (Bq/yr)				
Kr-85m	7.478E+03	1.098E-03	0.0E+00	0.0E+00	5.2E+12	1.5E+11	0.0E+00	0.0E+00	0.0E+00	0.0E+00	7.4E+10	5.6E+12
Kr-85	2.529E+05	3.617E-02	1.4E+14	5.2E+14	5.9E+14	5.2E+12	0.0E+00	0.0E+00	0.0E+00	0.0E+00	2.5E+12	1.3E+15
Kr-87	6.986E+03	9.653E-04	0.0E+00	0.0E+00	1.7E+12	1.5E+11	0.0E+00	0.0E+00	0.0E+00	0.0E+00	7.4E+10	2.0E+12
Kr-88	1.306E+04	1.902E-03	0.0E+00	0.0E+00	6.3E+12	2.6E+11	0.0E+00	0.0E+00	0.0E+00	0.0E+00	1.5E+11	6.7E+12
Xe-131m	4.521E+04	6.420E-03	4.8E+12	1.8E+13	1.0E+14	9.6E+11	0.0E+00	0.0E+00	0.0E+00	0.0E+00	4.4E+11	1.3E+14
Xe-133m	3.466E+03	5.132E-04	0.0E+00	0.0E+00	6.7E+12	7.4E+10	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	6.7E+12
Xe-133	1.391E+05	1.997E-02	2.0E+12	7.4E+12	3.0E+14	3.0E+12	0.0E+00	0.0E+00	0.0E+00	0.0E+00	1.4E+12	3.2E+14
Xe-135m	6.046E+03	8.677E-04	0.0E+00	0.0E+00	3.3E+11	1.1E+11	0.0E+00	0.0E+00	0.0E+00	0.0E+00	7.4E+10	5.2E+11
Xe-135	3.996E+04	5.846E-03	0.0E+00	0.0E+00	4.4E+13	8.5E+11	0.0E+00	0.0E+00	0.0E+00	0.0E+00	4.1E+11	4.4E+13
Xe-137	1.581E+03	2.281E-04	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00	0.0E+00
Xe-138	5.580E+03	8.033E-04	0.0E+00	0.0E+00	3.0E+11	1.1E+11	0.0E+00	0.0E+00	0.0E+00	0.0E+00	3.7E+10	4.4E+11
Total Noble Gases											1.8E+15	

Note: 0.0E+00 appearing in the table indicates release is less than 3.7E+10 Bq/yr for Noble Gases, 2.7E+06 Bq/yr for I.

Table 3.5-8—Annual Gaseous Effluent Releases

(Page 4 of 4)

Nuclide	Airborne Particulate Release Rate – Bq/yr				
	Waste Gas System (Bq/yr)	Building Ventilation			Total (Bq/yr)
		Reactor (Bq/yr)	Auxiliary (Bq/yr)	Handling (Bq/yr)	
Cr-51	5.2E+03	3.4E+06	1.2E+05	6.7E+04	3.6E+06
Mn-54	7.8E+02	2.0E+06	2.9E+04	1.1E+05	2.1E+06
Co-57	0.0E+00	3.0E+05	0.0E+00	0.0E+00	3.0E+05
Co-58	3.2E+03	9.3E+06	7.0E+05	7.8E+06	1.8E+07
Co-60	5.2E+03	9.6E+05	1.9E+05	3.0E+06	4.1E+06
Fe-59	6.7E+02	1.0E+06	1.9E+04	0.0E+00	1.0E+06
Sr-89	1.6E+04	4.8E+06	2.8E+05	7.8E+05	5.9E+06
Sr-90	6.3E+03	1.9E+06	1.1E+05	3.0E+05	2.3E+06
Zr-95	1.8E+03	0.0E+00	3.7E+05	1.3E+03	3.7E+05
Nb-95	1.4E+03	6.7E+05	1.1E+04	8.9E+05	1.6E+06
Ru-103	1.2E+03	5.9E+05	8.5E+03	1.4E+04	6.3E+05
Ru-106	1.0E+03	0.0E+00	2.2E+03	2.6E+04	2.9E+04
Sb-125	0.0E+00	0.0E+00	1.4E+03	2.1E+04	2.3E+04
Cs-134	1.2E+04	9.3E+05	2.0E+05	6.3E+05	1.8E+06
Cs-136	2.0E+03	1.2E+06	1.8E+04	0.0E+00	1.2E+06
Cs-137	2.8E+04	2.0E+06	2.7E+05	1.0E+06	3.3E+06
Ba-140	8.5E+03	0.0E+00	1.5E+05	0.0E+00	1.6E+05
Ce-141	8.1E+02	4.8E+05	9.6E+03	1.6E+02	4.8E+05

Table 3.5-9—Gaseous Waste Release Source Term Inputs

Gaseous Waste Inputs	Value
There is No Continuous Stripping of Full Letdown Flow	—
Flow Rate Through Gas Stripper	1.2763 gpm (4.832 lpm)
Holdup Time for Xenon	27.7 days
Holdup Time for Krypton	1.67 days
Fill Time of Decay Tanks for the Gas Stripper	0.0 days
Primary Coolant Leak to Auxiliary Building	160. lb _m /day (73. kg/day)
Gas Waste System Particulate Release Fraction	0.01
Fuel Handling Building Iodine Release Fraction	0.1
Particulate Release Fraction	0.01
Auxiliary Building Iodine Release Fraction	0.1
Particulate Release Fraction	0.01
Containment Volume	2,800,000 ft ³ (79,000 m ³)
Frequency of Primary Coolant Degassing	2 times/yr
Primary to Secondary Leak Rate	75. lb _m /day (34. kg/day)
There is a Kidney Filter	—
Containment Atmosphere Cleanup Rate	4,100 cfm (116,100 lpm)
Purge Time of Containment	16. hours
Fraction of Iodine Bypassing Condensate Demineralizer	.67
Iodine Partition Factor (Gas/Liquid) in Steam Generator	0.01
Frequency of Containment High Volume Purges	2 times/yr
Containment -High Volume Purge Iodine Release Fraction	0.1
Particulate Release Fraction	0.01
Containment - Low Volume Purge Rate	2,970. cfm (84,100. lpm)
Containment - Low Volume Purge Iodine Release Fraction	0.1
Particulate Release Fraction	0.01
Steam Leak to Turbine Building	1,700. lbs/Hr (770. kg/hr)
Fraction Iodine Released from Blowdown Tank Vent	0.0
Percent of Iodine Removed from Air Ejector Release	0.0
There is No Onsite Laundry	—

Table 3.5-10—Annual Solid Waste Generation Volumes

Waste Type	Quantity (ft ³)	Curie Content		Shipping Volume (ft ³)		Average Curies per Package		Maximum Number of Containers
		Expected	Maximum	Expected	Maximum	Expected	Maximum	
Evaporator Concentrates	710	1.50E+02	9.12E+03	—	140	7.81E+00	4.75E+02	19.2 (a)
Spent Resins (other)	90	1.07E+03	5.23E+04	90	90	1.07E+03	5.23E+04	1.0 (b)
Spent Resins (Rad Waste Demineralizer System)	140	1.50E+02	9.12E+03	140	140	9.38E+01	5.70E+03	1.6 (b)
Wet Waste from Demineralizers	8	1.50E+02	9.12E+03	8	8	1.50E+03	9.12E+04	0.1 (b)
Waste Drum for Solids Collection from Centrifuge System of Liquid Waste Processing System	8	1.50E+02	9.12E+03	—	8	1.36E+02	8.29E+03	1.1 (a)
Filters	120	6.86E+02	6.86E+02	120	120	5.28E+02	5.28E+02	1.3 (b)
Sludge	70	1.50E+02	9.12E+03	—	35	3.75E+02	2.28E+04	0.4 (b)
Total Solid Waste Stored in Drums	1,146	2.51E+03	9.86E+04	358	541			
Mixed Waste	2	0.04	2.43	2	2	0.13	8.10	0.3 (a)
Non-Compressible Dry Active Waste (DAW)	70	2.97E-01	1.81E+01	70	70	2.97E+00	1.81E+02	0.1 (c)
Compressible DAW	1,415	6.01E+00	3.66E+02	1,415	1,415	4.29E+00	2.61E+02	1.4 (c)
Combustible DAW	5,300	3.19E+01	1.94E+03	5,300	5,300	6.02E+00	3.66E+02	5.3 (c)
Total DAW	6,785	3.82E+01	2.32E+03	varies	varies	varies	varies	varies
Overall Totals	7,933	2.55E+03	1.01E+05	Varies	varies	varies	varies	varies

Notes:

- (a) 55 gal drum
- (b) 8-120 HIC
- (c) SEALAND

Table 3.5-11— Liquid Waste Management System Tank Capacity

Description	Number of Tanks	Capacity per Tank gallons (liters)	Total Capacity Gallons (liters)
Liquid Waste Storage	2 (Group I waste)	18,500 (70,028)	37,000 (140,056)
	2 (Group II waste)	18,500 (70,028)	37,000 (140,056)
	1 (Group III waste)	18,500 (70,028)	18,500 (70,028)
Concentrate Tanks	3	9,000 (34,068)	27,000 (102,203)
Monitor Tanks	2	18,500 (70,028)	37,000 (140,056)

Table 3.5-12—Liquid Waste Management System Process Parameters

Parameter	Process Value
Design Process Capacity (Nominal) – Evaporator Section	~1,050 gal/hr (3,975 liters/hr)
Design Process Capacity (Nominal) – Centrifuge Section	~ 1,300 gal/hr (4,920 liters/hr)
Design Process Capacity (Nominal) – Demineralizer & Filtration Section	~2,400 gal/hr (9,085 liters/hr)
Maximum Group I Waste Influent Waste Stream	~26,500 gal/wk (100,310 liters/wk)
Maximum Group II Waste Influent Waste Stream	~66,100 gal/wk (250,208 liters/wk)
Maximum Group III Waste Influent Waste Stream	~17,200 gal/wk (65,107 liters/wk)

Table 3.5-13—Radioactivity Sources for the Liquid Waste System

Source	Flow Rate	Activity
Shim Bleed	110 gpd (416 liters/day)	Primary Coolant Activity (PCA)
Equipment Drains	1,730 gpd (6,549 liters/day)	PCA
Clean Wastes	9,430 gpd (35,696 liters/day)	0.001 PCA
Dirty Wastes	0.0*	Not Applicable*
Steam Generator Blowdown	628,000 gpd (2,377,239 liters/day)	Steam Activity in the Secondary System (Table 3.5-3)
Primary to Secondary Leak Rate	75 lb _m /day (34 kg/day)	Activity in the Secondary System (Table 3.5-3)
Condensate Demineralizer Flow Fraction	0.33	Activity in the Secondary System (Table 3.5-3)

Note:

* Group III waste is not normally radioactive and is being neglected to maximize concentrations.

Table 3.5-14—Radioactivity Input to the Liquid Waste System

(Page 1 of 2)

1.	Containment Purge:	
	Purge Time of Containment	16 hours
	Frequency of Containment Building High Volume Purge	2 times/year
	Containment Volume	2.8 million ft ³ (79,000 m ³)
	Containment High Volume Purge:	
	Iodide Release Fraction	0.1
	Particulate Release Fraction	0.01
	Filters:	
	Charcoal	90%
	HEPA	99%
	Containment Low Volume Purge Rate	2,970 cfm (84,100 liters/min)
	Containment Low Volume Purge:	
	Iodide Release Fraction	0.1
	Particulate Release Fraction	0.01
	Filters:	
	Charcoal	90%
	HEPA	99%
	Containment Atmospheric Cleanup:	
	Flow Rate	4,100 cfm (116,100 liters/min)
	Filters:	
	Charcoal	90%
	HEPA	99%
2.	Auxiliary Building:	
	Primary Coolant Leakage to Auxiliary Building	160 lb _m /day (73 kg/day)
	Iodide Release Fraction	0.1
	Particulate Release Fraction	0.01
	Filters:	
	Charcoal	90%
	HEPA	99%
3.	Fuel Handling Building	
	Iodide Release Fraction	0.1
	Particulate Release Fraction	0.01
	Filters:	
	Charcoal	90%
	HEPA	99%
4.	Turbine Building:	
	Steam Leakage to Turbine Building	1,700 lb _m /hr (770 kg/hr)
	Primary to Secondary Leak Rate	75 lb _m /hr (34 kg/hr)
	Iodine Partition Factor (Gas/Liquid) in Steam Generator	0.01
	Fraction of Iodine Released from Blowdown Tank Vent	0.0
	Percent of Iodine Removed from Air Ejector Release	0.0
	Fraction of Iodine Bypassing condensate Demineralizer	0.67

Table 3.5-14—Radioactivity Input to the Liquid Waste System

(Page 2 of 2)

5.	Waste Gas System:	
	Flow Rate through Gas Stripper	1.276 gpm (4.832 l/min)
	Holdup Time for Xenon	27.7 days
	Holdup Time for Krypton	1.67 days
	Gas Waste System: Particulate Release Fraction	0.01
	Frequency of Primary Coolant Degassing	2 times/year
6.	Laundry:	
	There is no onsite laundry.	

Table 3.5-15—Radioactive Liquid Releases Due to Anticipated Operational Occurrences

Nuclide	Adjusted Total	
	(Ci/yr)	(Bq/yr)
Corrosion and Activation Products		
Na-24	0.00613	2.27E+08
Cr-51	0.00103	3.81E+07
Mn-54	0.00054	2.00E+07
Fe-55	0.00041	1.52E+07
Fe-59	0.00010	3.70E+06
Co-58	0.00155	5.74E+07
Co-60	0.00018	6.66E+06
Zn-65	0.00017	6.29E+06
W-187	0.00046	1.70E+07
Np-239	0.00058	2.15E+07
Fission Products		
Sr-89	0.00005	1.85E+06
Sr-91	0.00008	2.96E+06
Y-91M	0.00005	1.85E+06
Y-93	0.00036	1.33E+07
Zr-95	0.00013	4.81E+06
Nb-95	0.00010	3.70E+06
Mo-99	0.00175	6.48E+07
Tc-99M	0.00170	6.29E+07
Ru-103	0.00251	9.29E+07
Rh-103M	0.00251	9.29E+07
Ru-106	0.03050	1.13E+09
Rh-106	0.03050	1.13E+09
Ag-110M	0.00044	1.63E+07
Ag-110	0.00006	2.22E+06
Te-129M	0.00006	2.22E+06
Te-129	0.00004	1.48E+06
Te-131M	0.00031	1.15E+07
Te-131	0.00006	2.22E+06
I-131	0.03424	1.27E+09
TE-132	0.00048	1.78E+07
I-132	0.00115	4.26E+07
I-133	0.03488	1.29E+09
CS-134	0.00265	9.81E+07
I-135	0.01496	5.54E+08
CS-136	0.00031	1.15E+07
CS-137	0.00351	1.30E+08
BA-137M	0.00328	1.21E+08
BA-140	0.00421	1.56E+08
LA-140	0.00763	2.82E+08
CE-141	0.00005	1.85E+06
CE-143	0.00061	2.26E+07
PR-143	0.00005	1.85E+06
CE-144	0.00132	4.88E+07
PR-144	0.00132	4.88E+07
All Others	0.00002	7.40E+05
Total	0.19304	7.14E+09

Table 3.5-16—{Summary of Radioactive Liquid Releases Including Anticipated Operational Occurrences}

Nuclide	Total		Discharge Concentration		10CFR20 Appendix B Limits		Discharge Fraction of Limit
	(Ci/yr)	(Bq/yr)	(μ Ci/ml)	(Bq/ml)	(μ Ci/ml)	(Bq/ml)	
Corrosion and Activation Products							
Na-24	6.1E-03	2.3E+08	5.40E-10	2.04E-05	5.0E-05	1.9E+00	1.08E-05
Cr-51	1.0E-03	3.7E+07	8.86E-11	3.28E-06	5.0E-04	1.9E+01	1.77E-07
Mn-54	5.4E-04	2.0E+07	4.78E-11	1.77E-06	3.0E-05	1.1E+00	1.59E-06
Fe-55	4.1E-04	1.5E+07	3.63E-11	1.33E-06	1.0E-04	3.7E+00	3.63E-07
Fe-59	1.0E-04	3.7E+06	8.86E-12	3.28E-07	1.0E-05	3.7E-01	8.86E-07
Co-58	1.5E-03	5.6E+07	1.33E-10	4.96E-06	2.0E-05	7.4E-01	6.64E-06
Co-60	1.8E-04	6.7E+06	1.59E-11	5.94E-07	3.0E-06	1.1E-01	5.32E-06
Zn-65	1.7E-04	6.3E+06	1.51E-11	5.58E-07	5.0E-06	1.9E-01	3.01E-06
W-187	4.6E-04	1.7E+07	4.08E-11	1.51E-06	3.0E-05	1.1E+00	1.36E-06
Np-239	5.8E-04	2.1E+07	5.14E-11	1.86E-06	2.0E-05	7.4E-01	2.57E-06
Fission Products							
Sr-89	5.0E-05	1.9E+06	4.43E-12	1.68E-07	8.0E-06	3.0E-01	5.54E-07
Sr-91	8.0E-05	3.0E+06	7.09E-12	2.66E-07	2.0E-05	7.4E-01	3.54E-07
Y-93	3.6E-04	1.3E+07	3.19E-11	1.15E-06	2.0E-05	7.4E-01	1.59E-06
Zr-95	1.3E-04	4.8E+06	1.15E-11	4.25E-07	2.0E-05	7.4E-01	5.76E-07
Nb-95	1.0E-04	3.7E+06	8.86E-12	3.28E-07	3.0E-05	1.1E+00	2.95E-07
Mo-99	1.8E-03	6.7E+07	1.59E-10	5.94E-06	2.0E-05	7.4E-01	7.97E-06
Tc-99M	1.7E-03	6.3E+07	1.51E-10	5.58E-06	1.0E-03	3.7E+01	1.51E-07
Ru-103	2.5E-03	9.3E+07	2.21E-10	8.24E-06	3.0E-05	1.1E+00	7.38E-06
Rh-103M	2.5E-03	9.3E+07	2.21E-10	8.24E-06	6.0E-03	2.2E+02	3.69E-08
Ru-106	3.1E-02	1.1E+09	2.75E-09	9.75E-05	3.0E-06	1.1E-01	9.15E-04
Ag-110M	4.4E-04	1.6E+07	3.90E-11	1.42E-06	6.0E-06	2.2E-01	6.50E-06
Te-129M	6.0E-05	2.2E+06	5.32E-12	1.95E-07	7.0E-06	2.6E-01	7.59E-07
Te-129	4.0E-05	1.5E+06	3.54E-12	1.33E-07	4.0E-04	1.5E+01	8.86E-09
Te-131M	3.1E-04	1.1E+07	2.75E-11	9.75E-07	8.0E-06	3.0E-01	3.43E-06
Te-131	6.0E-05	2.2E+06	5.32E-12	1.95E-07	8.0E-05	3.0E+00	6.64E-08
I-131	3.4E-02	1.3E+09	3.01E-09	1.15E-04	1.0E-06	3.7E-02	3.01E-03
Te-132	4.8E-04	1.8E+07	4.25E-11	1.59E-06	9.0E-06	3.3E-01	4.73E-06
I-132	1.2E-03	4.4E+07	1.06E-10	3.90E-06	1.0E-04	3.7E+00	1.06E-06
I-133	3.5E-02	1.3E+09	3.10E-09	1.15E-04	7.0E-06	2.6E-01	4.43E-04
Cs-134	2.6E-03	9.6E+07	2.30E-10	8.51E-06	9.0E-07	3.3E-02	2.56E-04
I-135	1.5E-02	5.6E+08	1.33E-09	4.96E-05	3.0E-05	1.1E+00	4.43E-05
Cs-136	3.1E-04	1.1E+07	2.75E-11	9.75E-07	6.0E-06	2.2E-01	4.58E-06
Cs-137	3.5E-03	1.3E+08	3.10E-10	1.15E-05	1.0E-06	3.7E-02	3.10E-04
Ba-140	4.2E-03	1.6E+08	3.72E-10	1.42E-05	8.0E-06	3.0E-01	4.65E-05
La-140	7.6E-03	2.8E+08	6.73E-10	2.48E-05	9.0E-06	3.3E-01	7.48E-05
Ce-141	5.0E-05	1.9E+06	4.43E-12	1.68E-07	3.0E-05	1.1E+00	1.48E-07
Ce-143	6.1E-04	2.3E+07	5.40E-11	2.04E-06	2.0E-05	7.4E-01	2.70E-06
Pr-143	5.0E-05	1.9E+06	4.43E-12	1.68E-07	2.0E-05	7.4E-01	2.21E-07
Ce-144	1.3E-03	4.8E+07	1.15E-10	4.25E-06	3.0E-06	1.1E-01	3.84E-05
Pr-144	1.3E-03	4.8E+07	1.15E-10	4.25E-06	6.0E-04	2.2E+01	1.92E-07
H-3	1.7E+03	6.1E+13	1.51E-04	5.40E+00	1.0E-03	3.7E+01	1.51E-01

Table 3.5-17—Annual Radioactive Gaseous Releases Due to Anticipated Operational Occurrences

Radionuclide	Condition 1		Condition 2		Condition 3		Condition 4	
	Total Off Normal for 0.5% Failed Fuel		Total Off Normal 600 gpd Primary-Secondary Tube Leak for 90 Days		Total Off Normal 1 gpm Reactor Coolant Leakage for 10 Days		Total Off Normal 200 gpd Reactor Coolant leakage to Aux. Building for 90 days	
	(Ci/yr)	(Bq/yr)	(Ci/yr)	(Bq/yr)	(Ci/yr)	(Bq/yr)	(Ci/yr)	(Bq/yr)
I-131	3.7E-02	1.4E+09	8.8E-03	3.2E+08	1.8E-02	6.5E+08	1.9E-02	7.0E+08
I-133	1.3E-01	4.9E+09	3.2E-02	1.2E+09	5.9E-02	2.2E+09	7.1E-02	2.6E+09
Kr-85M	6.1E+02	2.3E+13	1.6E+02	5.9E+12	8.0E+02	3.0E+13	1.5E+02	5.6E+12
Kr-85	1.4E+05	5.2E+15	3.4E+04	1.3E+15	1.1E+05	4.0E+15	3.4E+04	1.3E+15
Kr-87	2.2E+02	8.2E+12	6.7E+01	2.5E+12	2.7E+02	1.0E+13	5.9E+01	2.2E+12
Kr-88	7.5E+02	2.8E+13	2.1E+02	7.8E+12	9.8E+02	3.6E+13	1.9E+02	7.1E+12
Xe-131M	1.4E+04	5.3E+14	3.5E+03	1.3E+14	1.7E+04	6.2E+14	3.5E+03	1.3E+14
Xe-133M	7.6E+02	2.8E+13	1.8E+02	6.7E+12	1.0E+03	3.8E+13	1.9E+02	6.8E+12
Xe-133	3.6E+04	1.3E+15	8.8E+03	3.3E+14	4.7E+04	1.7E+15	8.7E+03	3.2E+14
Xe-135M	5.8E+01	2.2E+12	2.8E+01	1.0E+12	5.6E+01	2.1E+12	1.9E+01	6.9E+11
Xe-135	5.1E+03	1.9E+14	1.3E+03	4.9E+13	6.9E+03	2.5E+14	1.3E+03	4.7E+13
Xe-137	a	a	a	a	a	a	a	a
Xe-138	5.0E+01	1.9E+12	1.9E+01	7.1E+11	5.0E+01	1.8E+12	1.7E+01	6.2E+11
Cr-51	4.0E-04	1.5E+07	9.7E-05	3.6E+06	5.3E-04	2.0E+07	1.0E-04	3.8E+06
Mn-54	2.4E-04	8.8E+06	5.7E-05	2.1E+06	3.1E-04	1.1E+07	5.8E-05	2.1E+06
Co-57	3.4E-05	1.3E+06	8.2E-06	3.0E+05	4.7E-05	1.7E+06	8.2E-06	3.0E+05
Co-58	2.0E-03	7.4E+07	4.8E-04	1.8E+07	1.7E-03	6.1E+07	5.1E-04	1.9E+07
Co-60	4.7E-04	1.7E+07	1.1E-04	4.2E+06	2.4E-04	8.7E+06	1.2E-04	4.5E+06
Fe-59	1.1E-04	4.2E+06	2.8E-05	1.0E+06	1.5E-04	5.7E+06	2.8E-05	1.0E+06
Sr-89	6.6E-04	2.5E+07	1.6E-04	5.9E+06	7.7E-04	2.8E+07	1.7E-04	6.3E+06
Sr-90	2.6E-04	9.7E+06	6.3E-05	2.3E+06	3.1E-04	1.1E+07	6.8E-05	2.5E+06
Zr-95	4.2E-05	1.6E+06	1.0E-05	3.7E+05	1.0E-05	3.7E+05	2.6E-05	9.5E+05
Nb-95	1.8E-04	6.5E+06	4.2E-05	1.6E+06	1.3E-04	4.7E+06	4.3E-05	1.6E+06
Ru-103	6.9E-05	2.6E+06	1.7E-05	6.2E+05	9.2E-05	3.4E+06	1.7E-05	6.3E+05
Ru-106	3.2E-06	1.2E+05	7.8E-07	2.9E+04	7.8E-07	2.9E+04	8.7E-07	3.2E+04
Sb-125	2.5E-06	9.4E+04	6.1E-07	2.3E+04	6.1E-07	2.3E+04	6.7E-07	2.5E+04
Cs-134	2.0E-04	7.4E+06	4.8E-05	1.8E+06	1.7E-04	6.1E+06	5.6E-05	2.1E+06
Cs-136	1.4E-04	5.0E+06	3.3E-05	1.2E+06	1.8E-04	6.8E+06	3.3E-05	1.2E+06
Cs-137	3.7E-04	1.4E+07	9.0E-05	3.3E+06	3.5E-04	1.3E+07	1.0E-04	3.7E+06
Ba-140	1.8E-05	6.5E+05	4.2E-06	1.6E+05	4.2E-06	1.6E+05	1.0E-05	3.9E+05
Ce-141	5.5E-05	2.0E+06	1.3E-05	4.9E+05	7.4E-05	2.8E+06	1.4E-05	5.1E+05

Note: (a) Less than 1.0 Ci/yr for noble gases

Table 3.5-18—Obtainable Dose Benefits for Gaseous Waste System Augment

Cases	Population Total Body Dose¹- Person-Rem (Person-Sievert)	Population Thyroid Dose⁽¹⁾ Person-Rem (Person-Sievert)
Baseline Configuration	0.375(0.00375)	0.421 (0.00421)
Extra Carbon Delay Bed	0.373 (0.00373)	0.419 (0.00419)
Obtainable dose benefit by augment	0.002 (0.00002)	0.002 (0.00002)

Note:

⁽¹⁾ Population dose estimates described in Section 5.4.

Table 3.5-19—{Gaseous Waste System Augment Thyroid Dose Cost Benefit Analysis}

Parameter	Value
Annual whole-body / Thyroid collective dose benefit to the population within 50 miles of the Callaway Plant site (0.002 person-rem x 60 yr = 0.12 person-rem).	0.128 person-rem (0.0012 person-sievert)
Nominal total collective dose over 60 years of operation (0.42 person-rem x 60 yr = 25.3 person-rem)	25.3 person-rem (0.253 person-sievert)
Value for estimating impact based on NUREG-1530	\$2,000 per person-rem (\$200,000 per person-sievert)
Obtainable benefit from addition of radwaste processing and control option (0.12 person-rem x \$2,000/person-rem = \$240)	\$240
Cost Options for radwaste processing and control technology upgrade from Regulatory Guide 1.110	3-ton charcoal absorber
Direct cost for option (using methodology in Regulatory Guide 1.110, Table A-1 based on 1975 Dollars)	\$67,000
Total O&M Annual Cost (From Regulatory Guide 1.110, Table A-2 based on 1975 Dollars)	Negligible
Total cost over 60 years of operation (direct cost + O&Mx60 years)	\$67,000
Benefit/Cost Ratio (Values greater than 1 should be included in plant system design) \$240 / \$67,000 = 0.004	0.004

Note:

Since the dose reduction benefit for both the total body and the thyroid give the same collective dose savings, the cost benefit results are directly applicable to both the total body and thyroid evaluations.

Table 3.5-20—Radiation Monitors

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Method	Monitoring Task	Radioisotopes	Range
Noble Gas Effluent Monitors			
Beta-sensitive detector (β)	Primary Coolant Beta Activity Monitor, Nuclear Auxiliary Building	Kr-85, Xe-133	3E-4 – 3E+2 μ Ci/cc 1E+1 – 1E+7 Bq/cc
Gamma-sensitive detector (γ)	Nuclear Auxiliary Building Ventilation System Beta Activity Cell 1	Kr-85, Xe-133	3E-7 – 1E-2 μ Ci/cc 1E-2 – 4E+2 Bq/cc
Gamma-sensitive detector (γ)	Noble gas radioactivity in the exhaust air of ventilation systems of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5) and Safeguard Buildings (cell 6).	Kr-85, Xe-133	3E-7 – 1E-2 μ Ci/cc 1E-2 – 4E+2 Bq/cc
Gamma-sensitive detector (γ)	Noble gas radioactivity in the exhaust air of ventilation systems of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5) and Safeguard Buildings (cell 6).	Kr-85, Xe-133	3E-7 – 1E-2 μ Ci/cc 1E-2 – 4E+2 Bq/cc
Gamma-sensitive detector (γ)	Noble gas radioactivity in the exhaust air of ventilation systems of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5) and Safeguard Buildings (cell 6).	Kr-85, Xe-133	3E-7 – 1E-2 μ Ci/cc 1E-2 – 4E+2 Bq/cc
Gamma-sensitive detector (γ)	Noble gas radioactivity in the exhaust air of ventilation systems of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5) and Safeguard Buildings (cell 6).	Kr-85, Xe-133	3E-7 – 1E-2 μ Ci/cc 1E-2 – 4E+2 Bq/cc
Gamma-sensitive detector (γ)	Noble gas radioactivity in the exhaust air of ventilation systems of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5) and Safeguard Buildings (cell 6).	Kr-85, Xe-133	3E-7 – 1E-2 μ Ci/cc 1E-2 – 4E+2 Bq/cc
Beta-sensitive detector (β)	Noble gas radioactivity in the exhaust air containment ventilation.	Kr-85, Xe-133	3E-7 – 1E-2 μ Ci/cc 1E-2 – 4E+2 Bq/cc
Beta-sensitive detector (β)	Noble gas activity in the region of the refueling machine within the containment while moving fuel assemblies.	Kr-85, Xe-133	1E-6 – 1E-2 μ Ci/cc 1E-2 – 4E+2 Bq/cc
Beta-sensitive detector (β)	Noble gas activity in the region of the spent fuel mast bridge within the fuel building while moving fuel assemblies.	Kr-85, Xe-133	1E-6 – 1E-2 μ Ci/cc 4E-2 – 4E+2 Bq/cc
Beta-sensitive detector (β)	Noble gas activity in the vent stack located in the Nuclear Auxiliary Building	Kr-85, Xe-133	1E-6 – 1E+2 μ Ci/cc 4E-2 – 4E+2 Bq/cc, 3E+4 – 1E+9 μ Ci /hr 1E+9 – 4E+13 Bq/hr
Calculated	Noble gas activity release rate is calculated using the measured values from noble gas activity rad. monitor and the air flow through the vent stack.	Kr-85, Xe-133	3E-6 – 1E-2 μ Ci/cc 1E-1 – 4E+2 Bq/cc, 3E+4 – 1E+9 μ Ci /hr 1E+9 – 4E+13 Bq/hr
Beta-sensitive detector (β)	Noble gas activity in the vent stack.	Kr-85, Xe-133	3E-6 – 1E-2 μ Ci/cc 1E-1 – 4E+2 Bq/cc, 3E+4 – 1E+9 μ Ci /hr 1E+9 – 4E+13 Bq/hr
Calculated	Noble gas activity release rate is calculated using the measured values from noble gas activity rad. monitor and the air flow through the vent stack.	Kr-85, Xe-133	3E-6 – 1E-2 μ Ci/cc 1E-1 – 4E+2 Bq/cc, 3E+4 – 1E+9 μ Ci /hr 1E+9 – 4E+13 Bq/hr
Gamma-sensitive multi-channel analyzer (γ)	Noble gas activity in the vent stack.	Xe-133	1 – 50000 cps
Gamma-sensitive multi-channel analyzer (γ)	Noble gas activity in the vent stack.	Xe-133	3E-6 – 1E-2 μ Ci/cc 1E-1 – 4E+2 Bq/cc, 3E+4 – 1E+9 μ Ci /hr 1E+9 – 4E+13 Bq/hr
Laboratory evaluation of samples.	Vent stack exhaust air using a sample drawn on demand. Samples are analyzed in the laboratory by gamma spectroscopic evaluation.	Kr-85, Xe-133	–

Table 3.5-20—Radiation Monitors

(Page 2 of 12)

Method	Monitoring Task	Radioisotopes	Range
Laboratory evaluation of samples.	Vent stack exhaust air using a sample drawn on demand. Samples are analyzed in the laboratory by gamma spectroscopic evaluation.	Kr-85, Xe-133	–
Laboratory evaluation of samples.	Vent stack exhaust air during and after accidents. Samples are analyzed in the laboratory by gamma spectroscopic evaluation.	Kr-85, Xe-133	–
Laboratory evaluation of samples.	Vent stack exhaust air during and after accidents. Samples are analyzed in the laboratory by gamma spectroscopic evaluation.	Kr-85, Xe-133	–
Gamma-sensitive detectors adjacent to the monitored air duct (γ)	Annulus air extraction system downstream of the filters. The instrument is to function also during a severe accident.	Kr-85, Xe-133	1E-4 — 1E+4 rad/hr 1E-6 – 1E+2 Gy/hr
Gamma-sensitive detectors adjacent to the monitored air duct (γ)	Annulus air extraction system downstream of the filters. The instrument is to function also during a severe accident.	Kr-85, Xe-133	1E-4 — 1E+4 rad/hr 1E-6 – 1E+2 Gy/hr
Gamma-sensitive detectors adjacent to the monitored air duct (γ)	Safeguard Building controlled-area ventilation system downstream of the filters. Instrument is functional during a severe accident.	Kr-85, Xe-133	1E-4 — 1E+4 rad/hr 1E-6 – 1E+2 Gy/hr
Gamma-sensitive detectors adjacent to the monitored air duct (γ)	Safeguard Building controlled-area ventilation system downstream of the filters. Instrument is functional during a severe accident.	Kr-85, Xe-133	1E-4 – 1E+4 rad/hr 1E-6 – 1E+2 Gy/hr
Gamma-sensitive detectors inside the stack (γ)	Vent Stack gas activity discharges during accidents. Instrument is functional during a severe accident.	Kr-85, Xe-133	1E-4 – 1E+4 rad/hr 1E-6 – 1E+2 Gy/hr
Gamma-sensitive detectors inside the stack (γ)	Vent Stack gas activity discharges during accidents. Instrument is functional during a severe accident.	Kr-85, Xe-133	1E-4 – 1E+4 rad/hr 1E-6 – 1E+2 Gy/hr
Iodine and Aerosol (Halogen and Particulate) Monitoring			
Laboratory evaluation of samples.	Aerosol and the gaseous iodine in the air of the annulus air extraction system downstream of the filters. A filter cartridge is evaluated in the laboratory.	Iodine	–
Laboratory evaluation of samples.	Aerosol and the gaseous iodine in the air of the annulus air extraction system downstream of the filters. A filter cartridge is evaluated in the laboratory.	Iodine	–
Laboratory evaluation of samples.	Aerosol and the gaseous iodine in the air of the safeguard building controlled-area ventilation system down-stream of the filters. A filter cartridge is evaluated in the laboratory.	–	–
Laboratory evaluation of samples.	Aerosol and the gaseous iodine in the air of the safeguard building controlled-area ventilation system down-stream of the filters. The filter cartridge is evaluated in the laboratory.	Iodine	–
Laboratory evaluation of samples.	Aerosol and the gaseous iodine in the exhaust air of the access building using a filter cartridge evaluated in the laboratory.	I-131	–
Laboratory evaluation of samples.	Aerosol and the gaseous iodine in the exhaust air of the access building using a filter cartridge evaluated in the laboratory.	I-131	–
Laboratory evaluation of samples.	Aerosol and the gaseous iodine down-stream of filters of the laboratory exhaust air in the Nuclear Auxiliary Building using a filter cartridge evaluated in the laboratory.	I-131	–

Table 3.5-20—Radiation Monitors

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Method	Monitoring Task	Radioisotopes	Range
Laboratory evaluation of samples.	Aerosol and the gaseous iodine down-stream of filters of the laboratory exhaust air in the Nuclear Auxiliary Building using a filter cartridge evaluated in the laboratory.	I-131	–
Laboratory evaluation of samples.	Aerosol and the gaseous iodine in the filtered system exhaust air of the radwaste building using a filter cartridge evaluated in the laboratory.	–	–
Laboratory evaluation of samples.	Aerosol and the gaseous iodine in the filtered system exhaust air of the radwaste building using a filter cartridge evaluated in the laboratory.	Iodine	–
Gamma-sensitive detector (γ)	Aerosol and gaseous iodine by continuous collection from exhaust air on filters during and after accidents in the Nuclear Auxiliary Building There are three ranges given; one for all activity, one for just I-131, and one for the remaining iodine isotopes.	I-131	5E-10 – 3E-2 μ Ci 2E-5 – 1E+3 Bq (entire activity) 5E-10 – 5E-4 μ Ci /cc 2E-5 – 2E+1 Bq/cc (I-131) 3E-9 – 5E-3 μ Ci /cc 1E-4 – 2E+2 Bq/cc (Iodine, less I-131)
Gamma-sensitive detector (threshold 350 keV). Alternatively aerosol radioactivity is monitored continuously by a beta-sensitive detector (β, γ)	Aerosol activity in the vent stack.	--	5E-10 – 3E-6 μ Ci 2E-5 – 1E-1 Bq, 1E-10 – 1E-6 μ Ci/cc 4E-6 – 4E-2 Bq/cc
Gamma-sensitive detector (γ)	Gaseous iodine activity (I-131) in the vent stack. In order to obtain the I-131 release the I-131 concentration in the vent stack is linked with the air flow through the stack.	–	3E-10 – 3E-6 μ Ci 1E-5 – 1E-1 Bq, 5E-11 – 3E-7 μ Ci/cc 2E-6 – 1E-2 Bq/cc, 1E+1 – 1E+4 μ Ci/hr 4E+5 – 4E+8 Bq/hr
Calculated release	Calculate the I-131 release using gaseous iodine activity in the vent stack and the stack air flow.	–	3E-10 – 3E-6 μ Ci 1E-5 – 1E-1
Gamma-sensitive detector (γ)	Aerosol and gaseous iodine by continuous collection from exhaust air on filters during and after accidents in the Nuclear Auxiliary Building.	I-131	5E-10 – 3E-2 μ Ci 2E-5 – 1E+3 Bq (entire activity), 5E-10 – 5E-4 μ Ci /cc 2E-5 – 2E+1 Bq/cc (I-131), 3E-9 – 5E-3 μ Ci /cc 1E-4 – 2E+2 Bq/cc (Iodine, less I-131)
Laboratory evaluation of samples	Samples from the vent stack exhaust air drawn continuously through redundant cartridges for organic and elemental iodine.	I-131	–
Laboratory evaluation of samples	Samples from the vent stack exhaust air drawn continuously through redundant cartridges for organic and elemental iodine.	I-131	–
Laboratory evaluation of samples	Samples from the vent stack exhaust air drawn continuously through redundant cartridges for organic and elemental iodine.	I-131	–
Laboratory evaluation of samples	Samples from the vent stack exhaust air drawn continuously through redundant cartridges for organic and elemental iodine.	I-131	–

Table 3.5-20—Radiation Monitors

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Method	Monitoring Task	Radioisotopes	Range
Laboratory evaluation of samples	Samples from the vent stack exhaust air drawn continuously through redundant cartridges for organic and elemental iodine. Functions before, during, and after an abnormal event.	I-131	–
Laboratory evaluation of samples	Samples from the vent stack exhaust air drawn continuously through redundant cartridges for organic and elemental iodine. Intended to function before, during, and after an abnormal event.	I-131	–
Laboratory evaluation of samples	Samples from the vent stack exhaust air drawn continuously through redundant cartridges for organic and elemental iodine. Intended to function before, during, and after an abnormal event.	I-131	–
Laboratory evaluation of samples	Samples from the vent stack exhaust air drawn continuously through redundant cartridges for organic and elemental iodine. Intended to function before, during, and after an abnormal event.	I-131	–
Laboratory evaluation of samples	Vent stack exhaust air including vapor, carbon dioxide and the other carbon compounds continuously. Redundant samples are evaluated in the laboratory for H-3 and C-14.	H-3 and C-14	–
Laboratory evaluation of samples	Vent stack exhaust air including vapor, carbon dioxide and the other carbon compounds continuously. Redundant samples are evaluated in the laboratory for H-3 and C-14.	H-3 and C-14	–
Process Monitors			
Gamma-sensitive detector (γ)	General area radiation level of the fuel pool floor. Assessing accessibility after abnormal events, fuel pool floor.	–	1E-4 – 1E+4 rem/hr 1E-6 – 1E+2 Sv/hr
Gamma-sensitive detector (γ)	All small items, tools etc. brought out of the controlled area are measured and released by an automatic release box in the Access Building.	Co-60, Cs-137	Co-60: 40% Cs-137: 22%
Gamma-sensitive detector (γ)	All small items, tools etc. brought out of the controlled area are measured and released by an automatic release box in the Access Building.	Co-60, Cs-137	Co-60: 40% Cs-137: 22%
Gamma-sensitive detector (γ)	All small items, tools etc. brought out of the controlled area are measured and released by an automatic release box in the Access Building.	Co-60, Cs-137	Co-60: 40% Cs-137: 22%
Gamma-sensitive detector (γ)	All small items, tools etc. brought out of the controlled area are measured and released by an automatic release box in the Access Building.	Co-60, Cs-137	Co-60: 40% Cs-137: 22%
Alpha- and beta-sensitive detectors and gamma-sensitive detectors (α, β, γ)	Personnel leaving the controlled area are controlled with regard to contamination with two step monitors. Exit monitors are equipped with gamma-sensitive detectors to detect gamma radiation and/or incorporated gamma nuclides in the Access Building.	Co-60, Cs-137	Co-60: 28% Cs-137: 40%
Alpha- and beta-sensitive detectors and gamma-sensitive detectors (α, β, γ)	Personnel leaving the controlled area are controlled with regard to contamination with two step monitors. Exit monitors are equipped with gamma-sensitive detectors to detect gamma radiation and/or incorporated gamma nuclides in the Access Building.	Co-60, Cs-137	Co-60: 28% Cs-137: 40%
Alpha- and beta-sensitive detectors and gamma-sensitive detectors (α, β, γ)	Personnel leaving the controlled area are controlled with regard to contamination with two step monitors. Exit monitors are equipped with gamma-sensitive detectors to detect gamma radiation and/or incorporated gamma nuclides in the Access Building.	Co-60, Cs-137	Co-60: 28% Cs-137: 40%

Table 3.5-20—Radiation Monitors

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Method	Monitoring Task	Radioisotopes	Range
Alpha- and beta-sensitive detectors and gamma-sensitive detectors (α, β, γ)	Personnel leaving the controlled area are controlled with regard to contamination with two step monitors. Exit monitors are equipped with gamma-sensitive detectors to detect gamma radiation and/or incorporated gamma nuclides in the Access Building.	Co-60, Cs-137	Co-60: 28% Cs-137: 40%
Alpha- and beta-sensitive detectors and gamma-sensitive detectors (α, β, γ)	Personnel leaving the controlled area are controlled with regard to contamination with two step monitors. Exit monitors are equipped with gamma-sensitive detectors to detect gamma radiation and/or incorporated gamma nuclides in the Access Building.	Co-60, Cs-137	Co-60: 28% Cs-137: 40%
Alpha- and beta-sensitive detectors and gamma-sensitive detectors (α, β, γ)	Personnel leaving the controlled area are controlled with regard to contamination with two step monitors. Exit monitors are equipped with gamma-sensitive detectors to detect gamma radiation and/or incorporated gamma nuclides in the Access Building.	Co-60, Cs-137	Co-60: 28% Cs-137: 40%
Alpha- and beta-sensitive detectors and gamma-sensitive detectors (α, β, γ)	Personnel leaving the controlled area are controlled with regard to contamination with two step monitors. Exit monitors are equipped with gamma-sensitive detectors to detect gamma radiation and/or incorporated gamma nuclides in the Access Building.	Co-60, Cs-137	Co-60: 28% Cs-137: 40%
Alpha- and beta-sensitive detectors and gamma-sensitive detectors (α, β, γ)	Personnel leaving the controlled area are controlled with regard to contamination with two step monitors. Exit monitors are equipped with gamma-sensitive detectors to detect gamma radiation and/or incorporated gamma nuclides in the Access Building.	Co-60, Cs-137	Co-60: 28% Cs-137: 40%
Gamma-sensitive electronic personnel dosimeter (γ)	At the entrance and exit of the controlled area the personnel dosimeters are read by dosimeter readers. The measured dose values together with personal identification codes are evaluated by the dosimetry system of the plant in the Access Building.	–	60 keV – 6 MeV Energy range 1E-4 – 1E+3 rem 1E-6 – 1E+1 Sv Dose range
Gamma-sensitive electronic personnel dosimeter (γ)	At the entrance and exit of the controlled area the personnel dosimeters are read by dosimeter readers. The measured dose values together with personal identification codes are evaluated by the dosimetry system of the plant in the Access Building.	–	60 keV – 6 MeV Energy range 1E-4 – 1E+3 rem 1E-6 – 1E+1 Sv Dose range
Gamma-sensitive electronic personnel dosimeter (γ)	At the entrance and exit of the controlled area the personnel dosimeters are read by dosimeter readers. The measured dose values together with personal identification codes are evaluated by the dosimetry system of the plant in the Access Building.	–	60 keV – 6 MeV Energy range 1E-4 – 1E+3 rem 1E-6 – 1E+1 Sv Dose range
Gamma-sensitive electronic personnel dosimeter (γ)	At the entrance and exit of the controlled area the personnel dosimeters are read by dosimeter readers. The measured dose values together with personal identification codes are evaluated by the dosimetry system of the plant in the Access Building.	–	60 keV – 6 MeV Energy range 1E-4 – 1E+3 rem 1E-6 – 1E+1 Sv Dose range
Gamma-sensitive detector (γ)	Decontamination room in the Radioactive Waste Processing Building.	–	1E-4 – 1E+1 rem/hr 1E-6 – 1E-1 Sv/hr
Integral gamma-measurement with a gamma-sensitive detector (threshold 100 keV) (γ)	Component cooling loop in the Reactor Building.	–	1E-6 – 1E-3 μ Ci/ml 4E-2 – 4E+1 Bq/ml

Table 3.5-20—Radiation Monitors

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Method	Monitoring Task	Radioisotopes	Range
Integral gamma-measurement with a gamma-sensitive detector (threshold 100 keV) (γ)	Component cooling loop in the Reactor Building.	–	1E-6 – 1E-3 μ Ci/ml 4E-2 – 4E+1 Bq/ml
Integral gamma-measurement with a gamma-sensitive detector (threshold 100 keV) (γ)	Component cooling loop in the Reactor Building.	–	1E-6 – 1E-3 μ Ci/ml 4E-2 – 4E+1 Bq/ml
Integral gamma-measurement with a gamma-sensitive detector (threshold 100 keV) (γ)	Component cooling loop in the Reactor Building.	–	1E-6 – 1E-3 μ Ci/ml 4E-2 – 4E+1 Bq/ml
Measurement with gamma-sensitive detectors (γ)	High-pressure coolers of the volume control system. Detectors are installed at the component cooling water inlet and outlet of each HP cooler. The purpose is to detect a leak from the primary side to the component cooling water side.	–	3E-5 – 3E+0 μ Ci/ml 1E+0 – 1E+5 Bq/ml
Measurement with gamma-sensitive detectors (γ)	High-pressure coolers of the volume control system. Detectors are installed at the component cooling water inlet and outlet of each HP cooler. The purpose is to detect a leak from the primary side to the component cooling water side.	–	3E-5 – 3E+0 μ Ci/ml 1E+0 – 1E+5 Bq/ml
Measurement with gamma-sensitive detectors (γ)	High-pressure coolers of the volume control system. The detectors are installed at the component cooling water inlet and outlet of each HP cooler. The purpose is to detect a leak from the primary side to the component cooling water side.	–	3E-5 – 3E+0 μ Ci/ml 1E+0 – 1E+5 Bq/ml
Measurement with gamma-sensitive detectors (γ)	High-pressure coolers of the volume control system. The detectors are installed at the component cooling water inlet and outlet of each HP cooler. The purpose is to detect a leak from the primary side to the component cooling water side.	–	3E-5 – 3E+0 μ Ci/ml 1E+0 – 1E+5 Bq/ml
Measuring arrangement of gamma detectors (γ)	Dose rate level at the top of the drum (in 10 cm distance) while the drum is rotated slowly in the Radioactive Waste Processing Building.	–	1E-4 – 1 rem/hr 1E-6 – 1E-2 Sv/hr
Measuring arrangement of gamma detectors (γ)	Dose rate level at the bottom of the drum (in 10 cm distance) while the drum is rotated slowly in the Radioactive Waste Processing Building.	–	1E-4 – 1 rem/hr 1E-6 – 1E-2 Sv/hr
Measuring arrangement of gamma detectors (γ)	Dose rate level at the shell of the drum (upper area, in 10 cm distance) while the drum is rotated slowly in the Radioactive Waste Processing Building.	–	1E-4 – 1 rem/hr 1E-6 – 1E-2 Sv/hr
Measuring arrangement of gamma detectors (γ)	Dose rate level at the shell of the drum (middle area, in 10 cm distance) while the drum is rotated slowly in the Radioactive Waste Processing Building.	–	1E-4 – 1 rem/hr 1E-6 – 1E-2 Sv/hr
Measuring arrangement of gamma detectors (γ)	Dose rate level at the shell of the drum (lower area, in 10 cm distance) while the drum is rotated slowly in the Radioactive Waste Processing Building.	–	1E-4 – 1 rem/hr 1E-6 – 1E-2 Sv/hr

Table 3.5-20—Radiation Monitors

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Method	Monitoring Task	Radioisotopes	Range
Measuring arrangement of gamma detectors (γ)	Dose rate level in 1 m distance of the drum while the drum is rotated slowly in the Radioactive Waste Processing Building.	–	1E-4 – 1 rem/hr 1E-6 – 1E-2 Sv/hr
Measuring arrangement of gamma detectors (γ)	Dose rate level in the vicinity of the drum measuring equipment as back ground measurement (in absence of a waste drum) in the Radioactive Waste Processing Building.	–	1E-4 – 1 rem/hr 1E-6 – 1E-2 Sv/hr
Gamma spectrometer with multi channel analyzer (γ)	Gamma spectroscopy system for a 200 liter drum in the Radioactive Waste Processing Building.	–	–
Gamma-sensitive detector (γ)	Upstream activity entering the delay beds of the gaseous waste disposal system in the Nuclear Auxiliary Building.	–	1E+0 – 1E+4 cps
Beta-sensitive detector (β)	Pipe leading from the gas delay line to the vent stack in the Nuclear Auxiliary Building.	–	1E-6 – 1E+2 μ Ci/ml 4E-2 – 4E+6 μ Sv/ml
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps

Table 3.5-20—Radiation Monitors

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Method	Monitoring Task	Radioisotopes	Range
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Gamma-sensitive detectors (γ)	Main steam system to detect leakage in the steam generator. This is monitored by four redundant instruments. The detectors are mounted adjacent to the monitored main steam lines within the main steam and feedwater valve compartments.	N-16	1E-1 – 1E+4 cps
Integral measurement with gamma-sensitive detector (threshold 100 keV) and ring vessel (γ)	Blowdown water of each individual steam generator located in the Nuclear Auxiliary Building.	–	3E-6 – 1E-2 μ Ci/ml 1E-1 – 4E+2 Bq/ml
Integral measurement with gamma-sensitive detector (threshold 100 keV) and ring vessel (γ)	Blowdown water of each individual steam generator located in the Nuclear Auxiliary Building.	–	3E-6 – 1E-2 μ Ci/ml 1E-1 – 4E+2 Bq/ml
Integral measurement with gamma-sensitive detector (threshold 100 keV) and ring vessel (γ)	Blowdown water of each individual steam generator located in the Nuclear Auxiliary Building.	–	3E-6 – 1E-2 μ Ci/ml 1E-1 – 4E+2 Bq/ml
Integral measurement with gamma-sensitive detector (threshold 100 keV) and ring vessel (γ)	Blowdown water of each individual steam generator located in the Nuclear Auxiliary Building.	–	3E-6 – 1E-2 μ Ci/ml 1E-1 – 4E+2 Bq/ml
Gamma-sensitive detector (γ)	Containment High Range Dose	–	1E-1 – 1E+7 rad/hr 1E-3 – 1E+5 Gy/hr
Gamma-sensitive detector (γ)	Containment High Range Dose	–	1E-1 – 1E+7 rad/hr 1E-3 – 1E+5 Gy/hr

Table 3.5-20—Radiation Monitors

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Method	Monitoring Task	Radioisotopes	Range
Beta-sensitive detector (β)	Turbine Building Main Condenser Beta Activity	–	3E-6 – 1E-2 μ Ci/cc 1E-1 – 4E+2 Bq/cc
Liquid Effluent Monitoring			
Gamma-sensitive detector (threshold 100 keV) (γ)	Liquid radwaste release line from the monitor tanks. This redundant instrument provides input to a control function.	Cs-137	5E-6 – 1E-3 μ Ci/ml 2E-1 – 4E+1 Bq/ml
Gamma-sensitive detector (threshold 100 keV) (γ)	Liquid radwaste release line from the monitor tanks. This redundant instrument provides input to a control function.	Cs-137	5E-6 – 1E-3 μ Ci/ml 2E-1 – 4E+1 Bq/ml
Integral measurement with gamma-sensitive detector (γ)	Liquid effluent from the Plant Drainage System before discharge.	–	3E-6 – 1E-2 μ Ci/ml 1E-1 – 4E+2 Bq/ml
Integral measurement with gamma-sensitive detector (γ)	Liquid effluent from the Plant Drainage System before discharge.	–	3E-6 – 1E-2 μ Ci/ml 1E-1 – 4E+2 Bq/ml
Airborne Monitoring			
Measurement with a gamma-sensitive detector (threshold 350 keV). Alternatively aerosol radioactivity is monitored continuously by a beta-sensitive detector (β, γ)	Aerosol in the exhaust air of containment ventilation.	–	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq
Gamma-sensitive detector (γ)	Gaseous iodine in the exhaust air of containment ventilation.	I-131	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq
Beta-sensitive detector (β)	Tritium in the exhaust air of containment ventilation.	–	3E-9 – 3E-4 μ Ci/cc 1E-4 – 1E+1 Bq
Gamma-sensitive detector (γ)	Air leaving the containment adjacent to a monitored air duct. This redundant instrument provides input to a control function.	–	1E-5 – 1E+0 rad/hr 1E-7 – 1E-2 Gy/hr
Measurement with gamma-sensitive detectors adjacent to monitored air duct KLA2 (γ)	Air leaving the containment adjacent to a monitored air duct. This redundant instrument provides input to a control function.		1E-5 – 1E+0 rad/hr 1E-7 – 1E-2 Gy/hr
Gamma-sensitive detector (threshold 350 keV). Alternatively aerosol radioactivity is monitored continuously by a beta-sensitive detector (β, γ)	Aerosol in the exhaust air of the Nuclear Auxiliary Building E (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building - (two cells).	–	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq/cc
Gamma-sensitive detector (γ)	Gaseous iodine in the exhaust air of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building - (two cells).	I-131	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq/cc

Table 3.5-20—Radiation Monitors

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Method	Monitoring Task	Radioisotopes	Range
Gamma-sensitive detector (threshold 350 keV). Alternatively aerosol radioactivity is monitored continuously by a beta-sensitive detector (β,γ)	Aerosol in the exhaust air of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building – (two cells).	–	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq/cc
Gamma-sensitive detector (γ)	Gaseous iodine in the exhaust air of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building – (two cells).	I-131	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 5E-8 μ Ci/cc 1E-5 – 2E-3 Bq/cc
Gamma-sensitive detector (threshold 350 keV). Alternatively aerosol radioactivity is monitored continuously by a beta-sensitive detector (β,γ)	Aerosol in the exhaust air of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building – (two cells).	–	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq/cc
Gamma-sensitive detector (γ)	Gaseous iodine in the exhaust air of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building – (two cells).	I-131	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 5E-8 μ Ci/cc 1E-5 – 2E-3 Bq/cc
Gamma-sensitive detector (threshold 350 keV). Alternatively aerosol radioactivity is monitored continuously by a beta-sensitive detector (β,γ)	Aerosol in the exhaust air of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building – (two cells).	–	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq/cc
Gamma-sensitive detector (γ)	Gaseous iodine in the exhaust air of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building – (two cells).	I-131	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 5E-8 μ Ci/cc 1E-5 – 2E-3 Bq/cc
Gamma-sensitive detector (threshold 350 keV). Alternatively aerosol radioactivity is monitored continuously by a beta-sensitive detector (β,γ)	Aerosol in the exhaust air of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building – (two cells).	–	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq/cc
Gamma-sensitive detector (γ)	Gaseous iodine in the exhaust air of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building – (two cells).	I-131	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 5E-8 μ Ci/cc 1E-5 – 2E-3 Bq/cc

Table 3.5-20—Radiation Monitors

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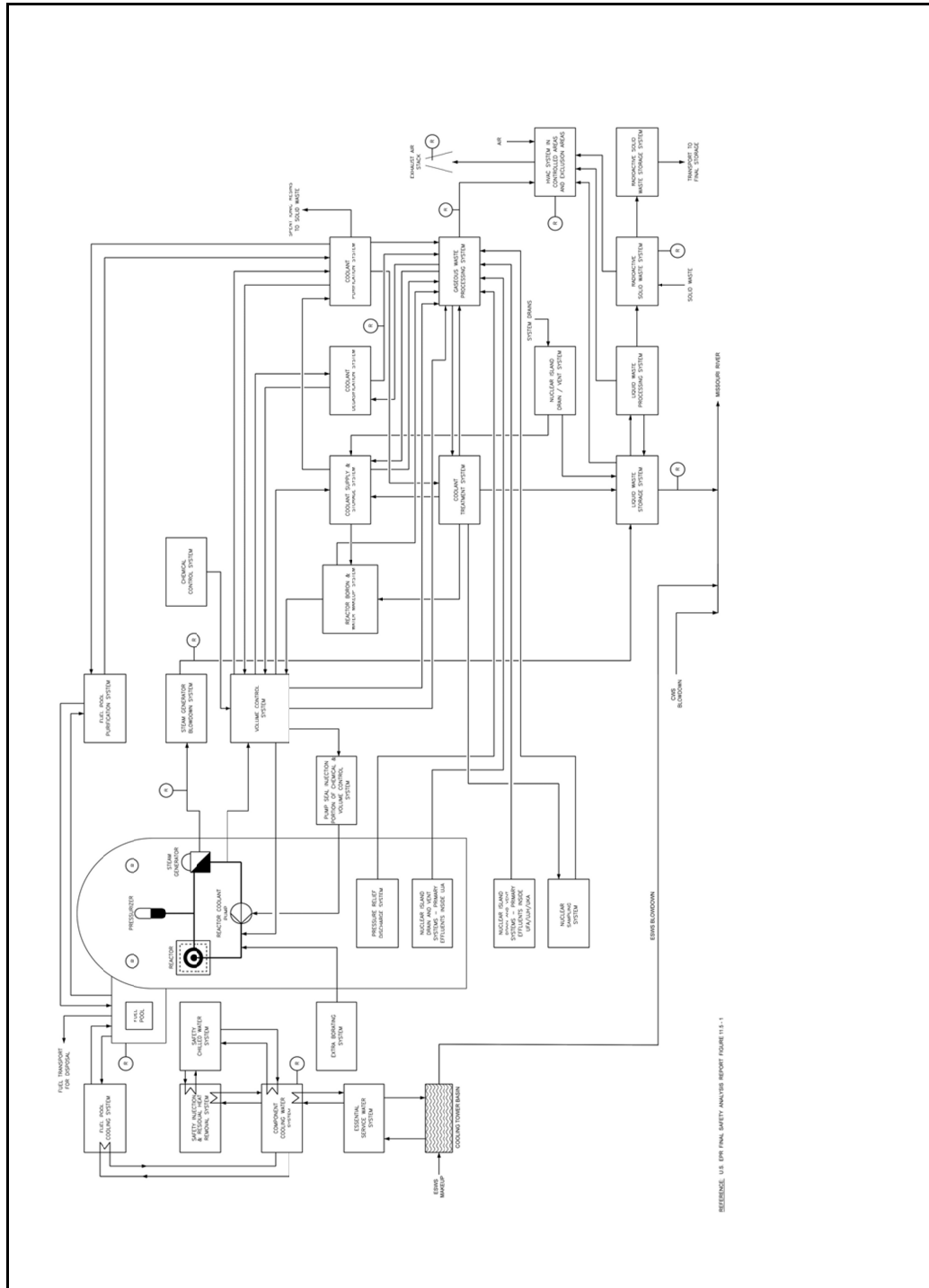
Method	Monitoring Task	Radioisotopes	Range
Gamma-sensitive detector (threshold 350 keV). Alternatively aerosol radioactivity is monitored continuously by a beta-sensitive detector (β,γ)	Aerosol in the exhaust air of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building – (two cells).	–	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq/cc
Gamma-sensitive detector (γ)	Gaseous iodine in the exhaust air of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building – (two cells).	I-131	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 5E-8 μ Ci/cc 1E-5 – 2E-3 Bq/cc
Gamma-sensitive detector (γ)	Air leaving the fuel handling area adjacent to the monitored air duct. This redundant instrument provides input to a control function.	–	1E-5 – 1E+0 rad/hr 1E-7 – 1E-2 Gy/hr Must be capable of detecting 10 DAC-hours
Gamma-sensitive detector (γ)	Air leaving the fuel handling area adjacent to the monitored air duct. This redundant instrument provides input to a control function.	–	1E-5 – 1E+0 rad/hr 1E-7 – 1E-2 Gy/hr Must be capable of detecting 10 DAC-hours
Gamma-sensitive detector (threshold 350 keV). Alternatively aerosol radioactivity is monitored continuously by a beta-sensitive detector (β,γ)	Aerosols in the laboratory room exhaust air before the filters.	–	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq/cc Must be capable of detecting 10 DAC-hours
Gamma-sensitive detector (threshold 350 keV). Alternatively aerosol radioactivity is monitored continuously by a beta-sensitive detector (β,γ)	Aerosol radioactivity in the exhaust air of the hot workshop before the filters.	–	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq/cc Must be capable of detecting 10 DAC-hours
Gamma-sensitive detector (threshold 350 keV). Alternatively aerosol radioactivity is monitored continuously by a beta-sensitive detector (β,γ)	Aerosol in the exhaust air of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building – (two cells).	–	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq/cc
Gamma-sensitive detector (γ)	Gaseous iodine in the exhaust air of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building – (two cells).	I-131	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 5E-8 μ Ci/cc 1E-5 – 2E-3 Bq/cc

Table 3.5-20—Radiation Monitors

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Method	Monitoring Task	Radioisotopes	Range
Gamma-sensitive detector (threshold 350 keV). Alternatively aerosol radioactivity is monitored continuously by a beta-sensitive detector (β,γ)	Aerosol in the exhaust air of the Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building – (two cells).	–	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq/cc
Gamma-sensitive detector (γ)	Gaseous iodine in the exhaust air of Nuclear Auxiliary Building (cell 1, cell 2 and cell 3), Fuel Building (cell 4 and cell 5), from the Safeguard Building - (cell 6) and from the Radioactive Waste Building – (two cells).	I-131	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 5E-8 μ Ci/cc 1E-5 – 2E-3 Bq/cc
Gamma-sensitive detector (threshold 350 keV). Alternatively aerosol radioactivity is monitored continuously by a beta-sensitive detector. (β,γ)	Aerosol in the exhaust air of the decontamination room in the Radwaste Building.	–	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq/cc Must be capable of detecting 10 DAC-hours
Gamma-sensitive detector (threshold 350 keV). Alternatively aerosol radioactivity is monitored continuously by a beta-sensitive detector. (β,γ)	Aerosol radioactivity in the exhaust air of the mechanical workshop in the Radwaste Building.	–	5E-4 – 3E+0 μ Ci 2E+1 – 1E+5 Bq, 3E-10 – 1E-6 μ Ci/cc 1E-5 – 4E-2 Bq/cc Must be capable of detecting 10 DAC-hours
Gamma-sensitive detector (γ)	Intake air of the main control room MCR inside each of the two MCR intake air ventilation ducts.	–	1E-5 – 1E+1 rad/hr 1E-7 – 1E+1 Gy/hr Must be capable of detecting 10 DAC-hours
Gamma-sensitive detector (γ)	Intake air of the main control room MCR inside each of the two MCR intake air ventilation ducts.	–	1E-5 – 1E+1 rad/hr 1E-7 – 1E+1 Gy/hr Must be capable of detecting 10 DAC-hours

Figure 3.5-1—Radwaste Effluent Flow Paths



SEE REFERENCE: US - EPR FINAL SAFETY ANALYSIS REPORT FIGURE 3.5-1

Figure 3.5-2—Liquid Radwaste Storage and Processing

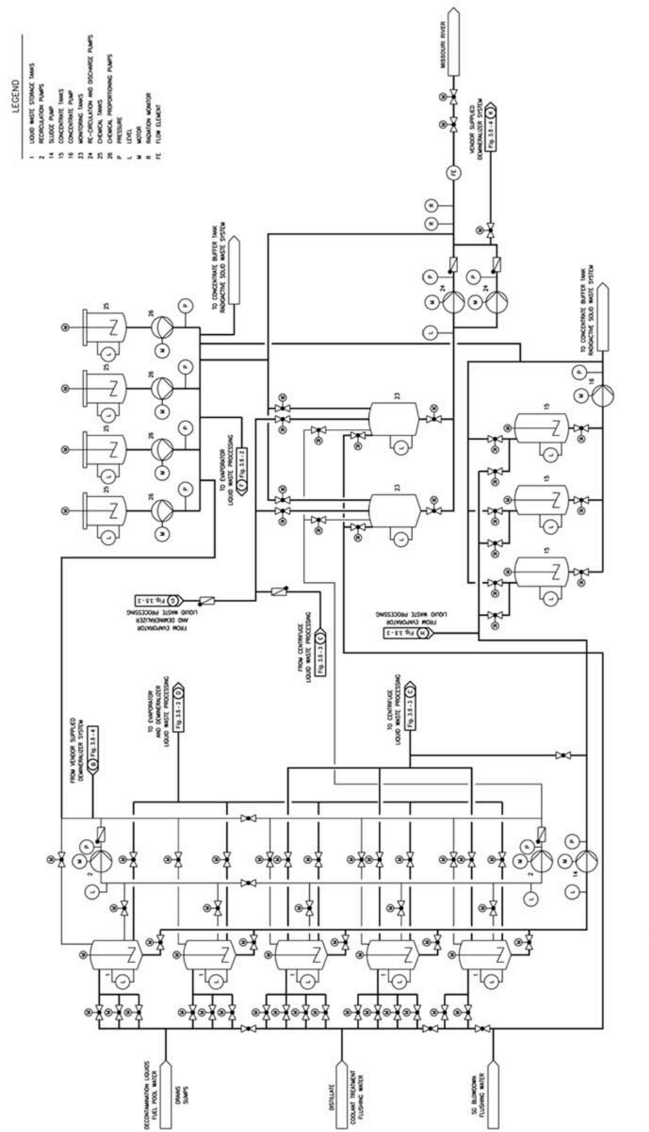


Figure 3.5-3—Liquid Waste Treatment Evaporator and Centrifuge

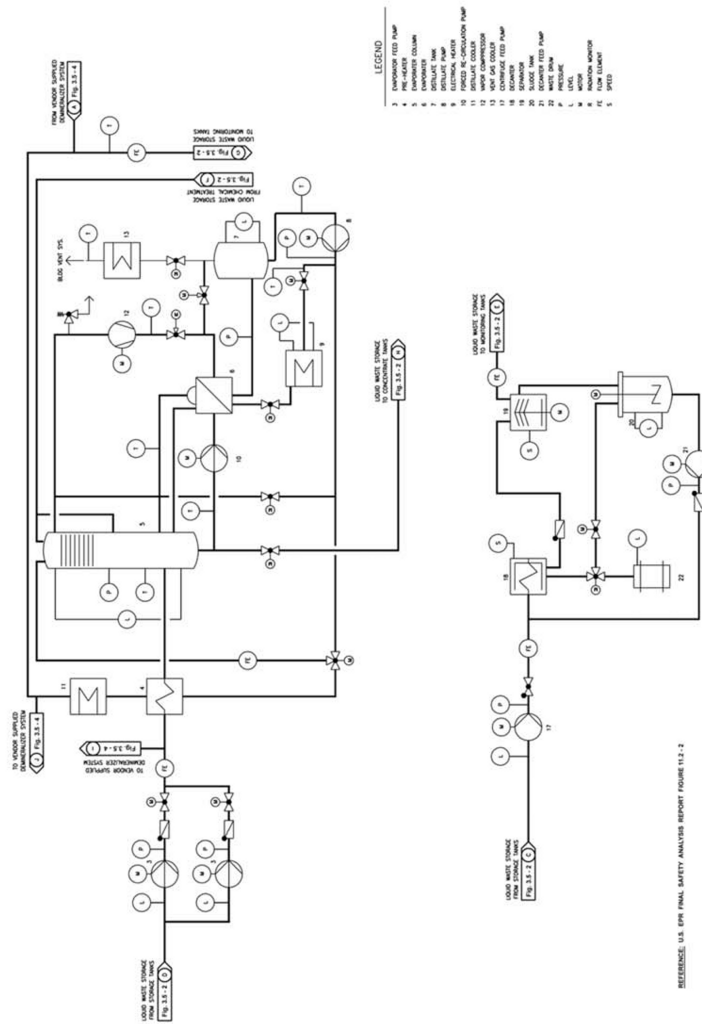


Figure 3.5-4—Liquid Waste Treatment Vendor Supplied Demineralizer System

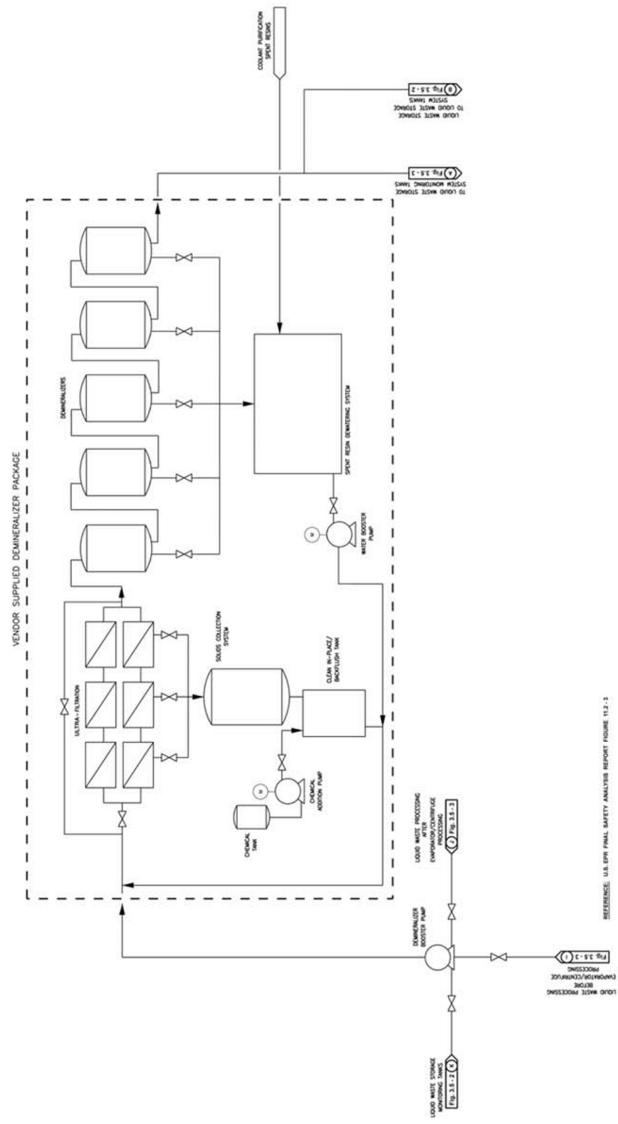


Figure 3.5-5—Gaseous Waste Processing and Sources

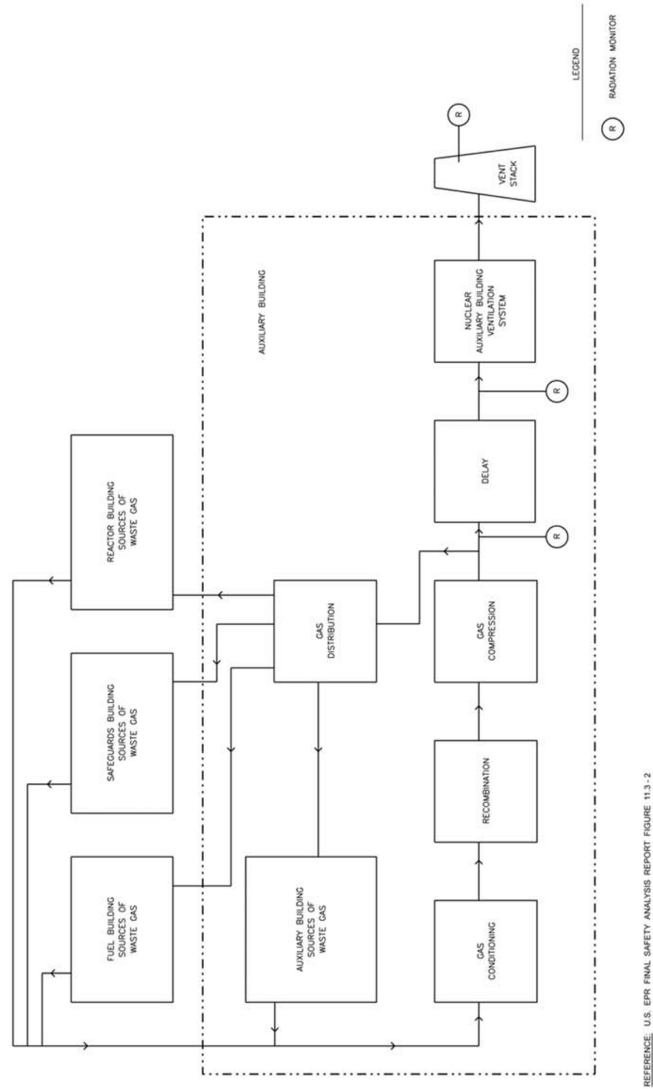
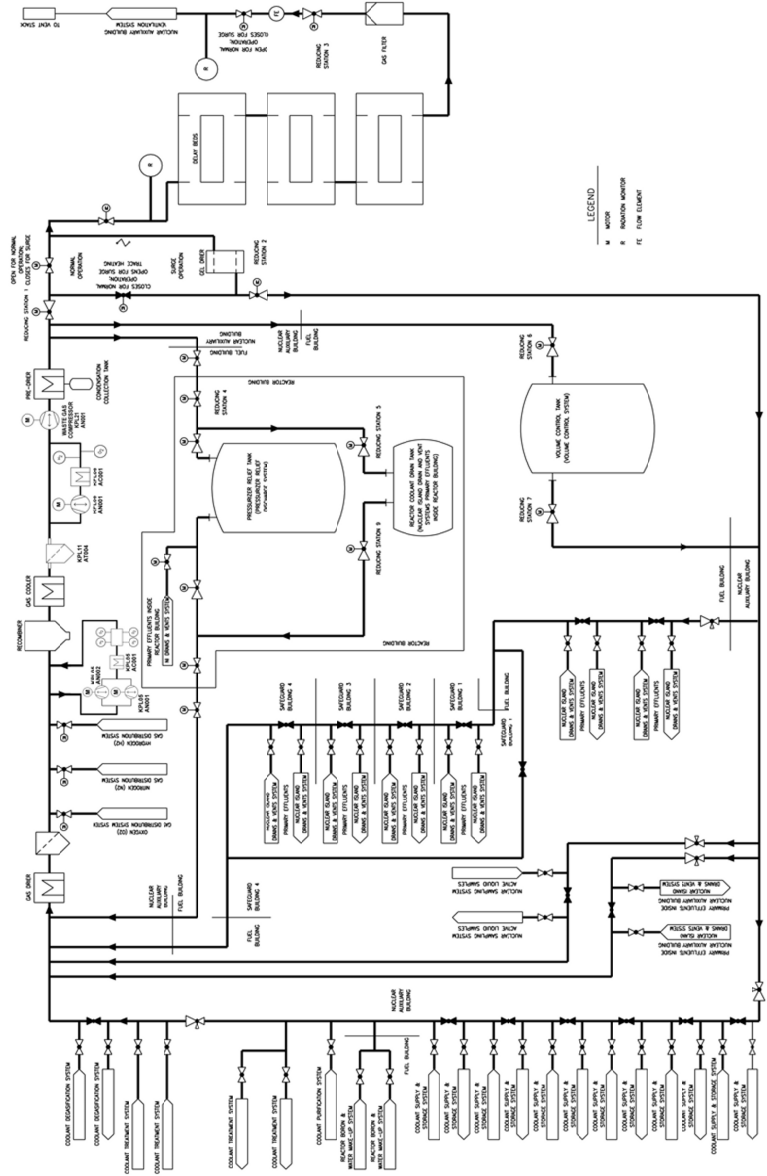
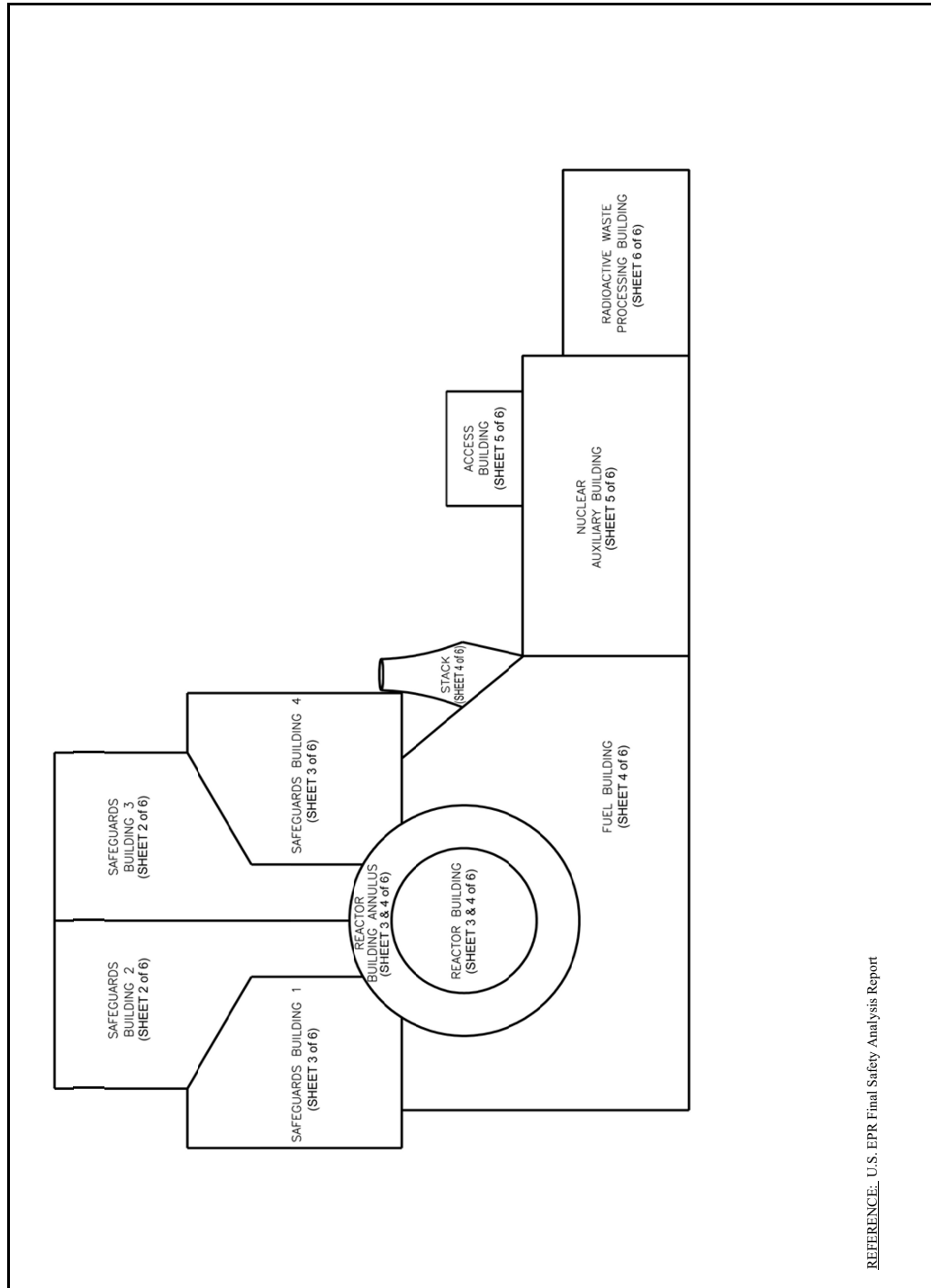


Figure 3.5-6—Gas Waste Treatment



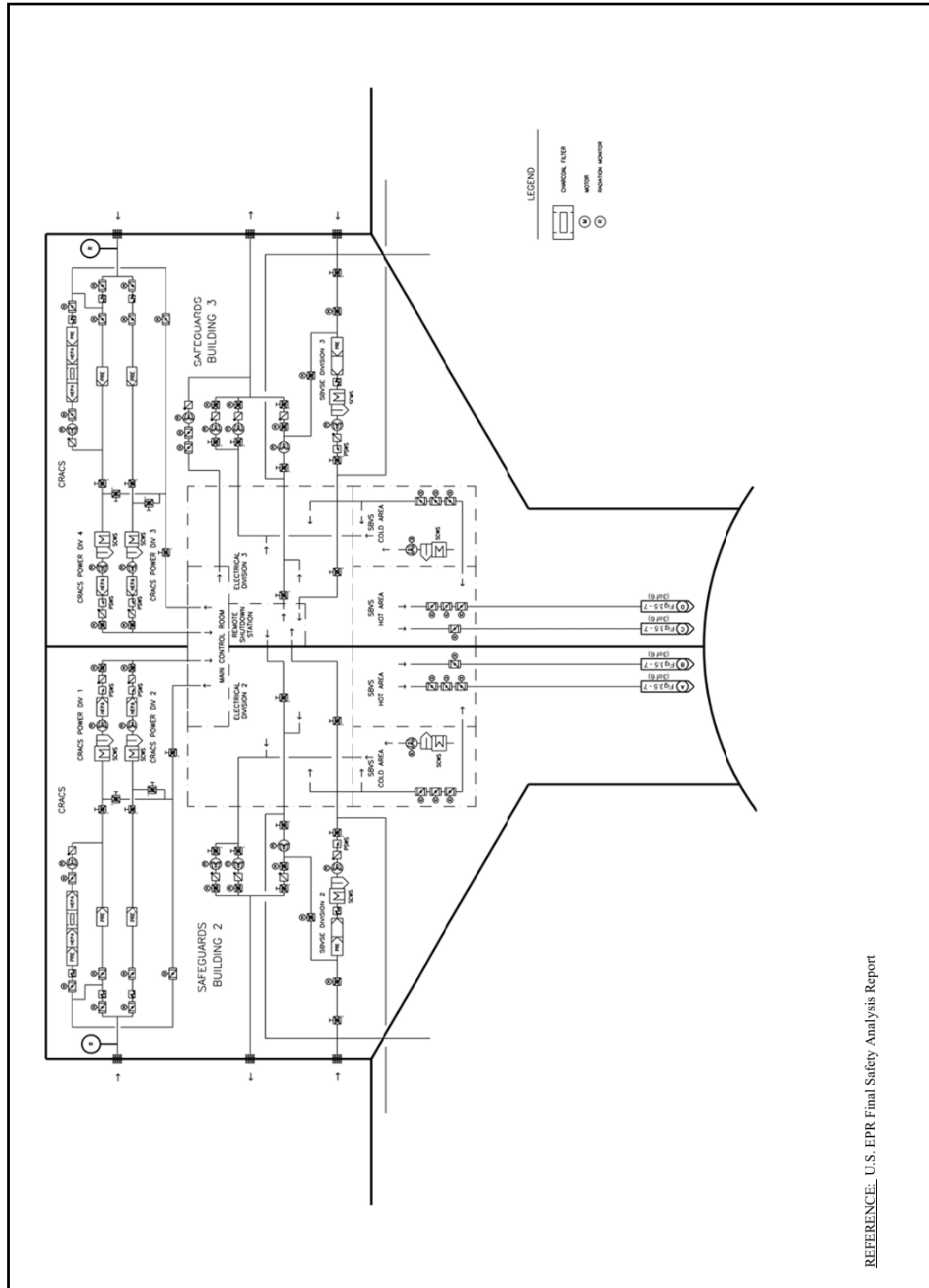
REFERENCE: U.S. EPR Final Safety Analysis

Figure 3.5-7—Controlled Area Ventilation Flow Diagram (Page 1 of 6)



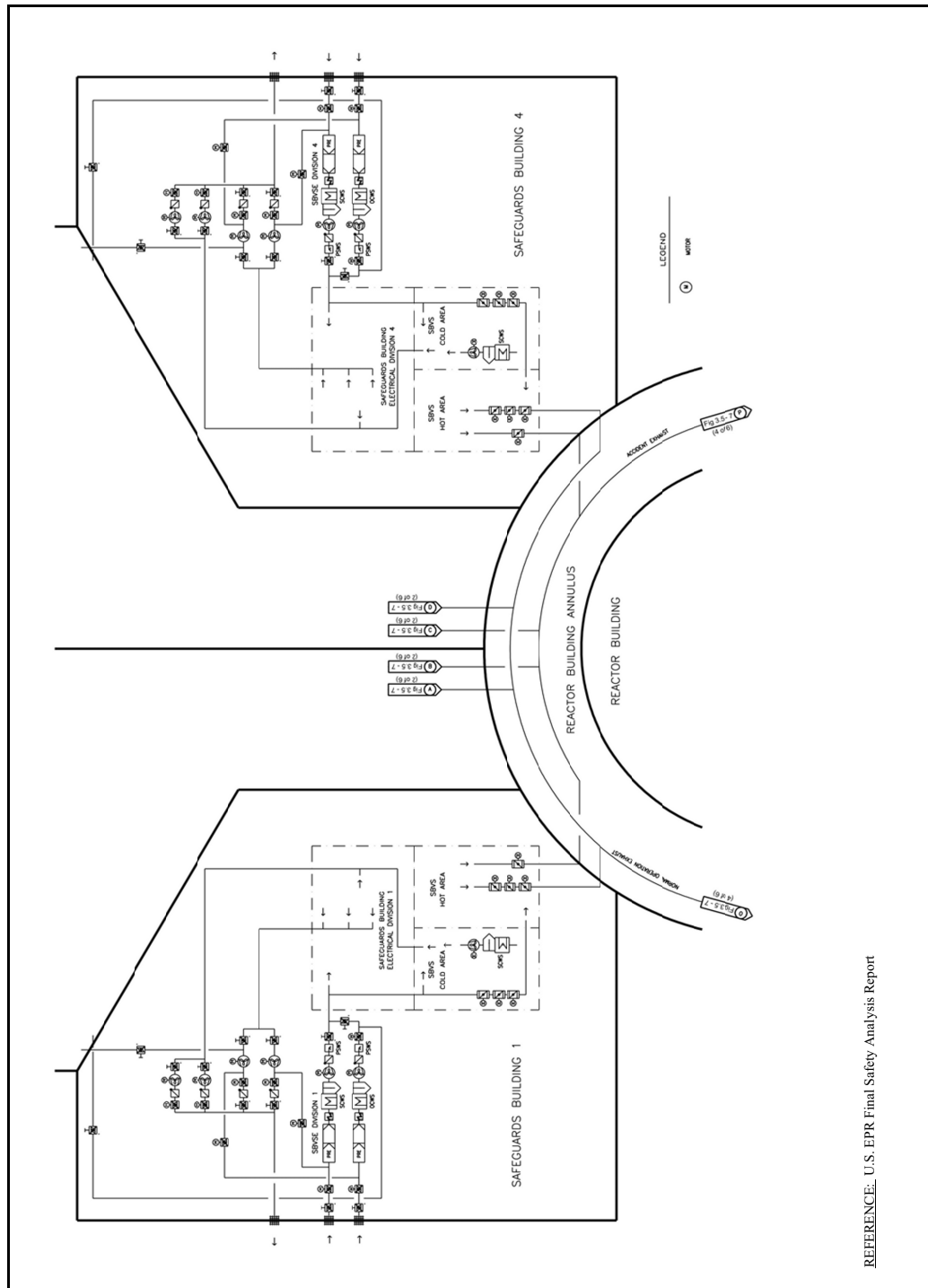
REFERENCE: U.S. EPR Final Safety Analysis Report

Figure 3.5-8—Controlled Area Ventilation Flow Diagram (Page 2 of 6)



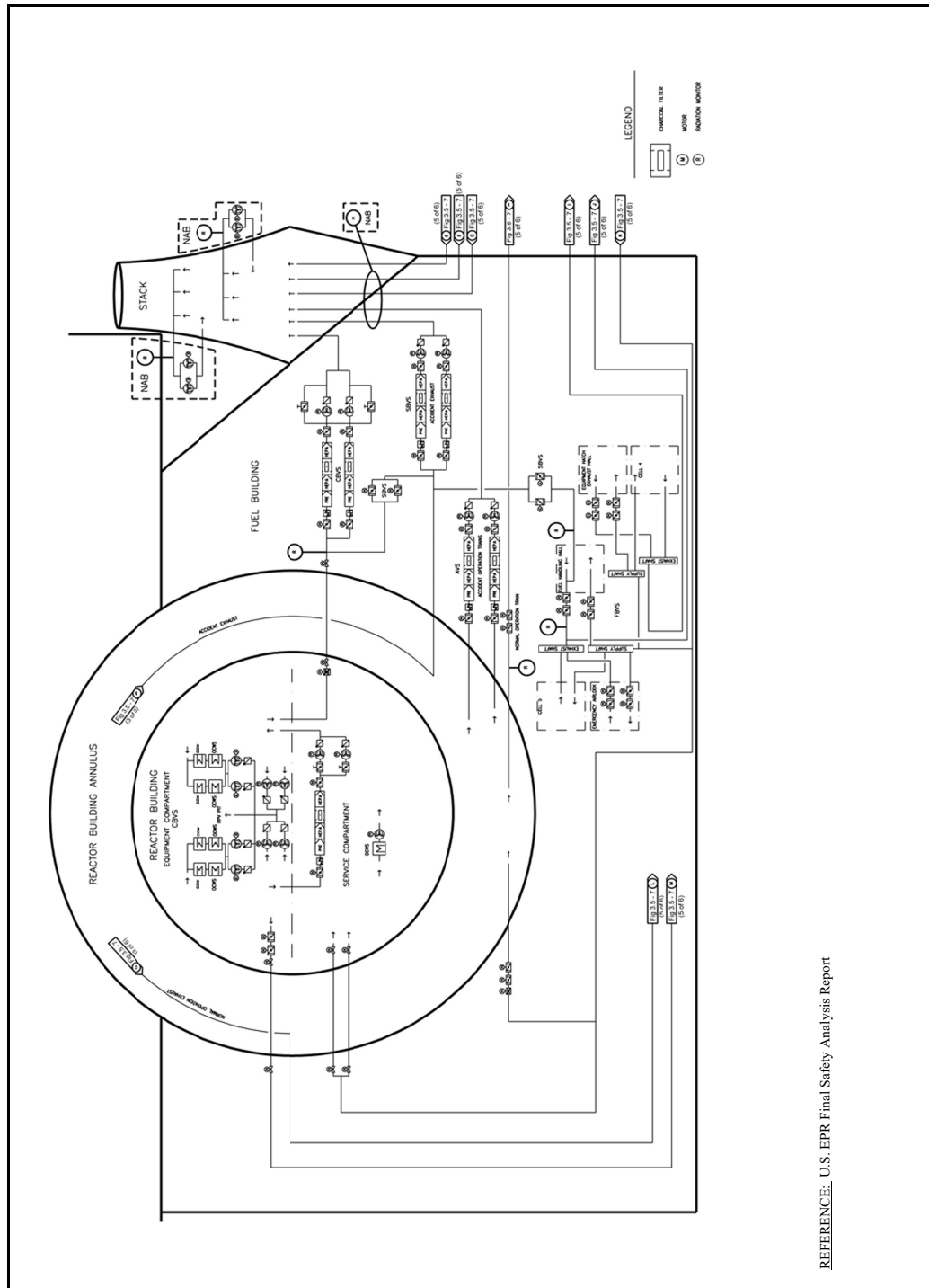
REFERENCE: U.S. EPR Final Safety Analysis Report

Figure 3.5-9—Controlled Area Ventilation Flow Diagram (Page 3 of 6)



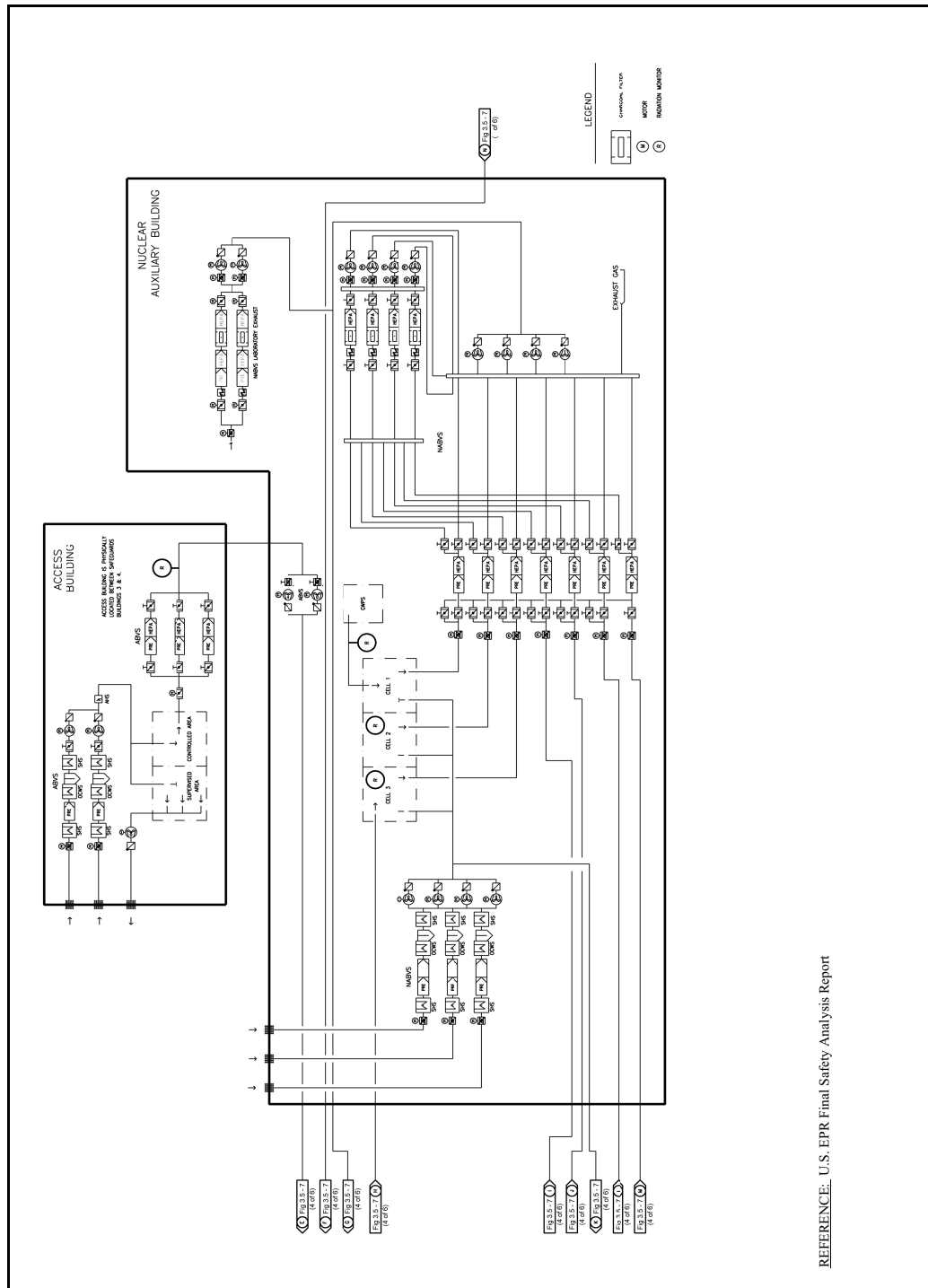
REFERENCE: U.S. EPR Final Safety Analysis Report

Figure 3.5-10—Controlled Area Ventilation Flow Diagram (Page 4 of 6)



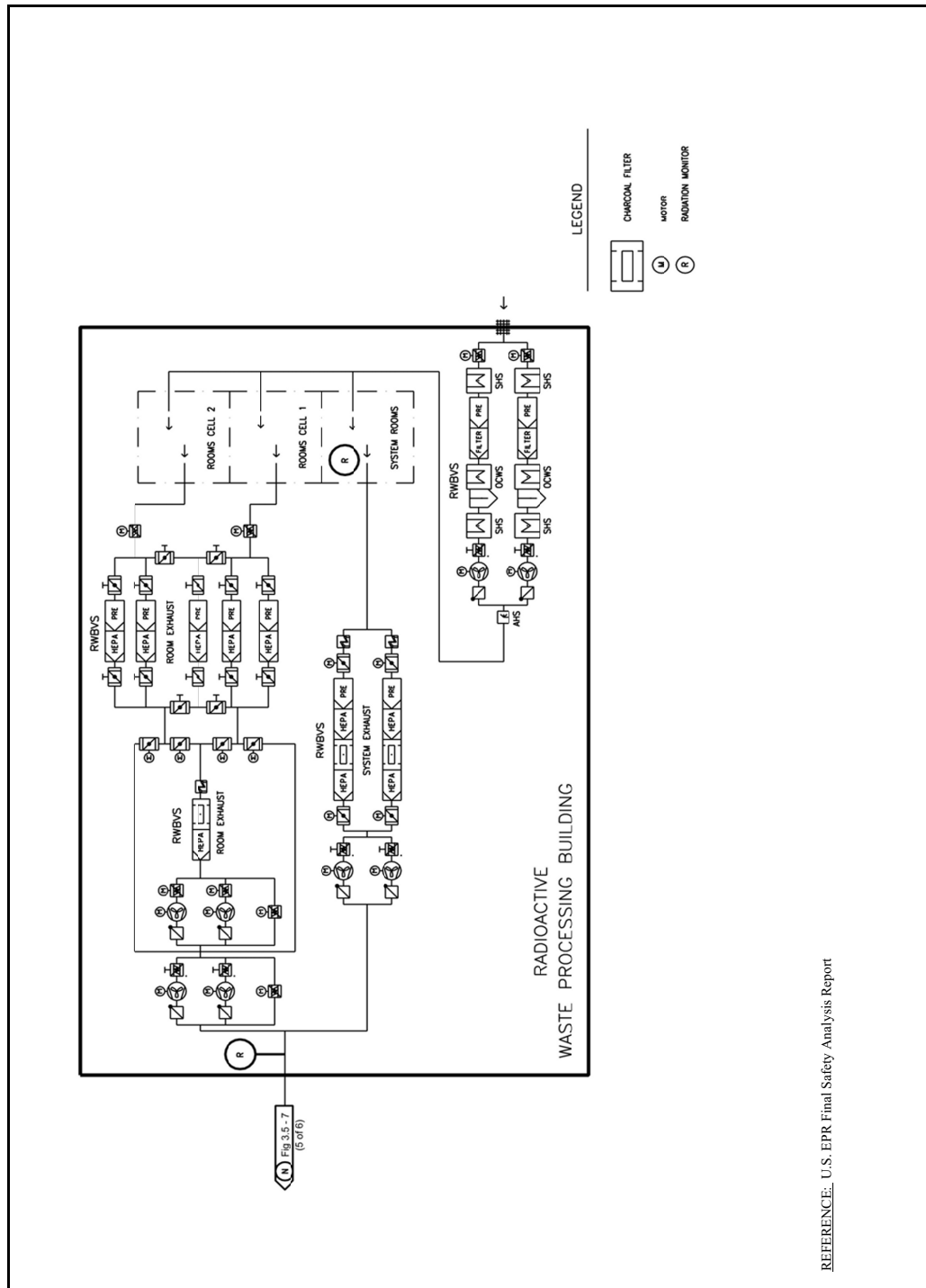
REFERENCE: U.S. EPR Final Safety Analysis Report

Figure 3.5-11—Controlled Area Ventilation Flow Diagram (Page 5 of 6)



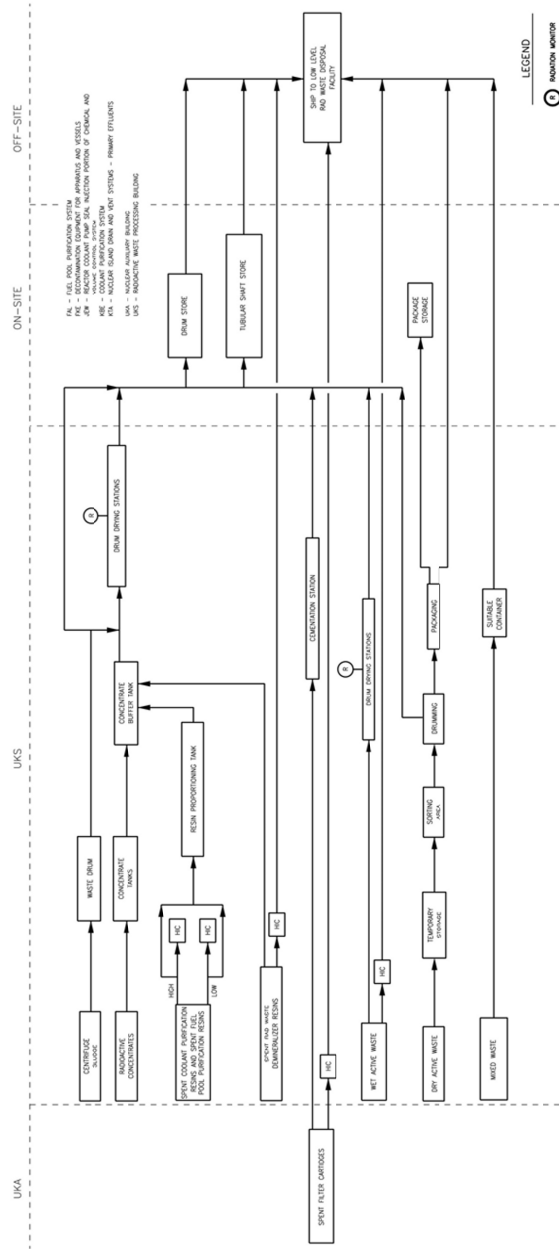
REFERENCE: U.S. EPR Final Safety Analysis Report

Figure 3.5-12—Controlled Area Ventilation Flow Diagram (Page 6 of 6)



REFERENCE: U.S. EPR Final Safety Analysis Report

Figure 3.5-13—Solid Waste System Flow Diagram



REFERENCE: U.S. EPR FINAL SAFETY ANALYSIS REPORT FIGURE 11.4.1

3.6 NON-RADIOACTIVE WASTE SYSTEMS

This section provides a description of non-radioactive waste systems for {Callaway Plant Unit 2} and the chemical and biocidal characteristics of each non-radioactive waste stream discharged from the unit. The non-radioactive waste streams include: (1) effluents containing chemicals or biocides; (2) sanitary system effluents; and (3) other effluents.

3.6.1 EFFLUENTS CONTAINING CHEMICALS OR BIOCIDES

Chemicals are typically used to control water quality, scale, corrosion and biological fouling. Sources of non-radioactive effluents include plant blowdown, sanitary wastes, floor and equipment drains, and storm water runoff.

As described in Section 3.3.2, the treatment of non-radioactive effluents is performed at the {Water Treatment Plant System,} Circulating Water Treatment System, the Essential Service Water Treatment System, the Liquid Waste Processing System and the Waste Water Treatment System. [Table 3.6-1](#) lists the various treatment chemical additions at these systems. Chemical concentrations within effluent streams from the plant are controlled through engineering and operational/administrative controls in order to meet {NPDES} requirements at the time of construction and operation.

{Naturally occurring processes (e.g. biological growth) are not changed in form or concentration by plant operations. These naturally occurring processes may continue, but are reduced (compared to Callaway Plant Unit 1) due to the use of Missouri River/ Missouri River Alluvial Aquifer (Aquifer) collector well system instead of direct water pumping from the river. Biological growth is controlled where necessary using biocide treatment. Collector well system installed in the Aquifer supplies cooling water makeup for Callaway Plant Unit 2. The use of a collector well system instead of direct pumping from the river (as is currently being done to supply Callaway Plant Unit 1), limits the need for advanced water treatment to remove turbidity and accumulation of excess solids requiring subsequent management and disposal. Aquifer extraction reduces influent turbidity compared to direct water pumping from the river. However, water quality is largely the same as in the Missouri River. Missouri River water quality is discussed in Section 2.3.3.1.2. Treatment to remove dissolved iron is necessary. This is noted in [Table 3.6-1](#).}

Evaporative cooling systems include the {natural draft Circulating Water System (CWS) cooling towers and the Essential Service Water System (ESWS) cooling towers.} Some of the cooling water associated with these systems is lost through evaporation via their cooling towers as discussed in Section 3.3. During warm weather, when the difference between the air temperature and the water temperature is relatively small, cooling of the water is almost entirely the result of the extraction of heat through evaporation of water to the air. Under extreme winter conditions (e.g., below zero), when the air is much colder than the water, as much as half of the cooling may be the result of sensible heat transfer from the water to the air with the remainder of the cooling being through evaporation. {The CWS and ESWS cooling towers are based on 4.8 and 2 cycles of concentration respectively.} No seasonal variations in cycles of concentration are expected.

Section 3.6.3.2 describes the effluent water chemical concentrations from other sources and the water treatment for general plant use and effluents from the resultant waste stream.

3.6.2 SANITARY SYSTEM EFFLUENTS

The purpose of this section is to identify the anticipated volume and type of sanitary waste effluents generated during construction and operation of {Callaway Plant Unit 2}. Sanitary

waste systems installed during pre-construction and construction activities include portable toilets supplied and serviced by a licensed sanitary waste treatment contractor. {During construction, the estimated quantity of sanitary waste expected to be generated is 6,500 gpd (24,600 lpd) for the first year, and 19,500 gpd (73,800 lpd) for years 2 through 5. Sanitary waste generated by portable toilet systems is removed offsite by the vendor during pre-construction and construction activities and do not add to the existing load to on-site discharge effluents.

Based on an anticipated construction work force of 1,000 people in the first year of construction activities and 3,000 people in the second through fifth year of constructions activities, peak construction potable water usage is estimated at 26 gpm (98.6 lpm) for Callaway Plant Unit 2. This estimate is calculated by assuming portable toilets will be used to accommodate the needs for most craft construction personnel as described above. These units will not require potable water or have an input into the sanitary system as they will be serviced by a licensed sanitary waste management contractor. The main input to the sanitary system will be from office personnel. The existing sewage lagoons have sufficient capacity to manage the additional sanitary waste load during both construction and operation. The functional operation of the existing sewage treatment lagoons are discussed below.}

During the Operations phase for {Callaway Plant Unit 2, the existing onsite sanitary wastewater treatment lagoons manage sanitary wastes. The newly installed Callaway Plant Unit 2 wastewater collection system collects sanitary sewage and wastewater from various site facilities (e.g. restrooms, shower, cafeteria areas, etc.) outside of radiological controlled areas, and transfers it for treatment to the existing series of sewage treatment lagoons. These lagoons are designed to provide unaided natural biological activity for treating wastewater in compliance with state standards. The sewage from Callaway Plant Unit 2 is combined with the sewage and miscellaneous wastewater (i.e. cafeteria) from Callaway Plant Unit 1, and is delivered to the sewage treatment lagoons via a common lift station. The multi-cell sewage lagoon arrangement utilizes a fermentation pit that promotes anaerobic biological action to allow the natural decay of solids for extended periods of time. The system accommodates a design flow rate of 40,000 gpd (151,000 lpd). The treatment system is designed for domestic waste only. Segregation, administration, and engineering control measures in place at Callaway Plant Unit 1 is similarly implemented throughout Callaway Plant Unit 2 to exclude industrial materials such as chemical laboratory wastes and other potentially hazardous materials that could impact the performance of the treatment system.

Effluent from the sewage treatment lagoons is pumped to wetlands adjacent to the sewage lagoons via a separate lift station. Effluent from the wetlands eventually makes its way through the system to a supernatant pump station that returns the combined effluent back to the cooling tower. Water quality discharged from the sludge lagoon wetland system is monitored in accordance with the current NPDES permit requirements as described below. The Waste Water Treatment Plant System is monitored and controlled by trained operators. Sanitary wastewater treatment does not affect storm water runoff.

Table 1.3-1 lists the environmental-related permits and authorizations for Callaway Plant Unit 2. Effluent discharges are regulated under the provisions of the Federal Water Pollution Control Act of 1972 (Clean Water Act) as amended (USC, 2007). Under the Act, the MDNR Water Pollution Control Branch has been delegated the authority from the U.S. EPA to issue NPDES permits. The discharge from the sanitary waste treatment lagoons to the constructed wetlands (sludge lagoon wetlands) is regulated under State of Missouri issued NPDES permit no. MO-0098001 for Outfall 2007. It includes provisions to monitor flow rates, pollutant concentrations, and the biochemical oxygen demand at the point of release. Specific chemical concentration limits are shown in **Table 6.3-1**.}

3.6.3 OTHER EFFLUENTS

This section describes miscellaneous non-radioactive gaseous, liquid, or solid effluents not addressed in Sections 3.6.1 or 3.6.2.

3.6.3.1 Gaseous Effluents

Non-radioactive gaseous effluents result from testing and operating the diesel generators. These effluents commonly include particulates, sulfur oxides, carbon monoxide, hydrocarbons and nitrogen oxides. Gaseous effluent releases comply with Federal, State, and local emissions standards. [Table 1.3-1](#) lists the environmental-related permits and authorizations for {Callaway Plant Unit 2}.

{Callaway Plant Unit 2} has six standby diesel generators (four Emergency Diesel Generators (EDGs), and two Station Blackout (SBO) diesel generators). The auxiliary boilers use electric heating, and do not contribute directly to air emissions.

Each EDG is tested approximately 4 hours every month, plus an additional 24 to 48 hours once every 2 years. Each SBO diesel generator is tested approximately 4 hours every quarter, plus an additional 12 hours every year for maintenance activities. The SBO diesels are also tested for an extended period of about 12 hours every 18 months.

Diesel generator emissions are released from an exhaust stack located on top of the diesel generator buildings at an elevation of {approximately 70 ft (21 m) above grade or 915 ft (279 m) msl}. Pre-treatment of diesel generator exhaust depends on future diesel technology that has yet to be determined. Diesel generator exhaust meets Environmental Protection Agency (EPA) Tier 4 requirements when {Callaway Plant Unit 2} is operational. Yearly emissions anticipated from the standby diesel generators are provided in [Table 3.6-2](#), and assume a conservative run time for each diesel of 100 hours per year.

{The Fire Water Distribution System (FWDS) includes one or more diesel-engine driven fire water pumps. These pumps are located adjacent to the Fire Water Storage Tanks. The diesel-engine driven pumps are tested on a quarterly basis for a minimum period of one hour per diesel engine.}

3.6.3.2 Liquid Effluents

{Missouri River/Missouri River Alluvial Aquifer (Aquifer)} water serves as the source of cooling water {makeup} for the {Circulating Water Supply System, Essential Service Water Supply System, Essential Service Water Emergency Makeup System, and Emergency Supply for the Fire Protection System}.

Deep wells provide water for the Demineralized Water Distribution System, the Potable and Sanitary Water Distribution System and the Fire Protection System as described in Section 3.3.

CWS takes Aquifer water into a closed cooling system, which utilizes two natural draft cooling towers to cool the water after it has cooled the plant's condensate. ESWS also takes Aquifer water into a closed cooling system, which utilizes four mechanical draft cooling towers (two of which are in use during normal plant operation) to remove heat from the Component Cooling Water System (CCWS) heat exchangers located in the Safeguards Building and the cooling jackets of the emergency diesel generators located in the Emergency Power Generating Buildings. A portion of the CWS and ESWS water is constantly returned as blowdown to the Missouri River via a discharge pipe. Callaway Plant Unit 1 has a number of NPDES permitted non-radioactive liquid effluents outfalls. Some drain to the Missouri River. Others, such as the

discharge from the wetlands and storm water sedimentation ponds, discharge to other areas or receiving streams. Regulatory limits for chemical concentrations in waste streams are given in [Table 6.3-1](#). It is anticipated that Callaway Plant Unit 2 will take advantage of existing discharge permitted outfalls to the extent possible. The existing permit will need to be reissued prior to the start-up of Callaway Plant Unit 2 to include any new permitted outfalls. It is anticipated that concentration limits in the new NPDES permit will be similar to those for Callaway Plant Unit 1. The facility will continue to monitor all discharges in accordance with State of Missouri issued NPDES permit for the facility.}

Anticipated effluent water chemical concentrations from {Callaway Plant Unit 2} are included in [Table 3.6-3](#).

{Other non-radioactive liquid waste effluents generated in the controlled area (i.e., Steam Generator Blowdown Demineralizing System), are managed and processed by the Liquid Waste Storage System and the Liquid Waste Processing System. Non-radioactive liquid waste first collects in a tank where it is pre-treated chemically or biologically. Chemical pre-treatment gives the waste an optimum pH value; biological pre-treatment allows organics to be consumed. If deemed cleaned, it is routed directly to one of the monitoring tanks; otherwise, once pre-treated, the wastes are forwarded to the Liquid Waste Processing System for treatment. Treatment may consist of evaporation, centrifugation, demineralization/filtration, chemical precipitation (in connection with centrifugation), or organic decomposition (in connection with centrifugation). After the waste water has been treated, it is received in one of two monitoring tanks, which also receive treated liquid radwaste. Wastewater is then sampled and analyzed, and if within the limits for discharge, is released.}

3.6.3.3 Hazardous Wastes

Hazardous wastes are materials with properties that make them dangerous or potentially harmful to human health or the environment, or that exhibit at least one of the following characteristics: ignitability, corrosivity, reactivity or toxicity. Federal Resource Conservation and Recovery Act (RCRA) regulations govern the generation, treatment, storage and disposal of hazardous wastes. Hazardous waste is defined as any solid, liquid or gaseous waste that is not mixed waste, is listed as hazardous by any federal or state regulatory agency or meets the criteria of Subpart D of 40 CFR 261 (CFR, 2007b) {or Code of Missouri State Regulations (CSR) Title 10 Division 25 Chapter 4 Part 261 (CSR, 2006a)}.

A Hazardous Waste Minimization Plan is developed and maintained that documents the current and planned efforts to reduce the amount or toxicity of the hazardous waste to be generated at {Callaway Plant Unit 2}. Hazardous wastes are collected and stored in a controlled access temporary storage area (TSA). A Hazardous Material and Oil Spill Response guideline is maintained that defines HAZMAT team positions and duties. Procedures are in place to minimize the impact of any hazardous waste spills in the unlikely event of a spill. Containers of known hazardous waste received at a TSA are transported offsite within 90 days of the containers accumulation date according to the applicable section/unit procedures. The {Operations Manager} is responsible for coordinating the activities of waste transport disposal vendors or contractors while they are on site, ensuring that the transporter {has an EPA identification number.}

{[Table 3.6-4](#) lists the types and quantities of hazardous waste generated at Callaway Plant Unit 1. The table is based on the Callaway biennial hazardous waste reports submitted for 2001, 2003, and 2005. The quantity of hazardous wastes generated at Callaway Plant Unit 2 is expected to be similar to or less than that at Callaway Plant Unit 1.}

3.6.3.4 Mixed Wastes

Mixed waste includes hazardous waste that is intermixed with a low level radioactive source, special nuclear material, or byproduct material. Federal regulations governing generation, management, handling, storage, treatment, disposal, and protection requirements associated with these wastes are contained in 10 CFR (NRC regulations) and 40 CFR (Environmental Protection Agency regulations). Mixed waste is generated during routine maintenance activities, refueling outages, radiation and health protection activities and radiochemical laboratory practices. Section 5.5.2 discusses mixed waste impacts, including quantities of mixed waste generated. The quantity of mixed waste generated at {Callaway Plant Unit 2} is expected to be small, as it is at other nuclear power plants.

{The management of mixed waste for Callaway Plant Unit 2 complies with the requirements of U.S. EPA's Mixed Waste Enforcement Policy until an approved, EPA permitted disposal facility becomes available. Callaway Plant Unit 1 currently ships some mixed waste offsite to permitted facilities. This occurs infrequently, and is dependent on the waste matrix. It is expected that Callaway Plant Unit 2 also infrequently ships some mixed waste to permitted facilities.

Mixed wastes stored in a Temporary Storage Area (TSA) are inventoried and a list is maintained according to Callaway Plant Unit 2 procedures, and periodic inspections of mixed waste are conducted according to these same procedures.}

3.6.3.5 Solid Effluents

{During construction, non-hazardous solid municipal and construction waste materials may be generated. Depending on the nature of the waste material, the operation of a construction (non-hazardous) solid-waste facility for private use at the Callaway Plant Unit 2 site requires a permit and must comply with the regulations imposed by the State of Missouri for construction, installation and operation of solid waste facilities. Some waste materials rather than being disposed may be stockpiled or conditioned for reuse or natural stabilization (e.g. mulching). Chemically clean natural construction wastes may be usable or conditioned for landscaping, site restoration, cover, or fill which include: land clearing debris; earthen material; including as clays, sands, gravel, silts, or topsoil, tree stumps, root mats, brush, limbs, logs, vegetation, and rock. In general, the State of Missouri requires construction and demolition debris to be recycled or disposed at an approved landfill or transfer station. In the State of Missouri, open burning of tree trunks, tree limbs and vegetation from land clearing operations is allowed only in the outstate area providing the burning takes place outside the city limits of any incorporated area or municipality and at least 200 yards from the nearest inhabited dwelling. Materials such as tires or waste oil may not be used to start the fires or be burned in the fires.

Other waste materials such as office paper and aluminum cans is recycled locally. Putrescible wastes are disposed in a permitted offsite disposal facility.

The types of solid effluents that are generated by Callaway Plant Unit 2 include hazardous waste, mixed wastes, and solid or spent materials from water management or treatment. Hazardous waste generation is discussed in Section 3.6.3.3, and mixed waste generation is discussed in Section 3.6.3.4.

Based on the operating experience at Callaway Plant Unit 1, it is expected that Callaway Plant Unit 2 will have essentially zero solid waste effluent. This is because Callaway Plant Unit 1 recycles, recovers, or sends offsite for disposal virtually all of its solid waste, and does not

release solid waste as an effluent. Disposal, recycling, and recovery of solid wastes (e.g., scrap metal, petroleum product waste, etc) are described in Section 5.5.1. In summary:

- ◆ Non-radioactive solid wastes (e.g., office wastes, recyclables) are collected temporarily on the Callaway Plant Unit 2 site and disposed of at offsite, licensed disposal and recycling facilities.
- ◆ Scrap metal, used oil, antifreeze (ethylene or propylene glycol), and universal waste will be collected and stored temporarily on the Callaway Plant Unit 2 site and recycled or recovered at an offsite permitted recycling or recovery facility, as appropriate. Used oil that is mixed with hazardous waste that solely exhibits the characteristic of ignitability, or is mixed with a listed hazardous waste that is listed solely because it exhibits the characteristic of ignitability, shall be managed as a used oil. (CSR, 2006b).
- ◆ Spent antifreeze can be managed as non-hazardous waste provided it is recycled and any waste or subsequent mixture does not exhibit the characteristic of ignitability. Antifreeze is not a controlled hazardous substance unless it has been combined or mixed with characteristic or listed hazardous wastes. Typically, used oil and antifreeze are recycled. If they are not, they will be disposed of as solid waste in accordance with the applicable regulations.
- ◆ U.S. EPA presumes that used oil is to be recycled unless a used oil handler disposes of used oil, or sends used oil for disposal. Except as provided in 40 CFR §279.11, waste oil not recycled or properly managed under the regulations, may be considered a hazardous waste. (CFR, 2007b).
- ◆ Waste antifreeze is not a listed hazardous waste under the federal hazardous waste regulations in 40 CFR §261 Subpart D. Spent antifreeze may contain metals, particularly lead, and other substances that would cause it to be classified as a characteristic hazardous waste. It is the generators responsibility to determine the chemical characteristics of the waste material or mixture and manage appropriately (40 CFR 261Subpart C) (MDNR, 1999).}

3.6.4 REFERENCES

{**CFR, 2007a.** Title 40, Code of Federal Regulations, Part 261, Identification and Listing of Hazardous Waste, 2007.

CFR, 2007b, Title 40 CFR, Part 279 Standards for Used Oil Generators, web page

CSR, 2006a. Title 10 - Missouri Department of Natural Resources Division 25 - Hazardous Waste Management Commission – Chapter 4, Part 261 - Methods for Identifying Hazardous Waste, Missouri Secretary of State Web Page

<http://www.sos.mo.gov/adrules/csr/current/10csr/10c25-4.pdf>, November 30, 2006, visited January 3, 2007.

CSR, 2006b. Title 10 - Missouri Department of Natural Resources Division 25 - Hazardous Waste Management Commission, Chapter 11, Part 279 - Recycled Used Oil Management Standards <http://www.sos.mo.gov/adrules/csr/current/10csr/10c25-11.pdf>, November 30, 2006, visited January 3, 2007.

MDNR, 1999, Missouri Department of Natural Resources Antifreeze Waste Management Guide, Fact Sheet, 7/1999, MDNR web page, <http://www.p2pays.org/ref/15/14915.pdf>, visited January 4, 2008

USC, 2007. Title 33, United States Code, Part 1251, Federal Water Pollution Control Act, 2007, CFR Title 40, Code of Federal Regulations, Part 261, Identification and Listing of Hazardous Waste, 2007.}

Table 3.6-1—{Treatment System Processing Chemicals}

System	Operating Cycle(s)	Chemical Processed	Estimated Total Amount Used per Year	Frequency of Use	Avg./Max. Concentration in Waste Stream (mg/l)
Water Treatment Plant System	Normal Operating Conditions and Normal Shutdown/Cooldown	Aeration	1.26 x 10 ⁶ ft ³ (3.57 x 10 ⁶ m ³)	Continuous	<0.1 average/ <1.0 maximum
		Oxidizer/Biocide ⁽¹⁾	278,511 gal. (1,054,162 l)	Continuous	Total residual chlorine: <0.1 average /<0.2 maximum
Circulating Water Treatment System	Normal Operating Conditions and Normal Shutdown/Cooldown	Sulfuric Acid	22,087,792 lb (10,019,023 kg)	Continuous	Total residual chemical: <0.1 average / <0.2 maximum
		Inhibitor	256,667 gal (971,485 l)	Continuous	35 average Water T/ 50 maximum
		Dispersant	436,334 lb (197,921 kg)	Continuous	5 average / 10 maximum
ESWS Cooling Tower Water Treatment System ^{(1) (2)}	Normal Operating Conditions and Normal Shutdown/Cooldown	Dispersant	9,695 gal (36,697 l)	Continuous	5 average / 10 maximum
		Sulfuric Acid	2,781,853 lb/yr (1,261,848 kg/yr)	Continuous	Total residual chemical: < 0.1 average / < 0.2 maximum
		Inhibitor	48,476 gal/yr (183,483 l/yr)	Continuous	35 average / 50 maximum
Liquid Waste Storage and Processing Systems ⁽³⁾	Normal Operating Conditions and Normal Shutdown/Cooldown	Sulfuric Acid	22,900 gal (86,686 l)	2 times/week	pH: 6.0 – 9.0
		Sodium Hydroxide	2,400 gal (9,085 l)	1/month	

Key:

gal – gallons
 L - liters
 kg – kilograms
 lbs – pounds
 CWS – Circulating Water System
 ESWS - Essential Service Water System
 TOC – Total Organic Carbon
 TRC – Total Residual Chlorine
 TSS – Total Suspended Solids

Notes:

- (1) An environmentally acceptable biocide such as hypochlorite may be used.
- (2) Referring to [Table 3.3-2](#), the estimated amount of sodium hypochlorite and surfactant used in ESWS blowdown was determined by dividing the quantity in [Table 3.3-2](#) by two (points of addition). The concentrations for TRC are based on limits in the Callaway Plant Unit1 NPDES 2006 Discharge Monthly Reports for cooling water, and are anticipated to be the same for Callaway Plant Unit 2.
- (3) Referring to [Table 3.3-2](#), an anti-foaming agent, complexing agent and/or precipitant may also be used. The anticipated pH range for Callaway Plant Unit 2 is based on the limits established in the Callaway Plant Unit 1 NPDES Permit for discharge from the neutralization tank.

Table 3.6-2—Non-Radioactive Gaseous Effluents

Source	Annual Releases in lbs/yr (kg/yr)			
	SO _x	Particulate	NO _x	Hydrocarbon
Emergency Diesel Generator	7.89 (3.58)	62.04 (28.14)	1,550.93 (703.50)	434.26 (196.98)
Station Blackout Diesel Generator	0.89 (0.40)	21.88 (9.92)	546.96 (248.10)	153.15 (69.47)
Annual Total (4xEDG + 2xSBO)	33.3 (15.1)	292 (132)	7,298 (3,310)	2,043 (927)

Table 3.6-3—{Anticipated Effluent Water Chemical Concentrations}

Outfall	Parameter(s) ⁽¹⁾	Concentration ⁽¹⁾ , mg/l	
		Daily Maximum	Monthly Average
Plant Effluent ⁽²⁾	Total Residual Chlorine	0.013	0.0075
Storm Water Runoff	Total Suspended Solids	100	30
	Oil & Grease	20	15
	pH	6.0 to 9.0	NA

Key:

mg/l – milligrams per liters

NA – Not Applicable

Notes:

- (1) The parameters and concentrations are based on current NPDES Permit limitations for the existing plant at outfalls monitoring similar system effluents. Similar to the existing plant, no quantity limitations on the above indicated parameters are anticipated.
- (2) Includes combined effluents for Turbine Island cooling tower blowdown, ESWS cooling tower blowdown, and miscellaneous low volume waste. For effluent concentrations associated with direct monitoring of cooling tower blowdown, refer to [Table 3.6-1](#). No direct monitoring of miscellaneous low volume waste is anticipated.

Table 3.6-4—{Biennial Hazardous Waste Management Callaway Plant Unit 1}

Hazardous Waste	Year/Quantity (lbs/kg)					
	2001		2003		2005	
	(lbs)	(kg)	(lbs)	(kg)	(lbs)	(kg)
Waste Paint with Solvent D001, D010, D039, F001, F002, D008, F003, F005	N/A	N/A	582	264	308	140
Waste Lab Solvent – D001, F001, F002, F003, D039, F005	N/A	N/A	372	169	319	145
Micrographics Cleaner, Corrosive, Sulfuric Acid, Sodium Dichromate – D002, D007	N/A	N/A	589	268	350	159
Unused Freon – 1,1,2-Trichloro-1, 2, 2-Trifluoroethane – F002	N/A	N/A	14	6.4	26	12
Silver Nitrate Photographic Waste – D001, D011, D007, D009	N/A	N/A	3	1.4	885	401
Leaded Silicone Radioactive Mixed Waste – D008	N/A	N/A	N/A	N/A	15	6.8
Lead Contaminated Rags – D008	N/A	N/A	N/A	N/A	362	164
Lead Paint Stripper – D008	N/A	N/A	N/A	N/A	74	34
Mercury Debris, Spill cleanup – D009	N/A	N/A	154	69.9	N/A	N/A
Spent Freon used in Testing, Mixed Radioactive Waste – F002	N/A	N/A	16	7.3	N/A	N/A
Solvent Containing Toluene, Mixed Radioactive Waste – D001, F005	N/A	N/A	33	15	N/A	N/A
Lab Chemicals – D001	14.5	6.58	N/A	N/A	N/A	N/A
Lab Chemicals – D005	218.0	(98.9)	N/A	N/A	N/A	N/A
Lab Chemicals – D008	6.0	2.72	N/A	N/A	N/A	N/A
Spent Lab Chemical – F002	16.0	7.26	N/A	N/A	N/A	N/A
Spent Solvent – D001, D039, F001, F002, F003	290.5	131.8	N/A	N/A	N/A	N/A
Spent Solvent – D039, D010, F001, F002	400	181.4	N/A	N/A	N/A	N/A
Lead Waste – D008	70	32	N/A	N/A	N/A	N/A
Spent Freon Used in Testing – F002	26	11.8	N/A	N/A	N/A	N/A
Unused Lab Chemical – U219	7	3.2	N/A	N/A	N/A	N/A
Corrosive Waste, Mixed Waste – D002, D007	275.0	124.7	N/A	N/A	N/A	N/A
Unused Aerosol Cans	11.5	5.22	N/A	N/A	N/A	N/A
Ignitable Aerosol Cans	230	104.3	N/A	N/A	N/A	N/A
Equipment Chemicals – D002, D007	557.5	252.9	N/A	N/A	N/A	N/A
Mercury Waste – D009	23.0	10.43	N/A	N/A	N/A	N/A
Lead Waste – D008	70.0	31.75	N/A	N/A	N/A	N/A
Spent Solvent, Mixed Waste	285.0	129.3	N/A	N/A	N/A	N/A
Ignitable Lab Pack	12.5	5.67	N/A	N/A	N/A	N/A
Total	2513	1140	1763	801	2339	1062
Key:						
N/A – Not Applicable						
(lbs) – pounds						
(kg) – kilogram						

3.7 POWER TRANSMISSION SYSTEM

{The transmission corridor siting is currently undergoing evaluation by the Midwest Independent Transmission System Operator (MISO) and has not been established. Therefore, construction of the transmission line required for the Callaway Plant Unit 2, as well as all impacts, are considered independent from the Callaway Plant Unit 2 project. These are discussed elsewhere in this document as appropriate under Cumulative Impacts.}

The NRC criteria for review of power transmission systems are presented in Section 3.7 of NUREG-1555 (NRC, 1999). To address these criteria, this section of the Environmental Report describes the transmission system from the {Callaway Plant Unit 2} switchyard to its connections with the existing {Callaway Plant Unit 1} transmission systems, including lines, corridors, towers, substations, and communication stations. {Callaway Plant Unit 2}, with an approximate additional 1,600 MWe net rating, would require the following new facilities and upgrades to connect to the existing transmission system:

- ◆ {One new 345 kV, 16 breaker, breaker-and-a-half switchyard to transmit power from Callaway Plant Unit 2.
- ◆ Two new 345 kV, 2,090 MVA (normal rating) circuits connecting the new Callaway Plant Unit 2 switchyard to the existing Callaway Plant Unit 1 switchyard,
- ◆ A new transmission line from a tie point on an existing 345 kV transmission line approximately 6.7 miles (11 km) south of the Callaway Plant Unit 1,
- ◆ Re-routing of the Callaway-Montgomery Lines 7 and 8 into the Callaway Plant Unit 2 switchyard from the Callaway Plant Unit 1 switchyard, and
- ◆ Completion of the breaker and a half scheme at Bland substation.}

The existing transmission system, constructed and operated for {Callaway Plant Unit 1}, was addressed in the Environmental Statement submitted with the original plant license application (UEC, 1975). The existing transmission system consists of two 345 kV circuits from the Montgomery Substation about 21 miles (34 km) to the northeast near New Florence, and a 345 kV circuit from the Bland Substation. The Bland Substation is located on the south side of the 345 kV Labadie-Franks-3 transmission line right-of-way, north of Owensville, Missouri and west of State Highway 19, and is about 29 airline miles (47 km) to the south of the plant. This arrangement provides two physically-separated offsite transmission lines comprised of three 345 kV circuits. The nearest distance between a Bland and a Montgomery line is the point where they attach to the switchyard arbors. The distance between the outside phases of these lines at this point will be approximately 1,300 ft (396 m). The existing transmission system will not be addressed in this section, except where it impacts or is impacted by the transmission facilities of Callaway Plant Unit 2. The routes for the existing two 345 kV circuits from the Callaway Plant site to the Montgomery Substation and single 345 kV circuit from the Callaway Plant site to the Bland Substation Generating Station are presented in [Figure 3.7-1](#).

Transmission System Expansion Planning will conform to the planning process established by the Midwest Independent Transmission System Operator (MISO).

3.7.1 {SWITCHYARD} AND CONNECTING CIRCUITS

3.7.1.1 {Callaway Plant Unit 2 Switchyard}

{The Callaway Plant Unit 2 switchyard occupies a 900 ft (274 m) by 650 ft (198 m) tract of land approximately 1,050 ft (320 m) southwest of the Callaway Plant Unit 2 reactor centerline and 700 ft (213 m) northwest of the existing switchyard as detailed in [Figure 3.7-2](#). The Callaway Plant Unit 2 switchyard is electrically integrated with the existing Callaway Plant Unit 1, 345 kV, switchyard by constructing two approximately 1 mile (1.6 km), 345 kV, 2,090 MVA lines on common towers. The Callaway Plant Unit 2 is connected to the transmission system via the two Callaway-Montgomery County lines to be re-routed to its switchyard.

The Callaway Plant Unit 2 switchyard and transmission lines is constructed in areas that, at present, are disturbed and utilized for other present Callaway Plant purposes, or are contiguous with the current transmission lines. Any vegetation which might be present will be removed. Areas under the transmission lines would be cleared of any vegetation that could pose a safety risk to the transmission system, either through arcing or reducing the structural integrity of towers.}

3.7.1.2 Connecting Circuits

{The Callaway Plant Unit 2 switchyard is electrically integrated with the existing Callaway Plant Unit 1, 345 kV, switchyard by constructing two approximately 1 mile (1.6 km), 345 kV, 2,090 MVA lines on common towers. A topographic map showing the location of the connecting circuits between the two switchyards is presented in [Figure 3.7-2](#).

A new transmission line is constructed from a tie point on an existing 345 kV transmission line approximately 6.7 miles (10.8 km) south of the Callaway Plant Unit 1 switchyard. This results in new 345 kV transmission line from a tie point on the existing Callaway-Loose Creek Line to the Callaway Plant Unit 1 switchyard. The new line is routed adjacent to the existing Callaway-Bland lines that are also routed to the Callaway Plant Unit 1 switchyard. An additional easement of 150 ft (46 m) is required over the existing 200 ft (61 m) easement for that section of the line not on AmerenUE property. This new Callaway-Loose Creek line allows the existing Callaway-Bland line to be restored as originally designed. The original Callaway-Bland line was broken apart in 2005 to allow routing of power to a new substation called Loose Creek This change was made to reduce line congestion to the south and to provide additional power to Jefferson City.

The routing of the new lines is contained within the Ameren UE property lines or widened offsite easements.}

3.7.2 ELECTRICAL DESIGN PARAMETERS

3.7.2.1 Circuit Design

The detailed design of the transmission lines has not begun but would include selection of the conductor and conductor configuration and the other design parameters specified by NUREG-1555 (NRC, 1999). Design and construction of transmission lines would be based on the guidance provided by the National Electric Safety Code (NESC) (ANSI/IEEE, applicable version), state and local regulations, {the Midwest Independent System Operator (MISO), and any requirements of the Missouri Public Service Commission (PSC)}.

{The conductors would be selected to meet the power delivery requirements of Callaway Plant Unit 2. The two 345 kV lines connecting the existing Callaway Plant Unit 1 switchyard and the Callaway Plant Unit 2 switchyard would be rated at 2,091 MVA. Each phase would use the same

dual sub-conductor bundles comprised of 954 circular mills, 54/7 ACSS conductors per phase with 18 inches (0.5 m) separation. There would also typically be two overhead ground wires of 3/8 inch (0.01 m), 7 strand, extra high strength steel wire.

The new portion of line from the tie point back to Callaway Plant Unit 1 would be comprised of two 954 circular mills, 54/7 ACSS conductors per phase with 18 inches (0.5 m) separation. There would also be typically two overhead ground wires of 3/8 inch (0.01 m), 7 strand, extra high strength steel wire.

The new lines would be designed to preclude crossing of lines wherever possible.}

3.7.2.2 Induced Current Analysis

{The design of the new transmission circuits would consider the potential for induced current as a design criterion. The NESC has a provision that describes how to establish minimum vertical clearances to the ground for electric lines having voltages exceeding 98 kV alternating current to ground. The clearance must limit the induced current due to electrostatic effects to 5 mA if the largest anticipated truck, vehicle, or equipment were short-circuited to ground. For this determination, the NESC specifies that the lines be evaluated assuming a final unloaded sag at 120°F (49°C). The calculation is a two step process in which the analyst first calculates the average field strength at 1.0 m (3.3 ft) above the ground beneath the minimum line clearance, and second, calculates the steady-state current value. The design and construction of the Callaway Plant Unit 2 switchyard and transmission circuits would comply with this NESC provision. At a minimum, conductor clearances over the ground would equal or exceed the NESC requirements.}

3.7.3 NOISE LEVELS

The noise impacts associated with the transmission system would be from three major sources: (1) corona from the transmission lines (a crackling or hissing noise); (2) operation of the {switchyard} transformers; and (3) maintenance work and vehicles.

3.7.3.1 Corona

Corona discharge is the electrical breakdown of air into charged particles caused by the electrical field at the surface of the conductors, and is increased by ambient weather conditions such as humidity, air density, wind, and precipitation and by irregularities on the energized surfaces.

{Audible noise from the corona effect for the existing 345 kV Callaway-Loose Creek line was calculated to be approximately 56 dB at the line surface during adverse weather conditions using the HVTRC calculation method. The noise level had attenuated to 52.1 dB at a distance of 80 ft (24.4 m) from the line. For reference, normal speech has a sound level of approximately 60 dB and a bulldozer idles at approximately 85 dB.

As shown in [Figure 3.7-2](#), the proposed Callaway Plant Unit 2 switchyard and transmission lines connecting the Callaway Plant Unit 2 switchyard to the existing Callaway Plant Unit 1 switchyard would be constructed entirely on the Callaway Plant site. The new portion of the Callaway-Loose Creek line would be constructed on the existing Callaway-Bland right-of-way. Therefore all newly constructed transmission systems will be contained in AmerenUE owned property or on existing easements.}

3.7.3.2 {Switchyard} Noise

{Switchyards} include transformer banks and circuit breakers that create “hum,” normally around 60 dBA, and occasional instantaneous sounds in the range of 70 to 90 dBA during activation of circuit breakers (SCE, 2006). {The proposed Callaway Plant Unit 2 switchyard would introduce these new noise sources (transformers and circuit breakers) to its location. The noise levels surrounding the switchyard would likely be close to 60 dBA near the switchyard fence, but would be significantly reduced near the site boundary, approximately 2,700 ft (820 m) to the south.}

3.7.3.3 Maintenance Noise

Regular inspections and maintenance of the transmission system and rights-of-way are performed. A patrol is performed twice annually of all transmission corridors, while more comprehensive inspections are performed on a rotating 5 year schedule. Maintenance is performed on an as-needed basis as dictated by the results of the line inspections and are generally performed on a {5 to 6 year rotating schedule for tree trimming, with herbicide use on a 3 year as-needed schedule.} The noise levels for maintenance activities would typically be those associated with tree trimming, spraying, mowing and vehicle driving. Noise levels for maintenance in the new onsite corridor are expected to be similar to those currently generated by maintenance activities.

3.7.4 STRUCTURE DESIGN

{The existing 345 kV transmission towers are designed and constructed to National Electric Safety Code and current AmerenUE standards. New towers added to support Callaway Plant Unit 2 will also conform to these criteria. The new towers will be wood H-frame design for the Loose Creek line, and will provide minimum clearances in accordance with the aforementioned standards. The two circuits connecting the existing Callaway Plant Unit 1 switchyard and the Callaway Plant Unit 2 switchyard would be carried on common steel towers. Near the plant switchyard the Loose Creek line may be changed to steel structures to minimize the footprint required for the towers. All structures would be grounded with a combination of ground rods or a ring counterpoise system. The towers for the new Callaway-Loose Creek line Missouri River crossing may exceed the 200 ft (61 m) height above the ground, thus navigation lights may be needed. The existing towers for the Callaway-Bland line have navigation beacons. Federal Aviation Administration permits may be required.}

3.7.5 INSPECTION AND MAINTENANCE

{Regular inspections and maintenance of the transmission system and rights-of-way will be performed. These inspections and maintenance include patrols and maintenance of transmission line hardware on a periodic and as-needed basis. Vegetation maintenance may include tree trimming and application of herbicide. Maintenance of the proposed onsite corridors including vegetation management will be implemented under AmerenUE existing programs in accordance with ANSI A300 (ANSI, 2001a; ANSI, 2006b) standards to promote safety, reliability, and environmental benefit.}

3.7.6 REFERENCES

{**ANSI/IEEE, applicable version.** National Electric Safety Code, ANSI/IEEE C2, version in effect at time of design, American National Standards Institute/Institute of Electrical and Electronics Engineers.

ANSI, 2001a. Tree, Shrub, and Other Woody Plant Maintenance – Standard Practices (Pruning), ANSI-A300 (Part 1), American National Standards Institute, 2001.

ANSI, 2001b. Integrated Vegetation Management, ANSI-A300 (Part 7), American National Standards Institute, 2001.

NRC, 1999. Environmental Standard Review Plan, NUREG-1555, Nuclear Regulatory Commission, October 1999.

SCE, 2006. Devers-Palo Verde 500 kV No. Project (Application No. A.05-04-015), Final Environmental Impact Report/ Environmental Impact Statement, State of California Public Utilities Commission, Southern California Edison, October 2006.

UEC, 1975. Final Environmental Statement Related to the Proposed Callaway Plant Units 1 and 2, Union Electric Company, docket nos. STN 50-483 and STN 50-486, U.S. Nuclear Regulatory Commission, March 1975.}

Figure 3.7-1—{Callaway Plant Unit 2 345 kV Circuit Corridors}

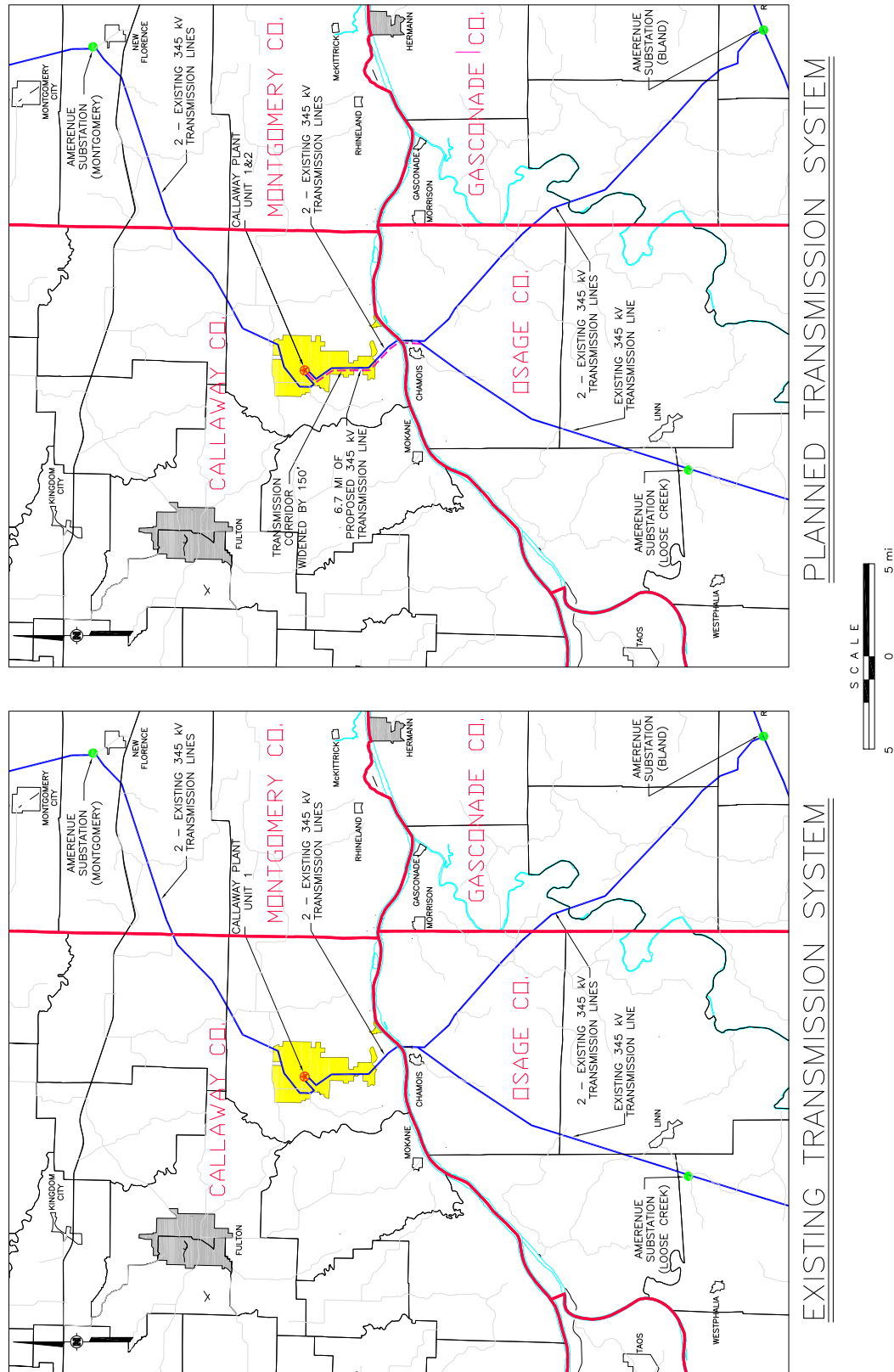
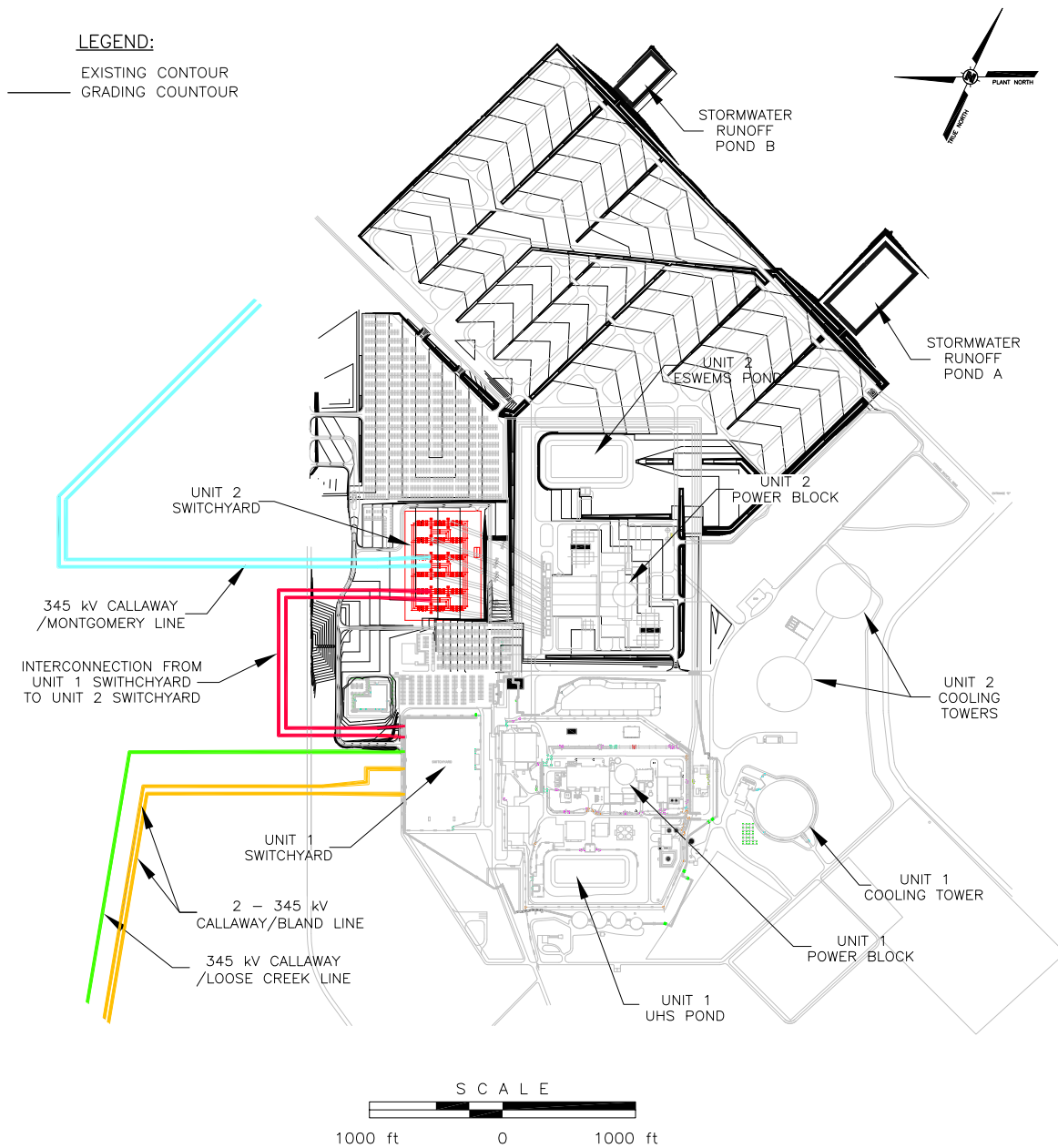


Figure 3.7-2—{Callaway Plant Unit 2 Topography and Switchyard Interconnection}



ER Chapter 3.0

3.8 TRANSPORTATION OF RADIOACTIVE MATERIALS

3.8.1 REACTOR DATA

The reactor for {Callaway Plant Unit 2} has a rated core thermal power of 4,590 (MWt). Although the U.S. EPR is to be licensed for 40 years, the proposed operating life of the U.S. EPR is 60 years.

The reactor core consists of 241 fuel assemblies. The fuel assembly structure supports the fuel rod bundles. Inside the assembly, the fuel rods are vertically arranged according to a square lattice with a 17x17 array. There are 265 fuel rods per assembly.

The fuel rods are composed of enriched uranium dioxide sintered pellets contained in a cladding tube made of M5[®] advanced zirconium alloy. The percentage of uranium enrichment and total quantities of uranium for the reactor core is as follows:

- ◆ Cycle 1 (initial) – average batch enrichment is between 2.23 to 3.14 weight percent U-235 and 2.66 weight percent U-235 for core reload with an enriched uranium weight of 285,483 lbs (129,493 kg).
- ◆ Cycle 2 (transition) – average batch enrichment is between 4.04 to 4.11 weight percent U-235 and 4.07 weight percent U-235 for core reload with an enriched uranium weight of 141,909 lbs (64,369 kg).
- ◆ Cycle 3 (transition) – average batch enrichment is between 4.22 to 4.62 weight percent U-235 and 4.34 weight percent U-235 for core reload with an enriched uranium weight of 113,395 lbs (51,435 kg).
- ◆ Cycle 4 (equilibrium) – average batch enrichment is between 4.05 to 4.58 weight percent U-235 and 4.30 weight percent U-235 for core reload with an enriched uranium weight of 113,417 lbs (51,445 kg).

Average batch enrichment is the average enrichment for each fuel assembly comprising a batch of fuel. The enrichment for core reload is the average enrichment for all fuel assemblies loaded in the core which is derived from the mass weighted average for the batches of fuel. The above values are 'beginning of life' enrichment values. Discharged enrichment values will be less at the 'end of life' of the assembly. Assembly enrichment reduction is directly proportional to the assembly burnup.

Discharge burnups for equilibrium cores are approximately between 45,000 and 59,000 MWd/MTU. The batch average discharge burnup for equilibrium cores is about 52,000 MWd/MTU.

3.8.2 ONSITE STORAGE FACILITIES FOR IRRADIATED FUEL

As discussed in Section 3.5.3, the spent fuel pool will be sized to accommodate at least 10 calendar years of wet storage, plus a full core offload. {Callaway Plant Unit 2} will utilize a 5 year minimum decay period between removal from the reactor and transportation offsite, as required by the Department of Energy (DOE) and as prescribed under 10 CFR 961, Appendix E, (CFR, 2007c).

3.8.3 TREATMENT AND PACKAGING OF RADIOACTIVE MATERIALS OTHER THAN IRRADIATED FUEL

Solid low level waste (LLW) shipped offsite for processing and disposal include dry activated wastes (DAW), aqueous cartridge type filters, solidified evaporator concentrates, resin beads, irradiated hardware, and small amounts of mixed wastes. The waste streams, annual generated volumes, and shipments are summarized in [Table 3.8-1](#).

The {Callaway Plant Unit 2} waste-streams identified in [Table 3.8-1](#) will be packaged in solid form in accordance with the requirements of 10 CFR 51.52(a) (4), 10 CFR 71, 49 CFR 173, and 49 CFR 178, (CFR 2007a, 2007b, 2007d, and 2007e), and as required for acceptance by the processor and disposal site's waste acceptance criteria.

3.8.4 TRANSPORTATION SYSTEM FOR FUEL AND OTHER RADIOACTIVE WASTES

Unirradiated fuel will be shipped to {Callaway Plant Unit 2} by {truck}.

The DOE is responsible for irradiated fuel shipments from {Callaway Plant Unit 2} to the repository. The DOE will make the decision regarding the mode of transport. It is anticipated that irradiated fuel will be shipped by truck or barge.

Radioactive waste from {Callaway Plant Unit 2} will be shipped by {truck}.

{Callaway Plant Unit 2} will operate in accordance with carrier procedures and policies that comply with the requirements of 10 CFR 51.52(a) (4), 10 CFR 71, 49 CFR 173, and 49 CFR 178, (CFR 2007a, 2007b, 2007d, and 2007e). {The procedures will be similar to those established for Callaway Plant Unit 1.}

3.8.5 TRANSPORTATION DISTANCE FROM THE PLANT TO THE STORAGE FACILITY

The detailed analysis of the transportation of fuel and wastes to and from the facility is provided in Sections 5.11 and 7.4. The discussion of the analysis includes the assumptions regarding the transportation distances to the appropriate storage facilities.

3.8.6 CONCLUSIONS

[Table 3.8-2](#) compares the conditions in 10 CFR 51.52(a) (CFR, 2007a) with the design parameters for {Callaway Plant Unit 2}. As noted in [Table 3.8-2](#), the design for Callaway Plant Unit 2 will not meet all of the conditions of 10 CFR 51.52(a) (CFR, 2007a). Therefore, the environmental impact from the transportation of fuel and wastes to and from the facility require detailed analyses as required in 10 CFR 51.52(b) (CFR, 2007a). Detailed analyses are presented in Sections 5.11 and 7.4.}

3.8.7 REFERENCES

{CFR, 2007a}. Code of Federal Regulations, Title 10, 51.52, Environmental Effects of Transportation of Fuel and Waste – Table S-4, 2007.

CFR, 2007b. Code of Federal Regulations, Title 10, Part 71, Packaging and Transportation of Radioactive Material, 2007.

CFR, 2007c. Code of Federal Regulations, Title 10, Part 961, Standard Contract for Disposal of Spent Nuclear Fuel and/or High-Level Radioactive Waste, Appendix E, 2007.

CFR, 2007d. Code of Federal Regulations, Title 49, Part 173, Shippers – General Requirements for Shipments and Packagings, 2007.

CFR, 2007e. Code of Federal Regulations, Title 49, Part 178, Specifications for Packagings, 2007.}

Table 3.8-1—Annual Solid Radioactive Wastes

Waste Type	Quantity ft ³ (m ³)	Activity Content ⁽¹⁾ Ci (Bq)		Shipping Volume ft ³ (m ³)	
		Expected	Maximum	Expected	Maximum
Solid Waste Stored in Drums					
Evaporator Concentrates ⁽²⁾	710 (20.1)	1.50E+02 (5.55E+12)	9.12E+03 (3.37E+14)	Varies	140 (3.96)
Spent Resins (Other)	90 (2.55)	1.07E+03 (3.96E+13)	5.23E+04 (1.93E+15)	90 (2.55)	
Spent Resins (Radwaste Demineralizer System)	140 (3.96)	1.50E+02 (5.55E+12)	9.12E+03 (3.37E+14)	140 (3.96)	
Wet Waste from Demineralizer	8 (0.23)	1.50E+02 (5.55E+12)	9.12E+03 (3.37E+14)	8 (0.23)	
Waste Drum for Solids Collection from Centrifuge System of Liquid Waste Storage & Processing	8 (0.23)	1.50E+02 (5.55E+12)	9.12E+03 (3.37E+14)	Varies	8 (0.23)
Filters (quantity)	120 (3.40)	6.86E+02 (2.54E+13)		120 (3.40)	
Sludge	70 (1.98)	1.50E+02 (5.55E+12)	9.12E+03 (3.37E+14)	Varies	35 (0.99)
Total Solid Waste Stored in Drums	1,146 (32.5)	2.51E+03 (9.29E+13)	9.86E+04 (3.65E+15)	358 (10.1)	541 (15.3)
Mixed Waste					
Mixed Waste	2 (0.057)	0.04 (1.48E+09)	2.43 (8.99E+10)	2 (0.057)	
Dry Active Waste (DAW)					
Non-Compressible DAW	70 (1.98)	2.97E-01 (1.10E+10)	1.81E+01 (6.70E+11)	70 (1.98)	
Compressible DAW	1,415 (40.1)	6.01E+00 (2.22E+11)	3.66E+02 (1.35E+13)	1415 (40.1)	
Combustible DAW	5,300 (150.1)	3.19E+01 (1.18E+12)	1.94E+03 (7.18E+13)	5,300 (150.1)	
Total Dry Active Waste	6,785 (192.1)	3.82E+01 (1.41E+12)	2.32E+03 (8.58E+13)	Varies	
Overall Totals	7,933 (224.6)	2.55E+03 (9.43E+13)	1.01E+05 (3.74E+15)	Varies	

Notes:

- (1) Activity contents represent waste activity after a defined period (i.e., 6 months) that covers onsite storage before shipping.
- (2) The volume of evaporator concentrates and sludge, and the number of waste drums will be determined by the method of treatment.

Table 3.8-2—Transportation Environmental Impact Comparison

10 CFR 51.52(a) Parameter	10 CFR 51.52(a) Condition	{Callaway Plant Unit 2}
(1) Reactor Power Level, MWt	3,800	4,590
(2) Fuel Form and U235 Enrichment, weight percent	Zircaloy encapsulated sintered uranium dioxide pellets at 4.0	M5 ⁺ advanced zirconium alloy encapsulated sintered uranium dioxide pellets at 4.58
(3) Average Irradiation Level and Minimum Decay, MWd/MTU	33,000 at 90 days decay	52,000 at 5 years decay
(4) Radioactive Waste Physical Form	Packaged as Solid	Packaged as Solid
(5) Transport Mode	New Fuel: Truck Irradiated Fuel: Truck, Rail, Barge LLW: Truck, Rail	New Fuel: Truck Irradiated Fuel: Truck, Barge LLW: Truck
(6) Environmental Impacts	Table S-4 of 10 CFR 51.52	Refer to Sections 5.11 and 7.4

Note:

LLW – Low Level Waste