

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401
400 Chestnut Street Tower II

May 17, 1983

WBRD-50-390/82-48
WBRD-50-391/82-45

U.S. Nuclear Regulatory Commission
Region II
Attn: Mr. James P. O'Reilly, Regional Administrator
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

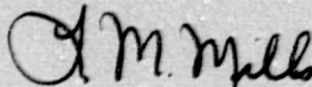
WATTS BAR NUCLEAR PLANT UNITS 1 AND 2 - INCORRECT SUPPORTS IN ANALYSIS
OF 3-INCH CVCS LINE - WBRD-50-390/82-48, WBRD-50-391/82-45 - THIRD
INTERIM REPORT

The subject deficiency was initially reported to NRC-OIE Inspector
D. Quick on May 5, 1982 in accordance with 10 CFR 50.55(e) as NCR
WBN C&E 8209. Interim reports were submitted on June 3, and December 15,
1982. Enclosed is our third interim report. We expect to submit our next
report on or about September 22, 1983.

If you have any questions, please get in touch with R. H. Shell at
FTS 858-2688.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



L. M. Mills, Manager
Nuclear Licensing

Enclosure

cc: Mr. Richard C. DeYoung, Director (Enclosure)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Records Center (Enclosure)
Institute of Nuclear Power Operations
1100 Circle 75 Parkway, Suite 1500
Atlanta, Georgia 30339

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ENCLOSURE

WATTS BAR NUCLEAR PLANT UNITS 1 AND 2
INCORRECT SUPPORTS IN ANALYSIS OF 3-INCH CVCS LINE
NCR WBN CEB 8209
WBN-50-390/82-18, WBRD-50-391/82-45
10 CFR 50.55(e)
THIRD INTERIM REPORT

Description of Deficiency

The 3-inch letdown line in the CVCS system from the crossover leg loop No. 3 to the regenerative heat exchanger has two inline isolation valves (node points 51 and 150 of EDS Nuclear, Incorporated's, analysis 0600200-03-10) which are analyzed with rigid supports on the valve operators in one direction. The as-designed supports for these locations are snubbers. This condition results in the analysis being incorrect with respect to pipe movements, pipe stresses, and support loads. Additional rigid support piping analysis problems, recently identified on safety injection system, steam generator blowdown system, and other CVCS lines, are also affected.

Interim Progress

TVA has reanalyzed problem 0600200-08-10 which was the only analysis problem identified on the original NCR. However, additional problems were identified and are currently being reanalyzed. Field modifications will be required. The reanalysis effort is being performed under Engineering Change Notices (ECNs) 3474 and 3653. Additional information will be provided in our next submittal.