



Tennessee Valley Authority, 1101 Market Street, LP 5A, Chattanooga, Tennessee 37402-2801

August 29, 2008

10 CFR 52.79

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

In the Matter of )  
Tennessee Valley Authority )

Docket No. 52-014 and 52-015

**BELLEVILLE COMBINED LICENSE APPLICATION – RESPONSE TO REQUEST FOR  
ADDITIONAL INFORMATION – REGULATORY GUIDE CONFORMANCE**

Reference: Letter from Ravindra G. Joshi (NRC) to Andrea L. Sterdis (TVA), Request for  
Additional Information Letter No. 071 Related to SRP Section 01 for the  
Belleville Units 3 and 4 Combined License Application, dated July 16, 2008

This letter provides the Tennessee Valley Authority's (TVA) response to the Nuclear Regulatory  
Commission's (NRC) request for additional information (RAI) items included in the reference  
letter.

A response to the NRC request in the subject letter is addressed in the enclosure which also  
identifies associated changes that will be made in a future revision of the BLN application.

If you should have any questions, please contact Tom Spink at 1101 Market Street, LP5A,  
Chattanooga, Tennessee 37402-2801, by telephone at (423) 751-7062, or via email at  
tespink@tva.gov.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on this 29<sup>th</sup> day of AUG, 2008.

  
Jack A. Bailey  
Vice President, Nuclear Generation Development

Enclosure  
cc: See Page 2

*DOB5  
NRD*

Document Control Desk  
Page 2  
August 29, 2008

cc: (w/Enclosure)

J. P. Berger, EDF  
E. Cummins, Westinghouse  
S. P. Frantz, Morgan Lewis  
M. W. Gettler, FP&L  
R. C. Grumbir, NuStart  
P. S. Hastings, NuStart  
P. Hinnenkamp, Entergy  
R. G. Joshi, NRC/HQ  
M. C. Kray, NuStart  
D. Lindgren, Westinghouse  
G. D. Miller, PG&N  
M. C. Nolan, Duke Energy  
N. T. Simms, Duke Energy  
G. A. Zinke, NuStart

cc: (w/o Enclosure)

B. C. Anderson, NRC/HQ  
M. M. Comar, NRC/HQ  
B. Hughes, NRC/HQ  
R. H. Kitchen, PGN  
M. C. Kray, NuStart  
A. M. Monroe, SCE&G  
C. R. Pierce, SNC  
R. Reister, DOE/PM  
L. Reyes, NRC/RII  
T. Simms, NRC/HQ  
K. N. Slays, NuStart  
J. M. Sebrosky, NRC/HQ

Enclosure  
TVA letter dated August 29, 2008  
RAI Responses

Responses to NRC Request for Additional Information letter No. 071 dated July 16, 2008  
(13 pages, including this list)

Subject: Regulatory Guide Conformance in the Final Safety Analysis Report

<u>RAI Number</u>	<u>Date of TVA Response</u>
01-07	This letter – see following pages

<u>Associated Additional Attachments / Enclosures</u>	<u>Pages Included</u>
None	

Enclosure  
TVA letter dated August 29, 2008  
RAI Responses

**NRC Letter Dated: July 16, 2008**

**NRC Review of Final Safety Analysis Report**

**NRC RAI NUMBER: 01-07**

Appendix 1A of the DCD provides an evaluation of the degree of AP1000 compliance with Regulatory Guides. For those Regulatory Guides applicable to AP1000, Table 1.9-1 of the DCD identifies the appropriate DCD cross-references. The cross-referenced sections contain descriptive information applicable to the regulatory guide positions found in Appendix 1A of the DCD.

Bellefonte FSAR Section 1.9.1.1 adds certain paragraphs (Pages 1.9-1 and 2) to the end of DCD Section 1.9.1.1. FSAR Section 1.9.1.1 states in part, "Appendix 1AA provides an evaluation of the degree of compliance with Division 1 regulatory guides as applicable to the content of this FSAR, or to the site-specific design, construction and/or operational aspects." In addition, FSAR section 1.9.1.1 states that "Table 1.9-201 identifies the appropriate regulatory guide to FSAR cross-references. The cross-referenced sections contain descriptive information applicable to the regulatory guide positions found in Appendix 1AA."

SRP Section 1.0 (Page 1.0-4), Item 9 of Areas of Review states, "A table of conformance with the NRC's regulatory guides that are applicable to the application is reviewed. The table should also include an identification and description of deviation from the guidance contained in the NRC's regulatory guides as well as suitable justifications for any alternative approaches proposed by the COL applicant **with appropriate references to the FSAR sections where they are addressed.**" (Emphasis added)

Appendix 1AA documents a conformance discussion for Regulatory Guides. However, Appendix 1AA does not provide cross references to FSAR sections for certain Regulatory Guides (e.g., Reg. Guides 1.133, 1.135, 1.143, 1.147, 1.154, 1.161) for locating the applicable descriptive information. Please note that this list does not include all the regulatory guides that do not have applicable cross references to FSAR sections; it is a sample for illustrative purposes. As the above-listed Regulatory Guides are not identified in Table 1.9-201, no descriptive information is referenced for the staff review. Accordingly, provide appropriate references to FSAR sections where conformance with specific Regulatory Guides is addressed, or justify why the cross references to the FSAR sections are not necessary.

**BLN RAI ID: 0746**

**BLN RESPONSE:**

As indicated in the question, FSAR Appendix 1AA does not provide cross references to FSAR sections for certain Regulatory Guides. Generally, FSAR Appendix 1AA does not provide any cross references to FSAR sections for any Regulatory Guides. A cross-reference to FSAR discussions of Regulatory Guides is provided in Table 1.9-201. However, since Table 1.9-201 does not currently address each Regulatory Guide identified in FSAR Appendix 1AA, it will be revised to include the additions and revisions identified in the Application Revisions section below. This change will make the Table entries consistent with the entries in the recently revised FSAR Appendix 1AA (see response to NRC RAI 01-05, BLN-RAI-LTR-066).

Note, however, that the FSAR does not contain "applicable descriptive information" to support each conformance statement in FSAR Appendix 1AA. Many of the Regulatory Guides listed in FSAR Appendix 1AA (and now FSAR Table 1.9-201) are fully addressed by the DCD and no additional information is provided in the FSAR. One such example of this type is Regulatory Guide 1.161. Appendix 1AA may list the Regulatory Guide simply because a later revision of the guidance has been issued since the DCD. Further, many of the Regulatory Guides listed in Appendix 1AA are not otherwise appropriate for discussion in the FSAR as evidenced by the lack of a request for discussion of these Regulatory Guides in other sections of the FSAR in Regulatory Guide 1.206 and in the SRPs. Finally,

Enclosure  
 TVA letter dated August 29, 2008  
 RAI Responses

FSAR Table 1.9-201 lists cross-references only where the Regulatory Guide is specifically mentioned in the FSAR. For example, Regulatory Guide 1.133, Loose-Part Detection Program for the Primary System of Light-Water-Cooled Reactors, has been added to Table 1.9-201, but since it is not specifically discussed in any other FSAR section, no FSAR cross reference is identified. The procedures discussed in this guidance are generally addressed by the procedures discussion in FSAR Subsection 13.5, but the Regulatory Guide is not specifically mentioned there. Thus, there is no FSAR section that contains a reference to this Regulatory Guide.

This response is expected to be STANDARD for the S-COLAs.

**ASSOCIATED BLN COL APPLICATION REVISIONS:**

1. COLA Part 2, FSAR, Chapter 1, Table 1.1-201 will be revised to include a new listing for an Acronym Used in the FSAR to read:

TS	Technical Specification(s)
----	----------------------------

2. COLA Part 2, FSAR, Chapter 1, Table 1.9-201 will be revised to include the new or revised Regulatory Guide cross reference information as shown below (Note that, for convenience, this listing is a complete replacement of the content of Table 1.9-201 including previously identified errata items. Change bars in the right margin identify the revised information.):

<b>Regulatory Guides</b>	<b>FSAR Chapter, Section, or Subsection</b>
<b>Division 1 Regulatory Guides</b>	
1.6 Independence Between Redundant Standby (Onsite) Power Sources and Between Their Distribution Systems (Rev. 0, March 1971)	16 (TS Bases 3.8.1)
1.7 Control of Combustible Gas Concentrations in Containment (Rev. 3, March 2007)	DCD discussion only; see DCD Table 1.9-1
1.8 Qualification and Training of Personnel for Nuclear Power Plants (Rev. 3, May 2000)	12.1 (NEI 07-08) Appendix 12AA Appendix 12AA (NEI 07-03) 13.1.1.4 13.1.3.1 13.2 (NEI 06-13A) 16 (TS 5.3.1)
1.12 Nuclear Power Plant Instrumentation for Earthquakes (Rev. 2, March 1997)	3.7.4.1
1.13 Spent Fuel Storage Facility Design Basis (Rev. 2, March 2007)	16 (TS Bases 3.7.11) 16 (TS Bases 3.7.12)
1.16 Reporting of Operating Information – Appendix A Technical Specifications (Rev. 4, August 1975)	14.2.3.2

Enclosure  
TVA letter dated August 29, 2008  
RAI Responses

1.20	Comprehensive Vibration Assessment Program for Reactor Internals During Preoperational and Initial Startup Testing (Rev. 3, March 2007)	DCD discussion only; see DCD Table 1.9-1
1.21	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents From Light-Water-Cooled Nuclear Power Plants (Rev.1, June 1974)	2.3.3.2.3 11.5.1.2 11.5.4.1 12.3.4
1.23	Meteorological Monitoring Program for Nuclear Power Plants (Rev. 1, March 2007)	2.3.3.2 2.3.4.2 2.3.4.3
1.26	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-Waste-Containing Components of Nuclear Power Plants (Rev. 4, March 2007)	5.2.4.1 17.5 (QAPD IV)
1.27	Ultimate Heat Sink for Nuclear Power Plants (Rev. 2, January 1976)	2.4.11.6
1.28	Quality Assurance Program Requirements (Design and Construction) (Rev. 3, August 1985)	Not referenced; see Appendix 1AA
1.29	Seismic Design Classification (Rev. 4, March 2007)	17.5 (QAPD IV)
1.30	Quality Assurance Requirements for the Installation, Inspection, and Testing of Instrumentation and Electric Equipment (Rev. 0, August 1972)	Not referenced; see Appendix 1AA
1.31	Control of Ferrite Content in Stainless Steel Weld Metal (Rev. 3, April 1978)	6.1.1.2
1.32	Criteria for Power Systems for Nuclear Power Plants (Rev. 3, March 2004)	16 (TS Bases 3.8.1)
1.33	Quality Assurance Program Requirements (Operation) (Rev. 2, February 1978)	16 (TS 5.4.1)
1.37	Quality Assurance Requirements for Cleaning of Fluid Systems and Associated Components of Water Cooled Nuclear Power Plants (Rev. 1, March 2007)	17.5 (QAPD IV)
1.38	Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage and Handling of Items for Water- Cooled Nuclear Power Plants (Rev. 2, May 1977)	DCD discussion only; see DCD Table 1.9-1
1.39	Housekeeping Requirements for Water-Cooled Nuclear Power Plants (Rev. 2, September 1977)	DCD discussion only; see DCD Table 1.9-1
1.44	Control of the Use of Sensitized Stainless Steel (Rev. 0, May 1973)	6.1.1.2

Enclosure  
TVA letter dated August 29, 2008  
RAI Responses

1.45	Reactor Coolant Pressure Boundary Leakage Detection Systems (Rev. 0, May 1973)	16 (TS Bases 3.4.7) 16 (TS Bases 3.4.9)
1.53	Application of the Single-Failure Criterion to Nuclear Power Plant Protection Systems (Rev. 2, November 2003)	DCD discussion only; see DCD Table 1.9-1
1.54	Service Level I, II, and III Protective Coatings Applied to Nuclear Power Plants (Rev. 1, July 2000)	1.9.4.2.3 6.1.2.1.6
1.57	Design Limits and Loading Combinations for Metal Primary Reactor Containment System Components (Rev. 1, March 2007)	DCD discussion only; see DCD Table 1.9-1
1.59	Design Basis Floods for Nuclear Power Plants (Rev. 2, August 1977)	2.4.2.2 2.4.3 2.4.4 2.4.4.1 2.4.5
1.60	Design Response Spectra for Seismic Design of Nuclear Power Plants (Rev. 1, December 1973)	Table 2.0-201
1.61	Damping Values for Seismic Design of Nuclear Power Plants (Rev. 1, March 2007)	DCD discussion only; see DCD Table 1.9-1
1.68	Initial Test Program for Water-Cooled Nuclear Power Plants (Rev. 3, March 2007)	16 (TS Bases 3.1.8)
1.70	Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (LWR Edition) (Rev. 3, November 2007)	1.1.6.1
1.71	Welder Qualification for Areas of Limited Accessibility (Rev 1, March 2007)	DCD discussion only; see DCD Table 1.9-1
1.75	Criteria for Independence of Electrical Safety Systems (Rev 3, February 2005)	DCD discussion only; see DCD Table 1.9-1
1.76	Design-Basis Tornado and Tornado Missiles for Nuclear Power Plants (Rev. 1, March 2007)	Table 2.0-201 2.3.1.4
1.77	Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized Water Reactors (Rev 0, May 1974)	16 (TS Bases 3.2.1) 16 (TS Bases 3.2.2) 16 (TS Bases 3.2.4) 16 (TS Bases 3.2.5)
1.78	Evaluating the Habitability of a Nuclear Power Plant Control Room During a Postulated Hazardous Chemical Release (Rev. 1, December 2001)	2.2.3.1.3 6.4.4.2 16 (TS Bases 3.7.8)
1.82	Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant Accident (Rev. 3, November 2003)	DCD discussion only; see DCD Table 1.9-1
1.83	Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes (Rev. 1, July 1975)	DCD discussion only; see DCD Table 1.9-1

Enclosure  
TVA letter dated August 29, 2008  
RAI Responses

1.84	Design, Fabrication, and Materials Code Case Acceptability, ASME Section III (Rev. 33, August 2005)	DCD discussion only; see DCD Table 1.9-1
1.86	Termination of Operating Licenses for Nuclear Reactors (Rev. 0, June 1974)	Not referenced; see Appendix 1AA
1.91	Evaluations of Explosions Postulated To Occur on Transportation Routes Near Nuclear Power Plants (Rev. 1, February 1978)	2.2.3.1.1 2.2.3.2 3.5.1.5
1.92	Combining Modal Responses and Spatial Components in Seismic Response Analysis (Rev. 2, July 2006)	DCD discussion only; see DCD Table 1.9-1
1.93	Availability of Electric Power Sources (Rev. 0, December 1974)	16 (TS Bases 3.8.1) 16 (TS Bases 3.8.5)
1.94	Quality Assurance Requirements for Installation, Inspection and Testing of Structural Concrete and Structural Steel During the Construction Phase of Nuclear Power Plants (Rev. 1, July 1976)	Not referenced; see Appendix 1AA
1.97	Criteria For Accident Monitoring Instrumentation For Nuclear Power Plants (Rev. 4, June 2006)	Appendix 12AA (NEI 07-03) 16 (TS Bases 3.3.3)
1.99	Radiation Embrittlement of Reactor Vessel Materials (Rev. 2, May 1988)	16 (TS Bases 3.4.3)
1.101	Emergency Response Planning and Preparedness for Nuclear Power Reactors (Rev. 5, June 2005)	9.5.1.8.2.2 Table 9.5-201 13.3 (Emergency Plan I.C.1)
1.109	Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I (Rev. 1, October 1977)	11.2.3.5 11.3.3.4 12.4.1.9.3 Table 12.4-201
1.110	Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power Reactors (Rev. 0, March 1976)	11.2.3.5.3 11.3.3.4.3
1.111	Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors (Rev. 1, July 1977)	2.3.5.1 12.4.1.9.3
1.112	Calculation of Releases of Radioactive Materials in Gaseous or Liquid Effluents from Light-Water-Cooled Power Reactors (Rev. 1, March 2007)	DCD discussion only; see DCD Table 1.9-1
1.113	Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor Releases for the Purpose of Implementing Appendix I (Rev. 1, April 1977)	11.2.3.3



Enclosure  
TVA letter dated August 29, 2008  
RAI Responses

1.114	Guidance to Operators at the Controls and to Senior Operators in the Control Room of a Nuclear Power Unit (Rev. 2, May 1989)	13.1.2.1.2.6 13.1.2.1.3
1.115	Protection Against Low-Trajectory Turbine Missiles (Rev. 1, July 1977)	3.5.1.3
1.116	Quality Assurance Requirements for Installation, Inspection, and Testing of Mechanical Equipment and Systems (Rev. 0-R, May 1977)	Not referenced; see Appendix 1AA
1.121	Bases for Plugging Degraded PWR Steam Generator Tubes (Rev. 0, August 1976)	16 (TS Bases 3.4.18)
1.124	Service Limits and Loading Combinations for Class 1 Linear-Type Supports (Rev. 2, February 2007)	DCD discussion only; see DCD Table 1.9-1
1.128	Installation Design and Installation of Vented Lead-Acid Storage Batteries for Nuclear Power Plants (Rev. 2, February 2007)	DCD discussion only; see DCD Table 1.9-1
1.129	Maintenance, Testing, and Replacement of Vented Lead-Acid Storage Batteries for Nuclear Power Plants (Rev. 2, February 2007)	Table 8.1-201 8.3.2.1.4 16 (TS Bases 3.8.1)
1.130	Service Limits and Loading Combinations for Class 1 Plate-And-Shell-Type Supports (Rev. 2, March 2007)	DCD discussion only; see DCD Table 1.9-1
1.132	Site Investigations for Foundations of Nuclear Power Plants (Rev. 2, October 2003)	2.5.4.2.1 2.5.4.4
1.133	Loose-Part Detection Program for the Primary System of Light-Water-Cooled Reactors (Rev. 1, May 1981)	DCD discussion only; see DCD Table 1.9-1
1.134	Medical Evaluation of Licensed Personnel at Nuclear Power Plants (Rev. 3, March 1998)	Not referenced; see Appendix 1AA
1.135	Normal Water Level and Discharge at Nuclear Power Plants (Rev. 0, September 1977)	DCD discussion only; see DCD Table 1.9-1
1.138	Laboratory Investigations of Soils and Rocks for Engineering Analysis and Design of Nuclear Power Plants (Rev. 2, December 2003)	2.5.4.2.1
1.139	Guidance for Residual Heat Removal (Rev. 0, May 1978)	DCD discussion only; see DCD Table 1.9-1
1.140	Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Normal Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants (Rev. 2, June 2001)	9.4.1.4 9.4.7.4 16 (TS Bases 3.9.6)
1.143	Design Guidance for Radioactive Waste Management Systems, Structures, and Components Installed in Light-Water-Cooled Nuclear Power Plants (Rev. 2, November 2001)	11.2.1.2.5.2 11.2.3.6 11.3.3.6 11.4.5 11.4.6.2

Enclosure  
TVA letter dated August 29, 2008  
RAI Responses

1.145	Atmospheric Dispersion Models for Potential Accident Consequence Assessments at Nuclear Power Plants (Rev. 1, November 1982)	2.3.4.1 2.3.4.2
1.147	Inservice Inspection Code Case Acceptability, ASME section XI, Division 1 (Rev. 14, August 2005)	5.2.4 6.6
1.149	Nuclear Power Plant Simulation Facilities for Use in Operator Training and License Examinations (Rev. 3, October 2001)	13.2 (NEI 06-13)
1.150	Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice Examinations (Rev. 1, February 1983)	DCD discussion only; see DCD Table 1.9-1
1.152	Criteria for Use of Computers in Safety Systems of Nuclear Power Plants (Rev. 2, January 2006)	DCD discussion only; see DCD Table 1.9-1
1.154	Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis Reports for Pressurized Water Reactors (Rev. 0, January 1987)	Not referenced; see Appendix 1AA
1.155	Station Blackout (Rev. 0, August 1998)	Table 8.1-201
1.159	Assuring the Availability of Funds for Decommissioning Nuclear Reactors (Rev. 1, October 2003)	Table 8.1-201
1.160	Monitoring the Effectiveness of Maintenance at Nuclear Power Plants (Rev. 2, March 1997)	17.6 (NEI 07-02A)
1.162	Format and Content of Report for Thermal Annealing of Reactor Pressure Vessels (Rev. 0, February 1996)	Not referenced; see Appendix 1AA
1.163	Performance-Based Containment Leak-Test Program (Rev. 0, September 1995)	6.2.5.1 6.2.5.2.2 16 (TS 5.5.8)
1.165	Identification and Characterization of Seismic Sources and Determination of Safe Shutdown Earthquake Ground Motion (Rev. 0, March 1997)	2.5.2.4.4 2.5.2.4.4.5 2.5.2.6.1
1.166	Pre-Earthquake Planning and Immediate Nuclear Power Plant Operator Post Earthquake Actions (Rev. 0, March 1997)	3.7.4.4
1.167	Restart of a Nuclear Power Plant Shut Down by a Seismic Event (Rev. 0, March 1997)	3.7.4.4
1.168	Verification, Validation, Reviews, and Audits for Digital Computer Software Used in Safety Systems of Nuclear Power Plants (Rev. 1, February 2004)	DCD discussion only; see DCD Table 1.9-1

Enclosure  
TVA letter dated August 29, 2008  
RAI Responses

1.174	An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis (Rev. 1, November 2002)	Not referenced; see Appendix 1AA
1.175	An Approach for Plant-Specific, Risk- Informed Decisionmaking: Inservice Testing (Rev. 0, August 1998)	Not referenced; see Appendix 1AA
1.177	An Approach for Plant-Specific, Risk- Informed Decisionmaking: Technical Specifications (Rev. 0, August 1998)	16 (TS Bases 3.5.1) 16 (TS Bases 3.7.10)
1.178	An Approach for Plant-Specific Risk- Informed Decisionmaking for Inservice Inspection of Piping (Rev. 1, September 2003)	Not referenced; see Appendix 1AA
1.179	Standard Format and Content of License Termination Plans for Nuclear Power Reactors (Rev. 0, January 1999)	Not referenced; see Appendix 1AA
1.180	Guidelines for Evaluating Electromagnetic and Radio-Frequency Interference in Safety-Related Instrumentation and Control Systems (Rev. 1, October 2003)	DCD discussion only; see DCD Table 1.9-1
1.181	Content of Updated Final Safety Analysis Report in Accordance with 10 CFR 50.71(e) (Rev. 0, September 1999)	Not referenced; see Appendix 1AA
1.182	Assessing and Managing Risk Before Maintenance Activities at Nuclear Power Plants (Rev. 0, May 2000)	16 (TS Bases SR 3.0.3) 17.6 (NEI 07-02A)
1.183	Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors (Rev. 0, July 2000)	16 (TS Bases 3.7.5) 16 (TS Bases 3.9.4) 16 (TS Bases 3.9.7)
1.184	Decommissioning of Nuclear Power Reactors (Rev. 0, July 2000)	Not referenced; see Appendix 1AA
1.185	Standard Format and Content for Post-shutdown Decommissioning Activities Report (Rev. 0, July 2000)	Not referenced; see Appendix 1AA
1.186	Guidance and Examples for Identifying 10 CFR 50.2 Design Bases (Rev. 0, December 2000)	Not referenced; see Appendix 1AA
1.187	Guidance for Implementation of 10 CFR 50.59, Changes, Tests, and Experiment (Rev. 0, November 2000)	Not referenced; see Appendix 1AA
1.188	Standard Format and Content for Applications To Renew Nuclear Power Plant Operating Licenses (Rev. 1, September 2005)	Not referenced; see Appendix 1AA
1.189	Fire Protection for Nuclear Power Plants (Rev. 1, March 2007)	9.5.1.8.1.1 9.5.1.8.2.2 Appendix 9A 13.1.2.1.2.9 17.5 (QAPD III.2)

Enclosure  
TVA letter dated August 29, 2008  
RAI Responses

1.191	Fire Protection Program for Nuclear Power Plants During Decommissioning and Permanent Shutdown (Rev. 0, May 2001)	Not referenced; see Appendix 1AA
1.192	Operation and Maintenance Code Case Acceptability, ASME OM Code (Rev. 0, June 2003)	Not referenced; see Appendix 1AA
1.193	ASME Code Cases Not Approved for Use (Rev 1, August 2005)	Not referenced; see Appendix 1AA
1.194	Atmospheric Relative Concentrations for Control Room Radiological Habitability Assessments at Nuclear Power Plants (Rev. 0, June 2003)	2.3.4.3
1.195	Methods and Assumptions for Evaluating Radiological Consequences of Design Basis Accidents at Light-Water Nuclear Power Reactors (Rev. 0, May 2003)	Not referenced; see Appendix 1AA
1.196	Control Room Habitability at Light-Water Nuclear Power Reactors (Rev. 1, January 2007)	Not referenced; see Appendix 1AA
1.197	Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors (Rev. 0, 2003)	DCD discussion only; see DCD Table 1.9-1
1.198	Procedures and Criteria for Assessing Seismic Soil Liquefaction at Nuclear Power Plant Sites (Rev. 0, November 2003)	2.5.4.8
1.199	Anchoring Components and Structural Supports in Concrete (Rev. 0, November 2003)	Not referenced; see Appendix 1AA
1.200	An Approach for Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk- Informed Activities (Rev. 1, January 2007)	19.59.10.6
1.201	Guidelines for Categorizing Structures, Systems, and Components in Nuclear Power Plants According to Their Safety Significance (Rev. 1, May 2006)	Not referenced; see Appendix 1AA
1.202	Standard Format and Content of Decommissioning Cost Estimates for Nuclear Power Reactors (Rev. 0, 2005)	Not referenced; see Appendix 1AA
1.203	Transient and Accident Analysis Methods (Rev. 0, December 2005)	Not referenced; see Appendix 1AA
1.204	Guidelines for Lightning Protection of Nuclear Power Plants (Rev. 0, November 2005)	Table 8.1-201
1.205	Risk-Informed, Performance-Based Fire Protection for Existing Light-Water Nuclear Power Plants (Rev. 0, May 2006)	Not referenced; see Appendix 1AA

Enclosure  
TVA letter dated August 29, 2008  
RAI Responses

1.206	Combined License Applications for Nuclear Power Plants (LWR Edition) (Rev. 0, June 2007)	See Appendix 1AA 1.1.6.1 2.0 2.5 2.5.1 2.5.4 14.3.2.3.1 14.3.2.3.2 Table 8.1-201
1.207	Guidelines for Evaluating Fatigue Analyses Incorporating the Life Reduction of Metal Components Due to the Effects of the Light-Water Reactor Environment for New Reactors (Rev. 0, March 2007)	Not referenced; see Appendix 1AA
1.208	A Performance-Based Approach to Define the Site-Specific Earthquake Ground Motion (Rev. 0, March 2007)	2.5.1 2.5.2 2.5.3 2.5.4
1.209	Guidelines for Environmental Qualification of Safety-Related Computer-Based Instrumentation and Control Systems in Nuclear Power Plants (Rev. 0, March 2007)	Not referenced; see Appendix 1AA

**Division 4 Regulatory Guides**

4.7	General Site Suitability Criteria for Nuclear Power Stations (Rev. 2, April 1998)	Not referenced; see Appendix 1AA
4.15	Quality Assurance for Radiological Monitoring Programs (Inception through Normal Operations to License Termination) – Effluent Streams and the Environment (Rev. 1, February 1979)	11.5.1.2 11.5.3 11.5.4 11.5.6.5

**Division 5 Regulatory Guides**

5.9	Guidelines for Germanium Spectroscopy Systems for Measurements of Special Nuclear Material (Rev. 2, December 1983)	Not referenced; see Appendix 1AA
5.12	General Use of Locks in the Protection and Controls of Facilities and Special Nuclear Materials (Rev. 0, November 1973)	Not referenced; see Appendix 1AA
5.65	Vital Area Access Controls, Protection of Physical Security Equipment, and Key and Lock Controls (Rev. 0, September 1986)	Not referenced; see Appendix 1AA

**Division 8 Regulatory Guides**

8.2	Guide for Administrative Practices in Radiation Monitoring (Rev. 0, February 1973)	12.1 (NEI 07-08) 12.3.4 Appendix 12AA (NEI 07-03)
-----	--	---

Enclosure  
TVA letter dated August 29, 2008  
RAI Responses

8.4	Direct-Reading and Indirect-Reading Pocket Dosimeters (Rev. 0, February 1973)	Appendix 12AA (NEI 07-03)
8.5	Criticality and Other Interior Evacuation Signals (Rev. 1, March 1981)	Appendix 12AA (NEI 07-03)
8.6	Standard Test Procedure for Geiger-Muller Counters (Rev. 0, May 1973)	Appendix 12AA (NEI 07-03)
8.7	Instructions for Recording and Reporting Occupational Radiation Exposure Data (Rev. 2, November 2005)	12.1 (NEI 07-08) Appendix 12AA (NEI 07-03)
8.8	Information Relevant to Ensuring That Occupational Radiation Exposures at Nuclear Power Stations Will Be as Low as Is Reasonably Achievable (Rev. 3, June 1978)	12.1 (NEI 07-08) 12.3.4 Appendix 12AA Appendix 12AA (NEI 07-03) 13.1.2.1.1 13.1.2.1.1.5
8.9	Acceptable Concepts, Models, Equations, and Assumptions for a Bioassay Program (Rev. 1, July 1993)	12.1 (NEI 07-08) Appendix 12AA (NEI 07-03)
8.10	Operating Philosophy for Maintaining Occupational Radiation Exposures as Low as Is Reasonably Achievable (Rev. 1-R, May 1977)	12.1 (NEI 07-08) 12.3.4 Appendix 12AA Appendix 12AA (NEI 07-03) 13.1.2.1.1 13.1.2.1.1.5
8.13	Instruction Concerning Prenatal Radiation Exposure (Rev. 3, June 1999)	12.1 (NEI 07-08) Appendix 12AA (NEI 07-03)
8.15	Acceptable Programs for Respiratory Protection (Rev. 1, October 1999)	12.1 (NEI 07-08) Appendix 12AA (NEI 07-03)
8.27	Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power Plants (Rev. 0, March 1981)	12.1 (NEI 07-08) Appendix 12AA (NEI 07-03)
8.28	Audible-Alarm Dosimeters (Rev. 0, August 1981)	12.1 (NEI 07-08) Appendix 12AA (NEI 07-03)
8.29	Instruction Concerning Risks from Occupational Radiation Exposure (Rev. 1, February 1996)	12.1 (NEI 07-08) Appendix 12AA (NEI 07-03)
8.34	Monitoring Criteria and Methods To Calculate Occupational Radiation Doses (Rev. 0, July 1992)	12.1 (NEI 07-08) Appendix 12AA (NEI 07-03)
8.35	Planned Special Exposures (Rev. 0, June 1992)	12.1 (NEI 07-08) Appendix 12AA (NEI 07-03)
8.36	Radiation Dose to the Embryo/Fetus (Rev. 0, July 1992)	12.1 (NEI 07-08) Appendix 12AA (NEI 07-03)

Enclosure  
TVA letter dated August 29, 2008  
RAI Responses

8.38	Control of Access to High and Very High Radiation Areas of Nuclear Plants (Rev. 1, May 2006)	12.1 (NEI 07-08) Appendix 12AA (NEI 07-03)
------	--	---

a. NEI templates are incorporated by reference. See Table 1.6-201.

**ASSOCIATED ATTACHMENTS/ENCLOSURES:**

None