

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401
400 Chestnut Street Tower II

June 16, 1983

Director of Nuclear Reactor Regulation
Attention: Ms. E. Adensam, Chief
Licensing Branch No. 4
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Ms. Adensam:

In the Matter of the Application of) Docket Nos. 50-390
Tennessee Valley Authority) 50-391

In my letter to you dated November 9, 1982, TVA provided schedules for responding to open and confirmatory items in the Watts Bar Nuclear Plant Safety Evaluation Report. Enclosed is an updated listing showing the status of each of these items.

If you have any questions concerning this matter, please get in touch with D. P. Ormsby at FTS 858-2682.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

L. M. Mills
L. M. Mills, Manager
Nuclear Licensing

Sworn to and subscribed before me
this _____ day of _____ 1983

Notary Public
My Commission Expires _____

Enclosure

cc: U.S. Nuclear Regulatory Commission (Enclosure)
Region II
Attn: Mr. James P. O'Reilly, Regional Administrator
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30303

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ENCLOSURE

WATTS BAR NUCLEAR PLANT UNITS 1 AND 2

TABLE 1
(Sheet 1)OUTSTANDING ISSUES

| <u>SER Item</u> | <u>Issue</u> | <u>Status</u> | <u>Schedule</u> |
|-----------------|--|---|-----------------------|
| 1. | Potential for liquefaction beneath ERW pipelines and Class 1E electrical conduit | An NRC-TVA meeting was held on 5/20/83. Based on this meeting and subsequent telephone calls, TVA is to provide field remedial action to be completed by fuel load of unit 1. TVA is evaluating the impact of 6-percent settlement. TVA-NRC to discuss this by 6/20/83. NRC to formally document their position that the top-of-ground motion should be .40g. | 6/20/83 |
| 2. | Buckling loads on Class 2 and 3 supports | TVA submittal of 1/29/82. Telephone conference call of 4/1/82 requested clarification. This information provided on 11/10/82 and 11/18/82. | Not applicable |
| 3. | Preservice and inservice pump and valve test program | The SER specifies that the NRC is reviewing TVA's submittal of 10/31/81. No TVA action is required. | Not applicable |
| 4. | Seismic and environmental qualification of equipment | TVA responses to NRC SQRT audit items was submitted 9/23/83 and 12/1/82. During a 3/11/83 telephone conference call, the NRC requested additional clarification. This confirmation was submitted on 6/10/83. | Not applicable |
| | | TVA is preparing a response for environmental qualification of equipment. | 9/1/83 |
| 5. | Preservice and inservice inspection program | Responses to NRC questions Q121.20 and Q121.22 were provided by letter dated 7/30/82. Responses to NRC questions Q121.21 and Q121.23 were provided by letter dated 9/29/82. TVA commitment to provide a listing of welds included in the response to Q121.23. This information provided 4/18/83. TVA to provide a total list before commercial operation. | Commercial Operations |
| 6. | Pressure-temperature limits for unit 2 | The requested pressure-temperature limit curves for unit 2 are included in Amendment 48 of the Final Safety Analysis Report. | Not applicable |
| 7. | Model D-3 steam generator preheater tube degradation | Information provided on 5/27/83. Completion of modifications is expected before fuel load. | Fuel load |

ENCLOSURE

WALTS BAR NUCLEAR PLANT UNITS 1 AND 2

TABLE 1
(Sheet 2)

OUTSTANDING ISSUES

| <u>SER Item</u> | <u>Issue</u> | <u>Status</u> | <u>Schedule</u> |
|-----------------|---|---|-----------------|
| 8. | BTP-CSB 6-4 and containment isolation dependability | NRC letter of 5/14/82 requested additional information concerning compliance with items 1.b, 1.g, 5.a, and 5.c of BTP CSB 6-4. TVA response submitted 4/26/83. NRC letter of 10/5/82 requested information concerning containment purge and vent valve operability. TVA response in preparation. | |

ENCLOSURE

WATTS BAR NUCLEAR PLANT UNITS 1 AND 2

TABLE 1
(Sheet 3)OUTSTANDING ISSUES

| <u>SER Item</u> | <u>Issue</u> | <u>Status</u> | <u>Schedule</u> |
|-----------------|--|---|-----------------|
| 9. | Hydrogen Analysis Review | By letter dated 9/21/82, TVA specified that the revision to FSAR Section 15.4.1 did not effect the response to NRC question Q22.70. By telephone on 10/6/82, TVA was asked to rereview this position. TVA has reevaluated the impact on this question and confirmed that the response is not effected. NRC project manager informed 10/29/82. | Not applicable |
| 10. | Safety valve sizing analysis | TVA response concerning applicability of WCAP-7769 submitted 4/18/83. | Not applicable |
| 11. | Compliance of proposed design change to the offsite power system piping and components | The FSAR was revised by Amendment 48 to reflect the design changes for automatic fast transfer. | Not applicable |
| 12. | Fire Protection Program | A review of SER Section 9.5.1 indicates that open items consist of emergency lighting, hydrogen line routing, and asbestos insulated transformers. These items were addressed in letters dated 7/9/82, 8/28/81, and 6/3/82, respectively. | 8/5/83 |
| 13. | Quality classification of diesel generator auxiliary system piping and components | NRC letter of 3/25/83 requested information demonstrating compliance with Appendix R. During a telephone conference call, the NRC requested additional information. | Not applicable |
| 14. | Diesel generator auxiliary systems design deficiencies | TVA's response to the NRC position specified in the SER concerning qualification of diesel generator piping and components was submitted 11/30/82. | Not applicable |
| 15. | Physical Security Plan | TVA responses to NRC concerns about diesel generator cooling and lube oil systems submitted 6/9/83. | Not applicable |
| | | TVA submitted a revision to the Physical Security Plan to the NRC on 2/3/83 showing use of the power block concept. NRC comments were provided on 5/24/83. TVA responses in preparation. | 8/5/83 |

ENCLOSURE

WATTS BAR NUCLEAR PLANT UNITS 1 AND 2

TABLE 1
(Sheet 4)

OUTSTANDING ISSUES

| <u>SER Item</u> | <u>Issue</u> | <u>Status</u> | <u>Schedule</u> |
|-----------------|----------------------|--|-----------------|
| 16. | Boron Dilution Event | IWA's response to NRC concerns about single failure provided by letter dated 9/21/82. | Not applicable |
| 17. | Q-List | The requested information was provided by letter dated 10/25/82. This is also included in FSAR Amendment 48. | Not applicable |

WATTS BAR NUCLEAR PLANT UNITS 1 AND 2

TABLE 2
(Sheet 1)CONFIRMATORY ITEMS

| <u>SER Item</u> | <u>Issue</u> | <u>Status</u> | <u>Schedule</u> |
|-----------------|--|---|---|
| 1. | Design basis groundwater level for the ER CW pipe-line | The requested information concerning ground water level was provided to the NRC by letter dated 6/24/82. | Not applicable |
| 2. | Material and geometric damping effect on SSE analysis | As a result of a TVA-NRC meeting 9/22-24/82 TVA provided the card deck used for computer runs to determine the 4 time histories for rock motion on 11/30/82. | Not applicable |
| 3. | Analysis of sheet-pile walls | NRC letter of 6/1/82 requested additional information concerning sheetpile walls and soil properties for buried pipe. TVA responses in preparation. | 8/5/83 |
| 4. | Design differential settlement of piping and electrical components between Class 1E structures | During a TVA-NRC meeting 9/22-24/82 the NRC stated this issue was satisfactorily resolved. | Not applicable |
| 5. | Upgrading ER CW system to seismic Category I | TVA will complete necessary upgrading for ER CW systems which service air cooling units and chiller packages. | Date will be provided in final 50.55(e) report to NRC-OIE |
| 6. | Seismic classification of structures, systems, and components important to safety | TVA is in the process of revising FSAR Table 3.2-2 to properly show seismic classification. This information will be included in FSAR Amendment 49. | 9/15/83 |
| 7. | Tornado missile protection of diesel generator exhaust | In an 11/24/82 submittal, TVA agreed to modify the diesel generator exhaust stack to ensure continued d.g. operation following impact of a tornado missile. | Fuel load |
| 8. | Steel containment building buckling research program | TVA to evaluate the WBN containment based on the results of the Los Alamos studies. Because Los Alamos has not released their final results, TVA cannot perform the evaluation. TVA does not expect to be able to complete this action until after fuel load. | 3/1/84 |
| 9. | Pipe support base plate flexibility and its effects on anchor bolt loads | The SER specifies that the NRC is reviewing TVA's response to Bulletin 79-02. No TVA action is necessary. | Not applicable |
| 10. | Thermal performance analysis | The thermal performance analysis has been | Not applicable |

WATTS BAR NUCLEAR PLANT UNITS 1 AND 2

TABLE 2
(Sheet 2)CONFIRMATORY ITEMS

| <u>SER Item</u> | <u>Issue</u> | <u>Status</u> | <u>Schedule</u> |
|-----------------|---|--|-----------------------------------|
| | | performed using PAD-3.3. | |
| 11. | Cladding collapse | Westinghouse calculations for cladding collapse utilized a model consistent with WCAP-R377. | Not applicable |
| 12. | Fuel rod bow evaluation | TVA's position on margins used to offset reduction in DNB due to fuel rod bowing were provided to the NRC by letter dated 9/15/82. | Not applicable |
| 13. | Loose parts monitoring system | TVA addressed specific NRC questions by letter to NRC on 11/10/82. | Not applicable |
| 14. | Installation of residual heat removal system flow alarm | An RHR flow alarm will be installed to alert the operator to initiate alternate cooling modes in the event of loss of RHR. | Fuel load |
| 15. | Natural circulation test | As specified in startup test SU-3.9 (Table 14.2-2A) a natural circulation test will be conducted for operator training as part of the startup test program. | After receipt of OL |
| | | TVA will evaluate the Diablo Canyon tests and assess the applicability to Watts Bar. | After completion of Diablo Canyon |
| 16. | Dump valve testing | The SER states that TVA is to confirm the feasibility of manual action to correct for single failure. TVA did not commit to verify manual actuation of atmospheric dump valves. TVA's position is included in part 1 of the response to NRC question 212.93 as provided by Amendment 44 of the FSAR. | Not applicable |
| 17. | Protection against damage to containment from external pressure | TVA submitted information to the NRC on effects of inadvertent containment spray and air return fan actuation on containment pressure by letter dated 6/4/82. | Not applicable |
| 18. | Designation of containment isolation valves for main and auxiliary feedwater bypass lines | TVA will install a safety grade isolation valve in the safety grade portion of each chemical feedline for each of four main feedwater, four feedwater bypass, and two auxiliary feedwater lines. | Fuel load |

WATTS BAR NUCLEAR PLANT UNITS 1 AND 2

TABLE 2
(Sheet 3)CONFIRMATORY ITEMS

| <u>SER Item</u> | <u>Issue</u> | <u>Status</u> | <u>Schedule</u> |
|-----------------|---|---|--------------------------|
| 19. | Compliance with GDC 51 | TVA has been informally requested to revise its submittal of 12/17/82 concerning GDC-51. | 8/5/83 |
| 20. | Insulation survey | The SER specifies TVA is to provide the results of an insulation survey in the vicinity of the sump before startup following the first refueling outage. This report submitted 11/23/82. | Not applicable |
| 21. | Safety system setpoint methodology | TVA provided a list of safety related devices and a summary of the data used in the setpoint analysis to assist the NRC in a review of the Technical Specifications. on 3/18/83. | Not applicable |
| 22. | Steam generator water level reference leg | As specified by TVA letter dated 6/21/82, it is TVA's intent to insulate the steam generator reference leg to minimize dependence problems. | Fuel load |
| 23. | Containment sump level measurement | The FSAR will be revised in Amendment 49 to reflect design changes to protect sump level sensors from debris. TVA is in the process of implementing these design changes. Completion by fuel load. | 9/16/83 Fuel load |

WATTS BAR NUCLEAR PLANT UNITS 1 AND 2

TABLE 2
(Sheet 4)CONFIRMATORY ITEMS

| <u>SER Item</u> | <u>Issue</u> | <u>Status</u> | <u>Schedule</u> |
|-----------------|--|--|---------------------------------|
| 24. | IE Bulletin 80-06 | Modifications are being made to ensure valves remain in the emergency mode after engineered safety features reset. The FSAR will be revised in Amendment 49 to show design changes to valves which ensure emergency mode after ESF reset. | Fuel load 9/15/83 |
| 25. | Overpressure protection during low temperature operation | Figure 7.6-5 of the FSAR was revised by Amendment 45 to show the switches for manual arming. TVA will install switches for manual arming on the main control panel. | Not applicable Fuel load |
| | | Manual arming will be included in the operating procedures. | Fuel load |
| 26. | Availability of offsite circuits | The FSAR was revised in Amendment 48 to describe the Watts Bar Hydro Plant switchyard. | Not applicable |
| 27. | Nonsafety loads powered from the Class 1E ac distribution system | The FSAR was revised in Amendment 48 to describe the design for each of the preferred offsite circuits. | Not applicable |
| 28. | Low and/or degraded grid voltage condition | The analysis will be verified before full power operation. | Before full power |
| 29. | Diesel generator reliability qualification testing | Copies of the Sequoyah diesel generator test results were provided to the NRC during a Watts Bar site visit on 7/26/82. | Not applicable |
| 30. | Diesel generator battery system | The FSAR was revised in Amendment 48 to show compliance of the 125V battery system. | Not applicable |
| 31. | Thermal overload protective bypass | NRC verification of the design of bypass of thermal-overload protection devices for motors and valves was conducted during the Watts Bar site visit on 7/26/82. | Not applicable |

WATTS BAR NUCLEAR PLANT UNITS 1 AND 2

TABLE 2
(Sheet 5)CONFIRMATORY ITEMS

| <u>SER Item</u> | <u>Issue</u> | <u>Status</u> | <u>Schedule</u> |
|-----------------|---|---|---------------------|
| 32. | Sharing of dc and ac distribution systems and power supplied between units 1 and 2 | The FSAR was revised in Amendment 48 to reflect requirements of the shared safety system. | Not applicable |
| 33. | Sharing of raceways between units | Confirmation of shared raceway design conformance with GDC 5 was conducted during the Watts Bar site visit on 7/26/82. | Not applicable |
| 34. | Testing Class 1E power systems | The FSAR was revised in Amendment 48 to describe testing of one of two class 1E power systems versus one of four systems. | Not applicable |
| 35. | Evaluation of penetrations capability to withstand failure of overcurrent protection device | Information concerning the capability of penetrations to withstand the total range of time current characteristics was submitted on 6/9/83 and 6/10/83. | Not applicable |
| 36. | Missile protection for diesel generator vent line | TVA will provide missile protection for the diesel generator vents. | Fuel load |
| 37. | Component booster pump relocation | TVA has completed relocating the component booster pumps above the maximum probable flood level. | Not applicable |
| 38. | Electrical penetrations documentation | SER pages 9-34 specifies that the NRC is comparing the TVA test methods for electrical fire barrier penetrations against ASTM E-119. No TVA action is required. | Not applicable |
| 39. | Compliance with NUREG-0660 | TVA will install a heavy duty turbo charger gear assembly to ensure operability under no-load or light-load conditions and full rated speed. | Before second cycle |
| 40. | No-load, low-load testing operations for diesel generators | The operating procedures provide for minimizing no-load and light-load operation. | Not applicable |

WATTS BAR NUCLEAR PLANT UNITS 1 AND 2

TABLE 2
(Sheet 6)

CONFIRMATORY ITEMS

| <u>SER Item</u> | <u>Issue</u> | <u>Status</u> | <u>Schedule</u> |
|-----------------|----------------------|---|-----------------|
| 41. | Initial test program | A revised FSAR Table 14.2-1 on preoperational testing was included in Amendment 43. | Not applicable |