

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

October 2, 2008

Mr. Charles G. Pardee President and Chief Nuclear Officer Exelon Nuclear 4300 Winfield Road Warrenville, IL 60555

SUBJECT:

PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3: REQUEST FOR ADDITIONAL INFORMATION REGARDING RELIEF REQUEST I4R-47 ASSOCIATED WITH THE FOURTH INSERVICE INSPECTION INTERVAL

(TAC NOS. MD8304 AND MD8305)

Dear Mr. Pardee:

By letter to the Nuclear Regulatory Commission (NRC) dated February 29, 2008 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML080640587), Exelon Generation Company, LLC, submitted Relief Request No. I4R-47, related to the Fourth 10-Year Interval Inservice Inspection Program for the Peach Bottom Atomic Power Station, Units 2 and 3. The NRC staff has reviewed the request submitted by the licensee and has identified a need for additional information as set forth in the Enclosure.

The draft questions were sent to Tom Loomis, from your staff, to ensure that the questions were understandable, the regulatory basis for the questions was clear, and to determine if the information was previously docketed. On September 9, 2008, Mr. Loomis indicated that Exelon will submit a response by October 10, 2008. If you have any questions, please contact John Hughey at (301) 415-3204.

Sincerely.

John D. Hughey, Project Manager

Plant Licensing Branch I-2

Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket Nos. 50-277 and 50-278 Enclosure: Request for Additional

Information

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REQUEST FOR ADDITIONAL INFORMATION

REGARDING RELIEF REQUEST 14R-47 ASSOCIATED

WITH THE FOURTH INSERVICE INSPECTION INTERVAL

PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3

DOCKET NOS. 50-277 AND 50-278

By letter dated February 29, 2008, (Agencywide Documents Access and Management System (ADAMS) Accession No. ML080640587), Exelon Generation Company, LLC, submitted Relief Request (RR) No. I4R-47, related to the Fourth 10-Year Interval Inservice Inspection Program for the Peach Bottom Atomic Power Station, Units 2 and 3. In RR No. I4R-47, the licensee proposed an alternative to perform the Code-required end-of-interval system leakage test of the Control Rod Drive (CRD) pressure boundary during pressurization resulting from Scram Time testing of each CRD. Scram Time testing is routinely completed prior to achieving 40% reactor power.

The Nuclear Regulatory Commission (NRC) staff has reviewed the request for relief the licensee provided in the February 29, 2008, submittal. In order for the NRC staff to complete its evaluation, response to the following request for additional information (RAI) questions is requested.

- PAI-1) During Scram Time testing of the CRD, how long will the CV-2(3)-03A-13-127 valve for each CRD remain open? Specifically, how long will the CRD pressure boundary between the CV-2(3)-03A-13-127 valve and the HV-2(3)-03A-13112 valve remain pressurized at reactor coolant system pressure while a VT-2 visual examination of the pressure boundary is performed?
- RAI-2) Provide an estimate of the rate of pressure loss following pressurization of the pressure boundary between the CV-2(3)-03A-13-127 valve and the HV-2(3)-03A-13112 valve during Scram Time testing.

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/ra/

John D. Hughey, Project Manager Plant Licensing Branch I-2 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

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ADAMS Accession Number: ML082420589 *by memo dated August 21, 2008

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