

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401
400 Chestnut Street Tower II

May 23, 1984

WBRD-50-390/84-25
WBRD-50-391/84-23

34 MAY 29 AIO: 46

U.S. Nuclear Regulatory Commission
Region II
Attn: Mr. James P. O'Reilly, Regional Administrator
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

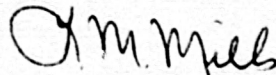
WATTS BAR NUCLEAR PLANT UNITS 1 AND 2 - CONDUIT SUPPORT DATA NOT PROVIDED FOR
SOME AREAS WITH CONDUIT INSTALLED - WBRD-50-390/84-25, WBRD-50-391/84-23 - FIRST
INTERIM REPORT

The subject deficiency was initially reported to NRC-OIE Inspector
P. E. Fredrickson on April 24, 1984 in accordance with 10 CFR 50.55(e) as NCR
WBN CEB 8407. Enclosed is our first interim report. We expect to submit our
next report on or about June 29, 1984.

If you have any questions, please get in touch with R. H. Shell at
FTS 858-2688.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



L. M. Mills, Manager
Nuclear Licensing

Enclosure

cc: Mr. Richard C. DeYoung, Director (Enclosure)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Records Center (Enclosure)
Institute of Nuclear Power Operations
1100 Circle 75 Parkway, Suite 1500
Atlanta, Georgia 30339

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ENCLOSURE

WATTS BAR NUCLEAR PLANT UNITS 1 AND 2
CONDUIT SUPPORT DATA NOT PROVIDED FOR SOME AREAS
WITH CONDUIT INSTALLED
NCR WBN CEB 8407
WBRD-50-390/84-25, WBRD-50-391/84-23
10 CFR 50.55(e)
FIRST INTERIM REPORT

Description of Deficiency

WB-DC-40-31.10, "Design Criteria for Seismically Qualifying Conduit Supports," provides allowable span lengths and support loads for supporting seismic category 1 conduit but does not include such information for conduit located in the Auxiliary Building above elevation 755.5', Reactor Building interior concrete structure above elevation 755.6', Reactor Shield Building above elevation 763.0', or Diesel Generator Building above elevation 742.0'. Also, WB-DC-40-31.10 does not include data for supporting conduit in the north steam valve room or attached to the steel containment vessel.

Apparently, when WB-DC-40-31.10 was developed, it was thought that conduit would not be installed in the above areas. However, conduit has been installed in some of the above areas using data which may not be conservative for those areas.

Interim Progress

WB-DC-40-31.10 has been revised to include span lengths and support loads for conduit in the Auxiliary Building above elevation 755.5', Reactor Shield Building above elevation 763.0', Reactor Building interior concrete structure above elevation 755.6', Diesel Generator Building above 742.0', and north steam valve room. It has been determined that there is no safety-related conduit rigidly attached to or supported from the steel containment vessel.

The allowable span lengths included in the revision to WB-DC-40-31.10 equal or exceed the allowable span lengths provided to TVA's Division of Construction (CONST) on drawing No. 47A056-1D, R0, for all sizes of uninsulated steel conduit and 5" nominal size uninsulated aluminum conduit. Therefore, no hardware modifications to correct span lengths are required. (Note: Insulated conduit is not addressed because the existing data for insulated conduit was made invalid by ECN 4742 (reference NCR WBN EEB 8408, WBRD-50-390,391/84-18). For aluminum conduit, only 5" nominal size is addressed because other sizes of aluminum conduit have not been used.)

Support design loads increased for some conduit sizes in some areas of the plant. Where the loads are increased more than 20 percent above the previous design loads, the support designs are being reviewed to see if the additional loads can be accommodated.

More information will be supplied in TVA's final report to the NRC.