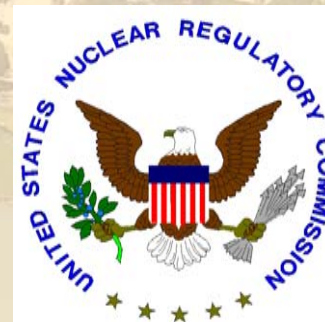


# ***Review of ORNL / NRC Studies Regarding Risk-Informing Regulations for Normal Operating Transients***

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Computational Sciences and Engineering Division  
Oak Ridge National Laboratory***

***Mark EricksonKirk / Eric Focht  
Office of Nuclear Regulatory Research  
United States Nuclear Regulatory Commission  
NRC / Nuclear Industry Representative meeting  
August 21, 2008 - Rockville, Maryland***



# Overview of Presentation

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- **Statement of Objectives**
- **Current Regulations for Normal Operating Transients**
- **Cool-down transients associated with reactor shutdown**
  - (a) **Currently bounding cool-down transients**
  - (b) **Potential regulatory relaxations that ensure safety**
  - (c) **Parameterized transients – more realistic conditions**
  - (d) **Inclusion of Inner Surface Breaking Flaws**
- **Heat-up transients associated with reactor start-up**
- **Conclusions and what's next**

# **Objective:** Derive a technical basis for a risk-informed revision of regulations for **Normal Operating Transients**

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- Consistent with revision to the Pressurized Thermal Shock (Hypothetical Accident Transients) Regulations

- PTS - proposed new acceptance criteria of  $1.0 \times 10^{-6}$  failed RPV per reactor operating year

Note: Risk acceptance criteria has not yet been agreed to by staff for routine heat up and cool-down transients. Factors to consider include (a) definition of “failure,” and (b) tolerable frequency

- Method – Perform PFM\* analyses for normal transients

**\*Probabilistic Fracture Mechanics Application of latest version of FAVOR**

***The P-T curve is currently derived using  
ASME Section XI – Appendix G***

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- (1) assumes a surface breaking flaw of depth equal to ¼ of the RPV wall**
- (2) includes a factor of 2 to account for sources of stress not included in the formulation**
- (3) maximum heat-up / cool-down rate of 100 °F /hr (56 °C /hr)**

**For a given cool-down transient  
the allowable pressure is determined by:**

$$P(t) = K_{lc}(t) - K_{IT}(t) / 2 C_p$$

**where:**

**$K_{lc}(t)$  is the ASME lower-bound crack initiation curve**

**$K_{IT}(t)$  is the thermally-induced stress intensity factor ( t / 4 flaw)**

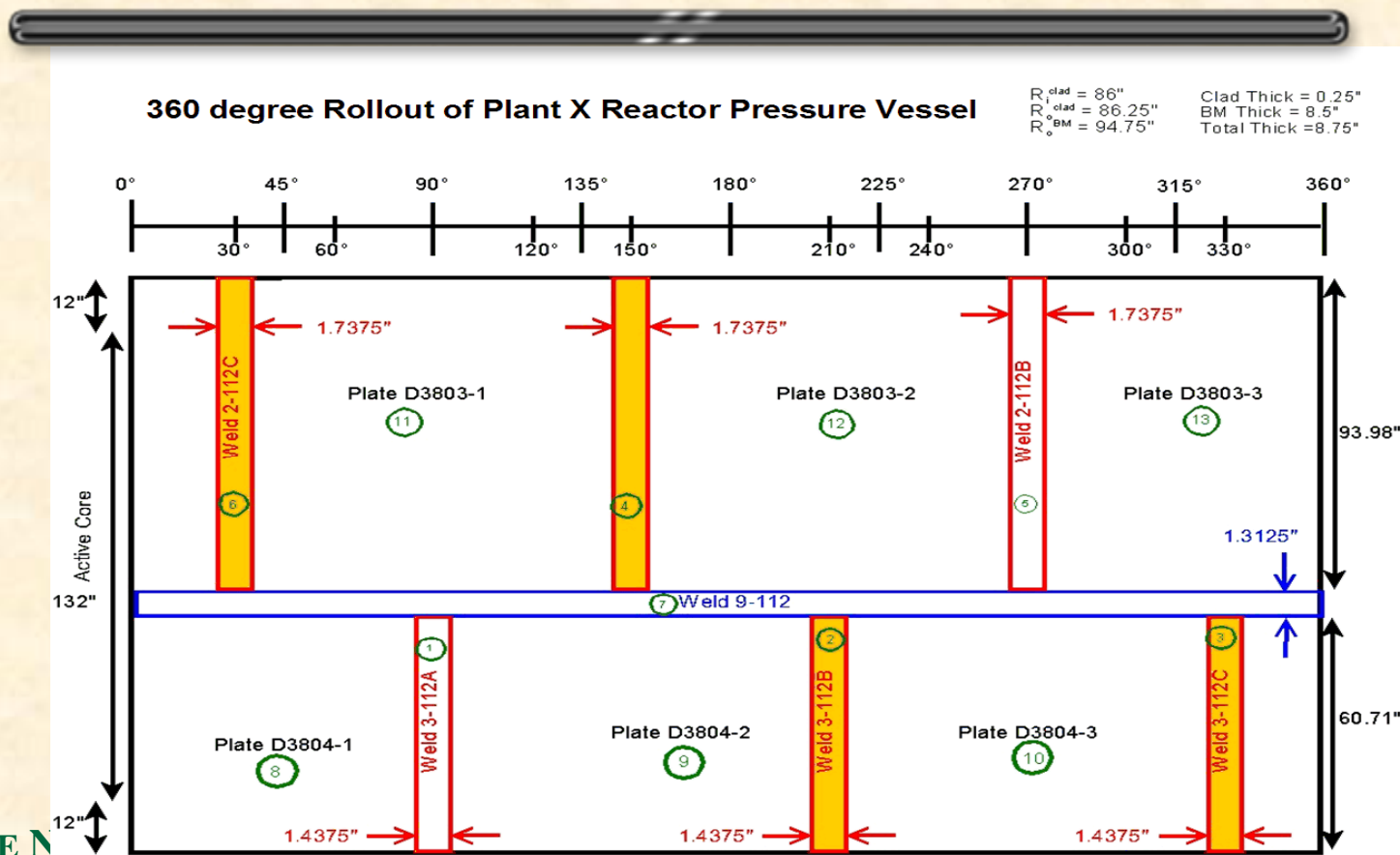
**$C_p$  = pressure-induced stress intensity factor produced by 1 ksi pressure loading**



All PFM analyses for normal cool-down transients thus far (previous and current publications) were performed on Plant X using neutron fluence maps that correspond to 60 EFPY

RPV discretized into over 60 K subregions to accommodate azimuthal and axial variations in neutron fluence

Each RPV in Monte Carlo PFM analysis postulated to have approximately 5700 embedded flaws uniformly distributed though the first 3/8 of the base metal



# Overview of Presentation

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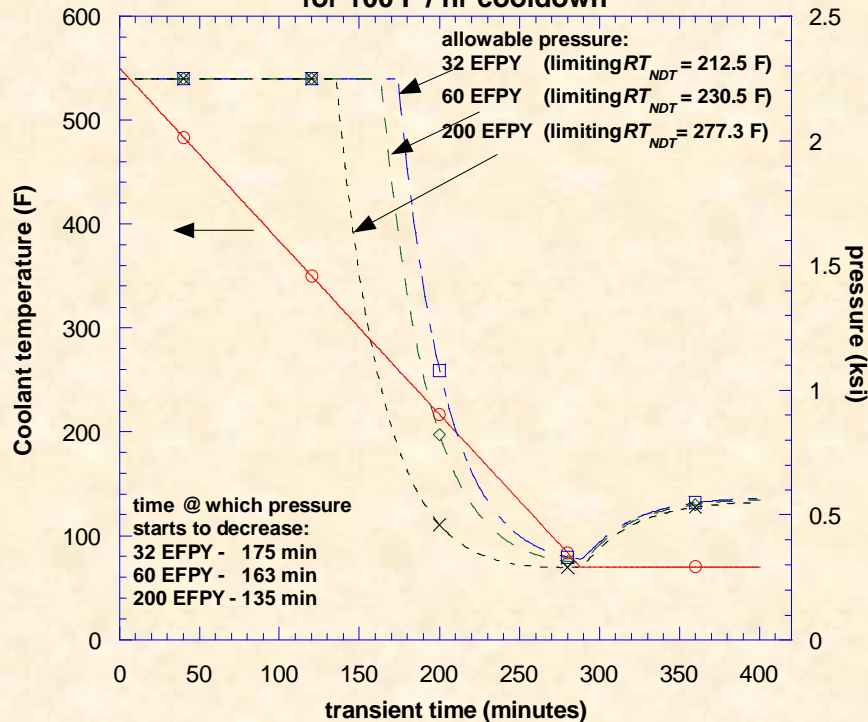
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- **Conclusions**

# Scoping PFM analysis results for bounding cool-down transients are in compliance with proposed new acceptance criteria (for PTS) of $1.0 \times 10^{-6}$ failed RPVs per reactor operating year for over 60 EFPY

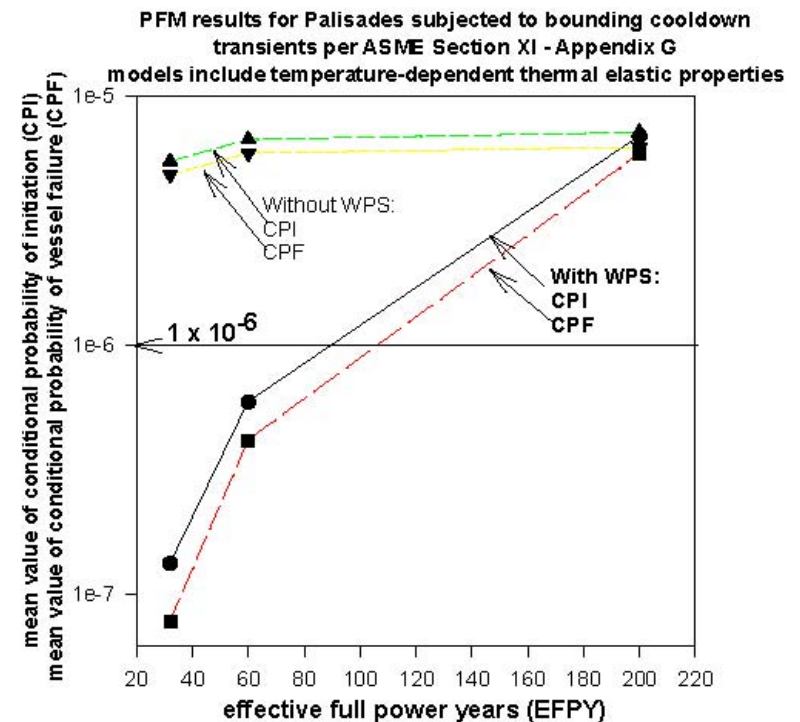
**(when model includes WPS)**

## Bounding cool-down transients for Palisades per Section – XI Appendix G

Shutdown transients for Palisades derived per Section XI - Appendix G for 100 F / hr cooldown

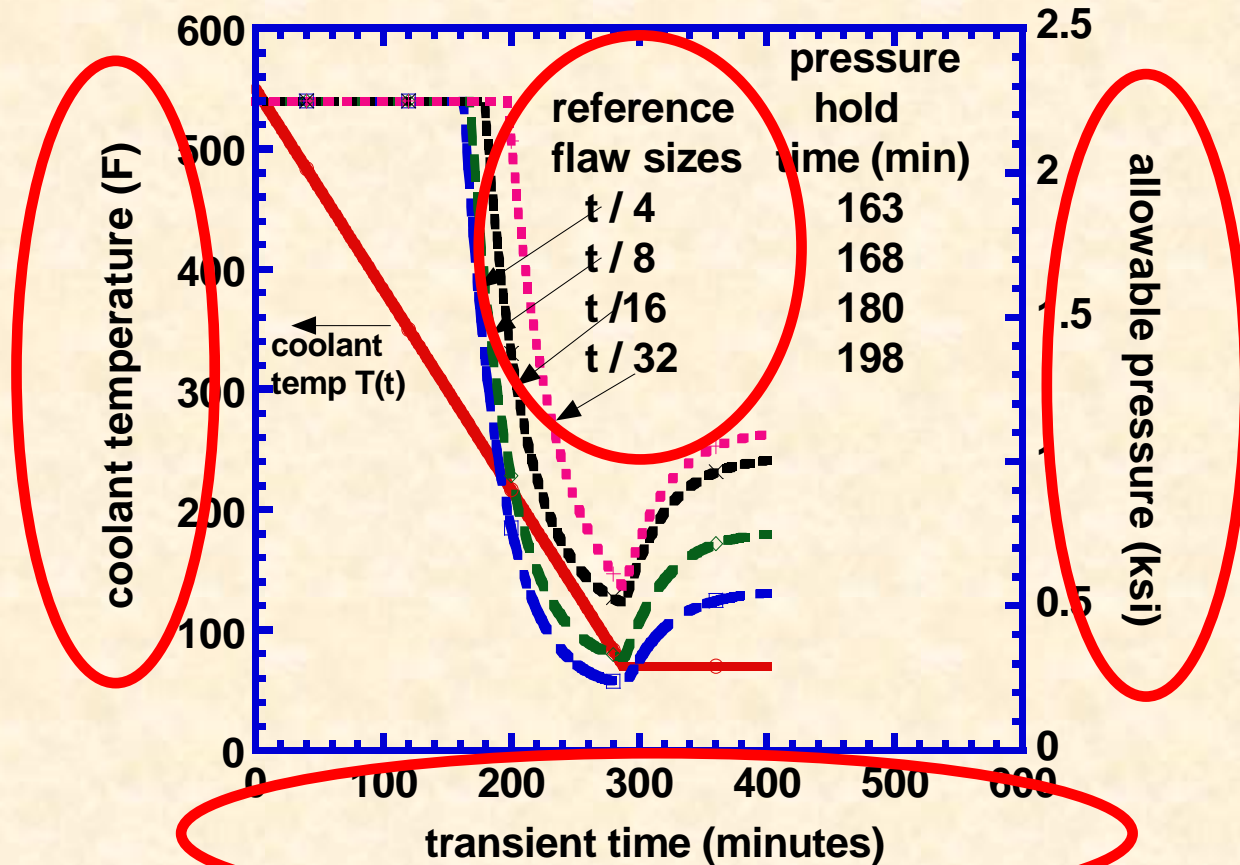


## CPI and CPF computed with and without WPS:



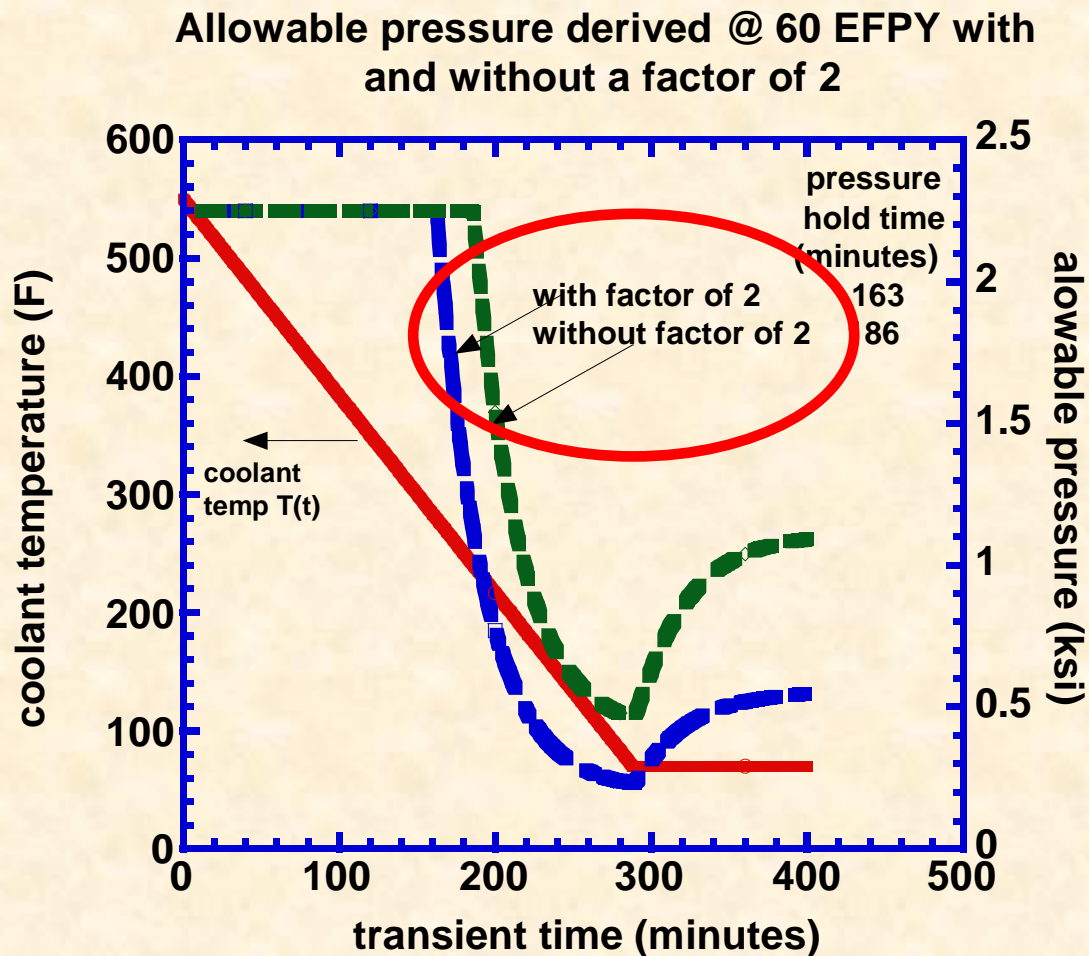
# PFM analyses were performed to determine impact of smaller reference flaw sizes in the derivation of allowable pressure

Allowable pressure derived @ 60 EFPY  
using smaller reference flaw sizes

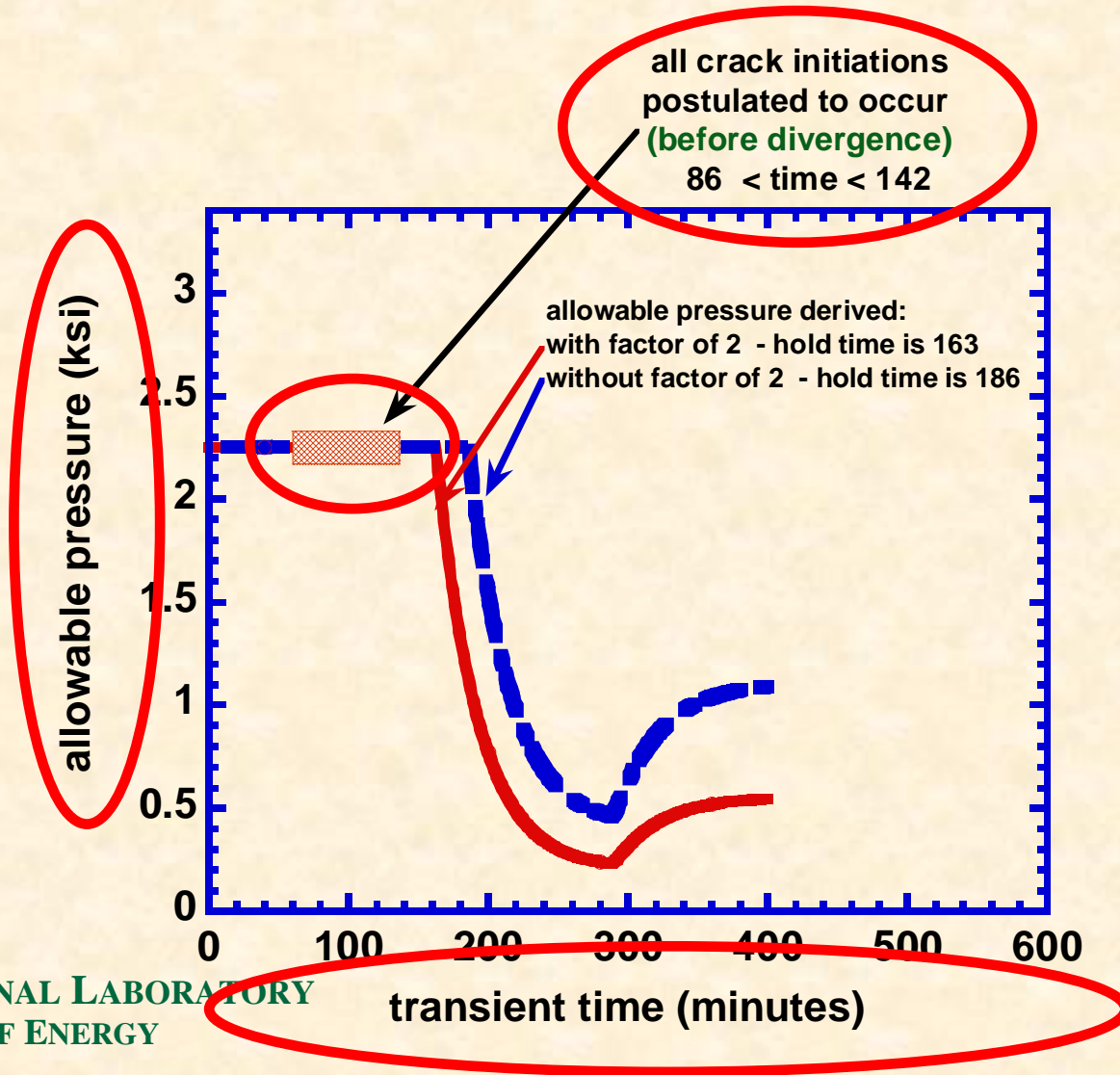




# PFM analyses were performed to determine impact of removing factor of 2 in the derivation of allowable pressure



**These Simplified Relaxations do not Impact PFM Solutions;  
all predicted crack initiations and failures occur before transients diverge**



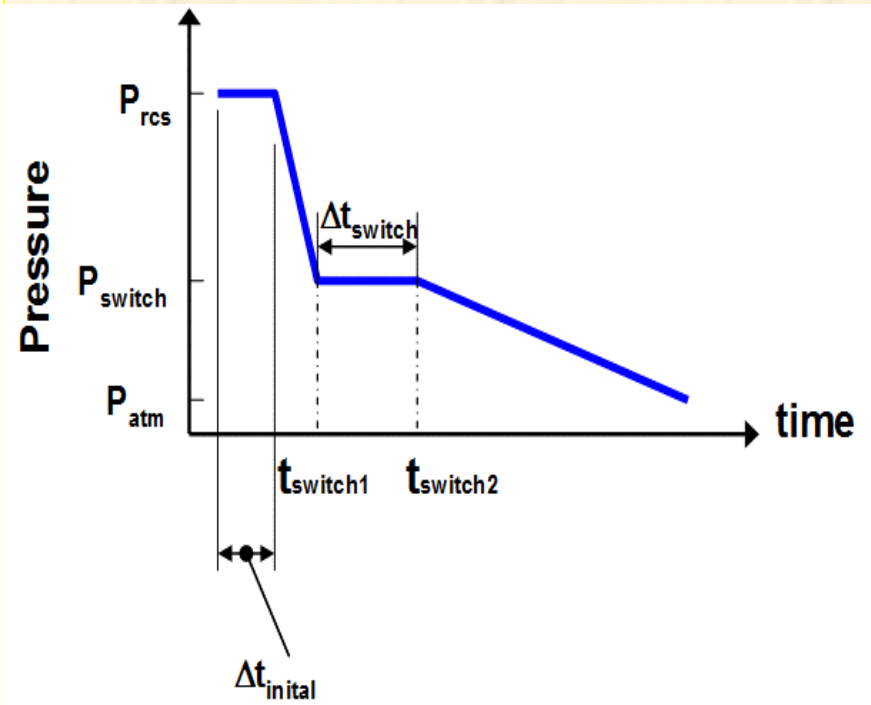
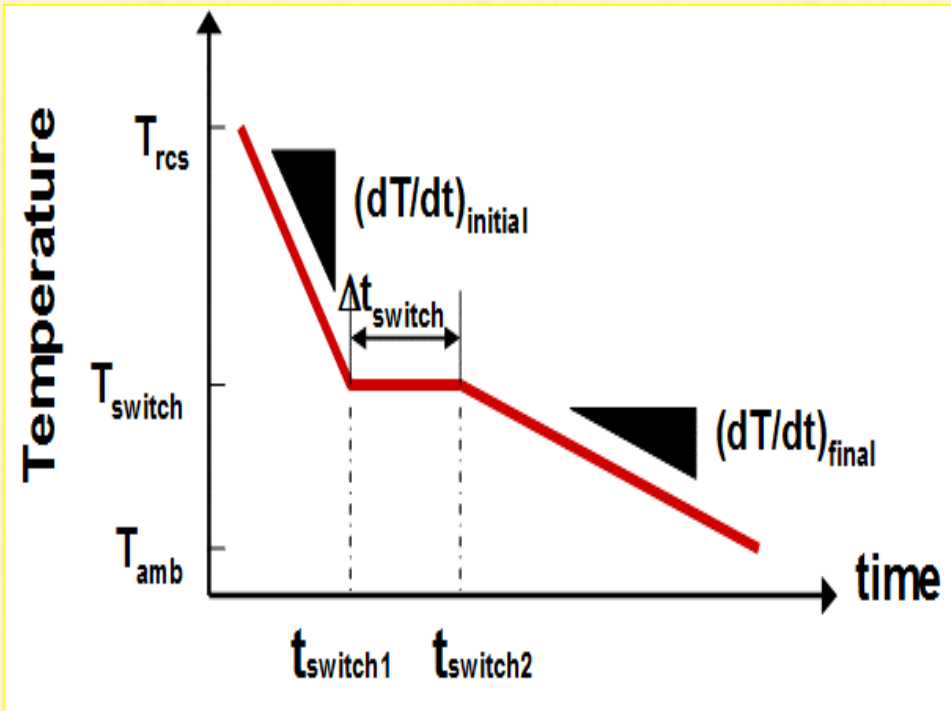
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# Cool-down transients associated with reactor shutdown were parameterized in terms of several variables

- cool-down rates:  $\{(dT/dt)_{\text{initial}}, (dT/dt)_{\text{final}}\}$ ,
- plateau temperature and pressure  $\{(T_{\text{switch}}, P_{\text{switch}})\}$ ,
- time duration pressure and temperature remain at plateau ( $\Delta t_{\text{switch}}$ ),
- pressure hold time  $\Delta t_{\text{initial}}$



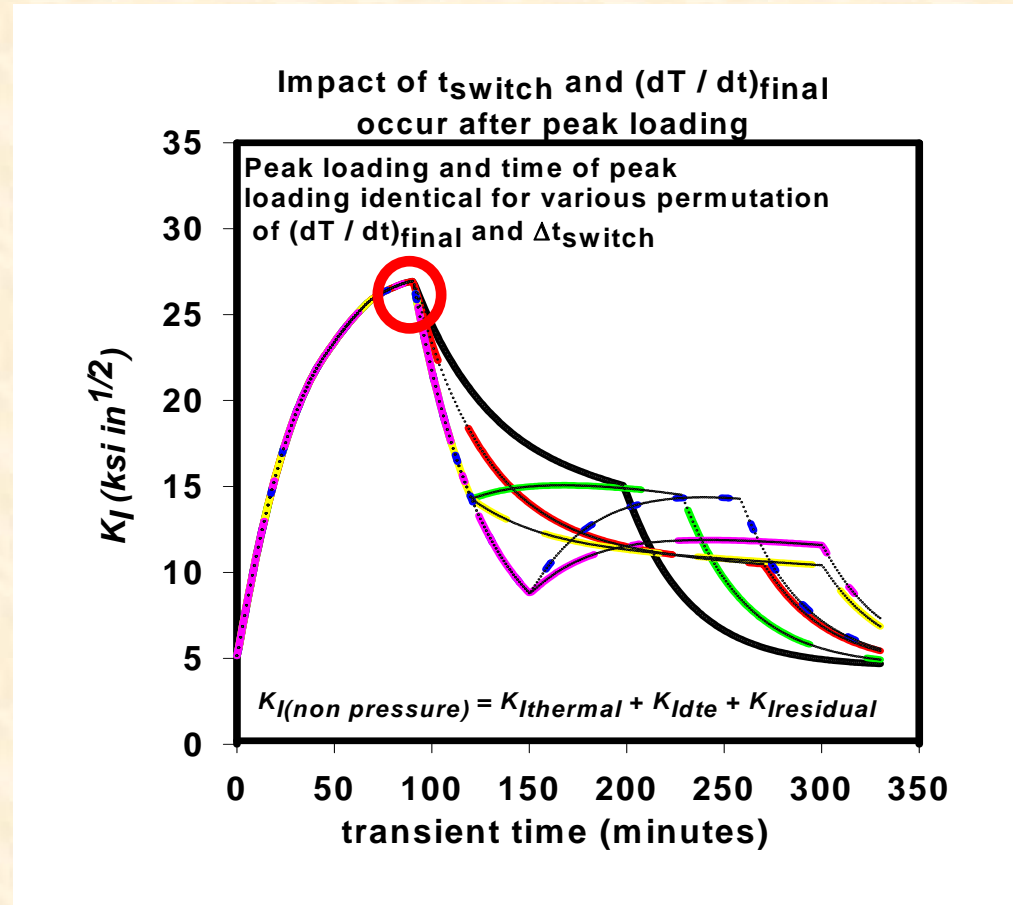
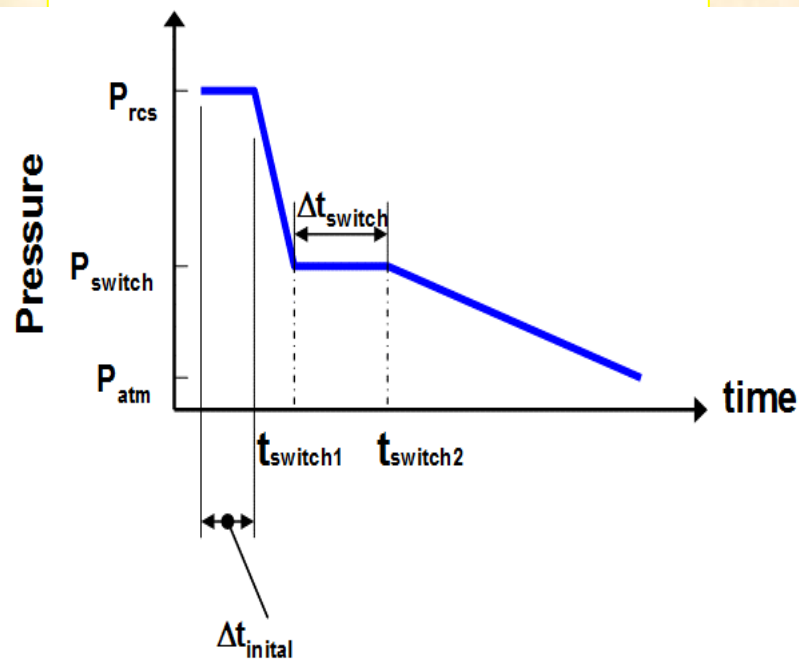
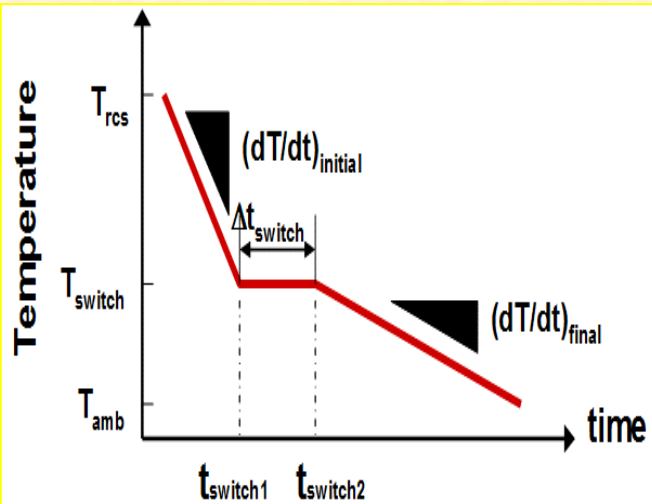
# Review of results and subsequent conclusions from previous PFM analyses / publications\* of parameterized cool-down transients associated with reactor shutdown

- \* Dickson, T.L. and EricksonKirk, M.T., **Scoping Analyses of Parameterized Cool- Down Transients Associated with Reactor Shutdown**, Paper number PVP2007-26865, Proceedings of 2007 ASME Pressure Vessels and Piping Division Conference July 22-26, 2007, San Antonio, Texas.
- Dickson, T.L. and EricksonKirk, M.T., **Review of Studies Regarding Risk-Informing Regulations for Normal Operating Transients**, Paper number G02/1, Proceedings of the 19th Structural Mechanics in Reactor Technology (SMiRT) Conference, August 2007, Toronto, Canada.
- Dickson, T.L. and EricksonKirk, M.T., ***The Inclusion of Inner Surface Breaking Flaws in Probabilistic Fracture Mechanics Analyses of Reactor Vessels Subjected to Planned Normal Cool-Down Transients***, Paper number PVP2008-61392, Proceedings of 2008 ASME Pressure Vessels and Piping Division Conference July, 2008, Chicago, Texas.



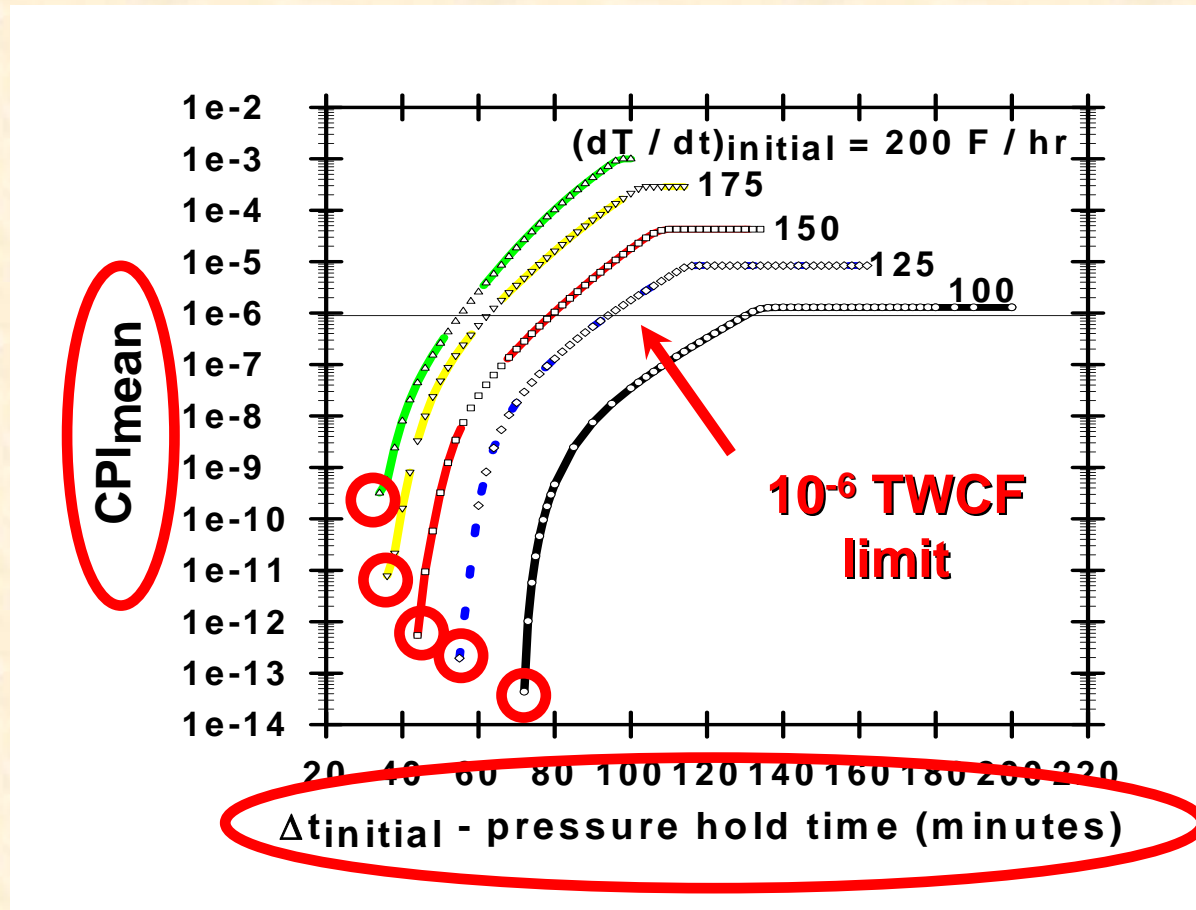
# PFM solutions are invariant with respect to $\Delta t_{\text{switch}}$ and $(dT / dt)_{\text{final}}$ when warm prestress is included in the model

effects of  $\Delta t_{\text{switch}}$  and  $(dT/dt)_{\text{final}}$  occur after the time of peak loading



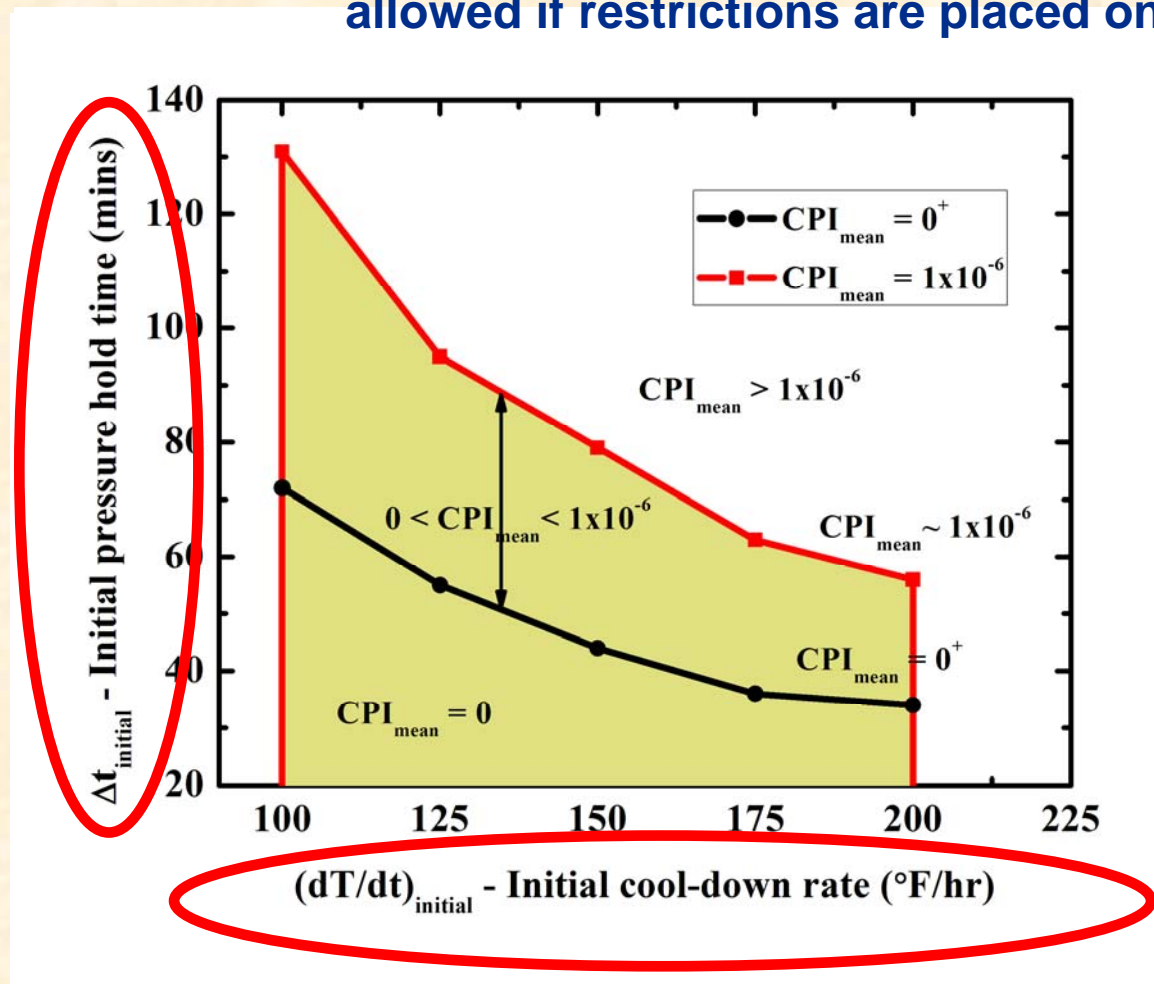
# PFM analyses were performed with FAVOR 06.1 for Plant X @ 60 EFPY for a range of cool-down scenarios

$(dT/dt)_{\text{initial}} = 100 \text{ to } 200 \text{ }^{\circ}\text{F (56 to 111 }^{\circ}\text{C) / hr}$  ;  $P_{\text{switch}} = 0.40 \text{ ksi (2.8 MPa)}$   
over a range of pressure hold time  $\Delta t_{\text{initial}}$



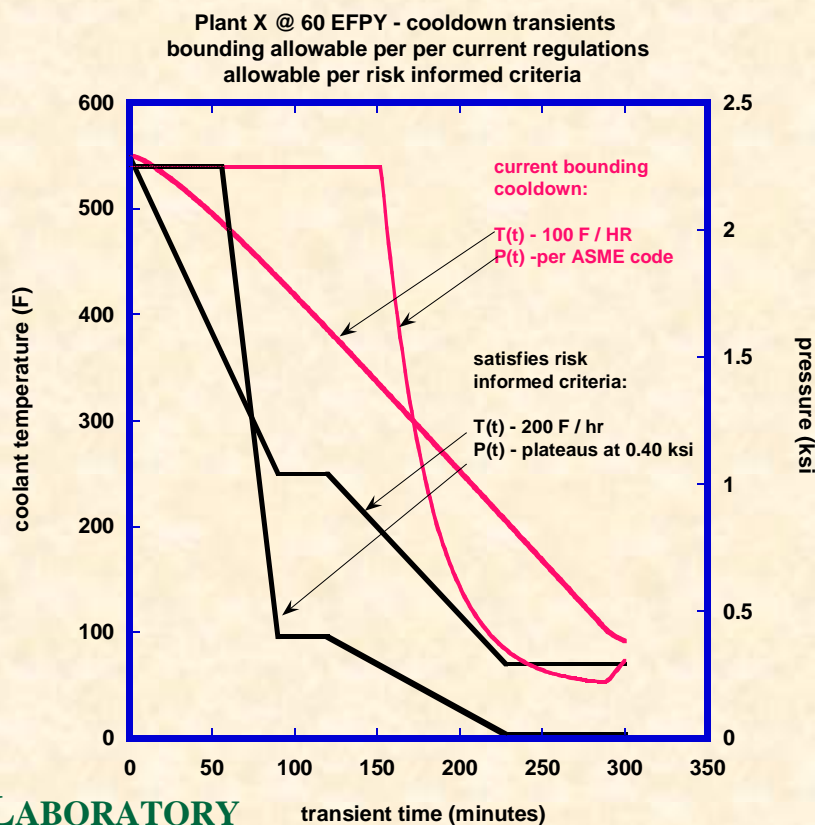
PFM results can be applied to determine a range of parameters (a window of operations) that a risk informed limit, such as  $(10^{-6})$

Initial cooling rates exceeding current limit of 100 °F / hr can be allowed if restrictions are placed on pressure hold time



**NOTE:**  $CPI_{\text{mean}}$  of 0 and  $10^{-6}$  used for purpose of illustration only.

# Risk informed approach results in considerable relaxation of maximum cool-down rate relative to bounding transient derived from ASME Section XI – Appendix G



# **How Does the Introduction of Inner-Surface Breaking into the PFM Model**

**Impact the results and subsequent conclusions  
of the previous PFM analyses of parameterized cool-  
down transients associated with reactor shutdown**

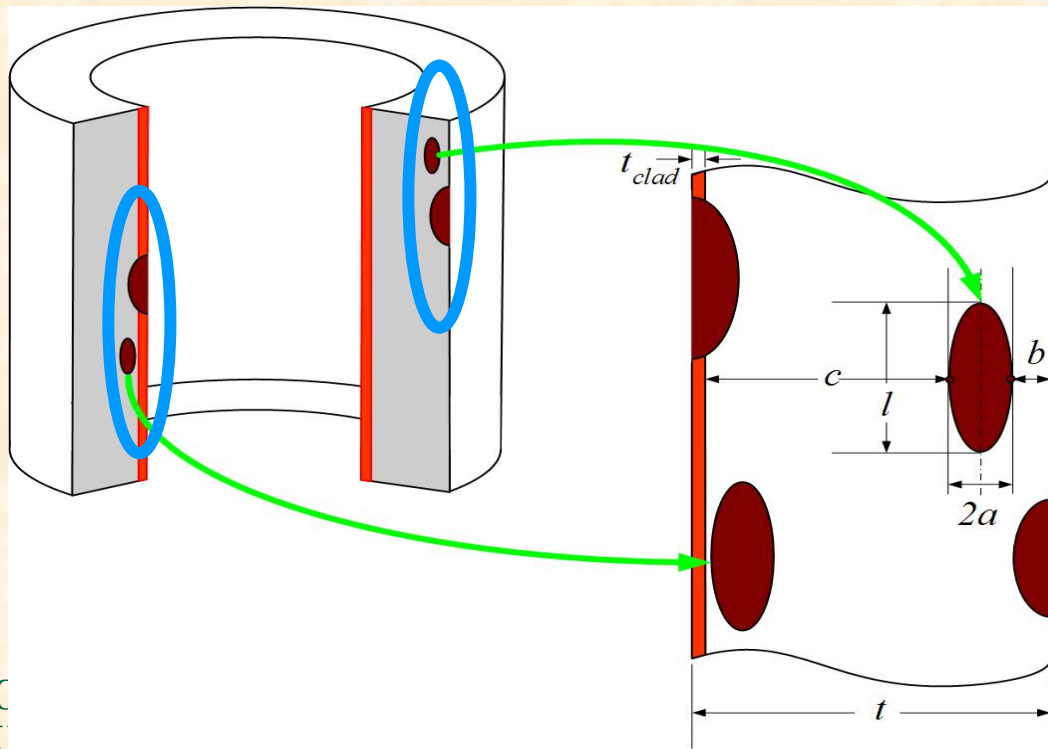
**Attempt to generalize previous results / conclusions**



Each RPV in the Monte Carlo analysis was postulated to have 2 circumferentially-oriented inner surface breaking flaws of specified depth

aspect ratio distribution: 67.45%, 20.76%, 3.96%, 7.83% have aspect ratios of 2, 6, 10, and continuous 360 degree flaws, respectively

Consistent with PNNL flaw distribution for RPVs with single layer cladding



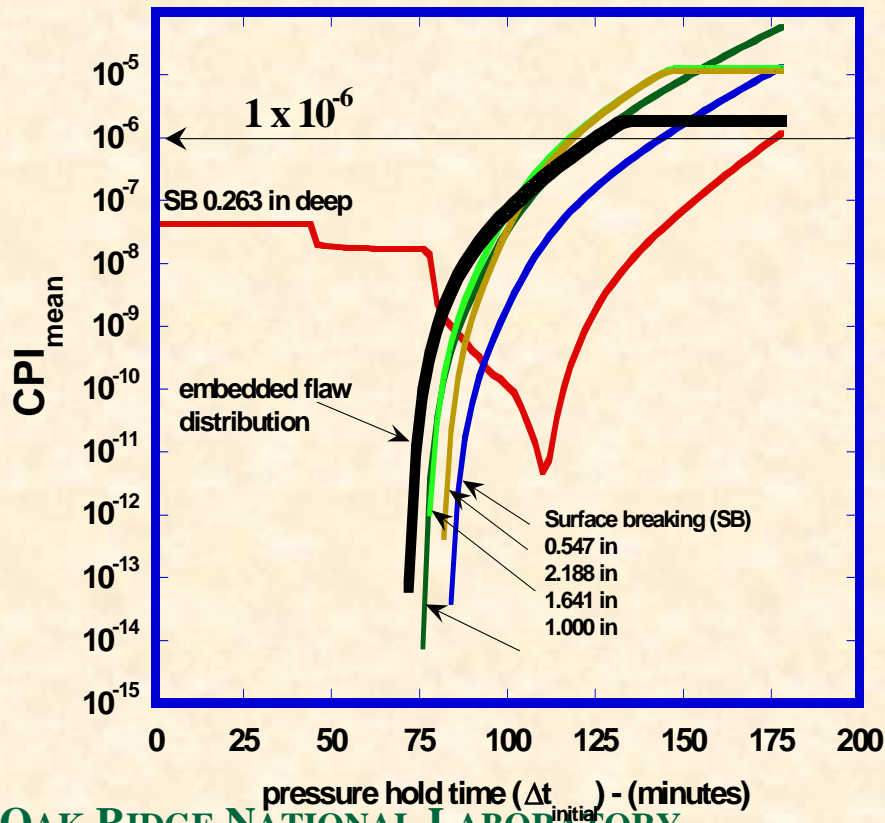
- ▶ External surface-breaking flaws and embedded flaws near outer surface are relevant to heat-up transient
- ▶ Internal surface-breaking flaws and embedded flaws near inner surface are relevant to cool-down transient

For  $(dT / dt)_{\text{initial}} = 100 \text{ F / hr}$ ; the inclusion of inner surface breaking flaws in the model (in addition to embedded flaws) reduces the allowable pressure hold time ( $\Delta t_{\text{initial}}$ ) from 125 minutes to 115 minutes



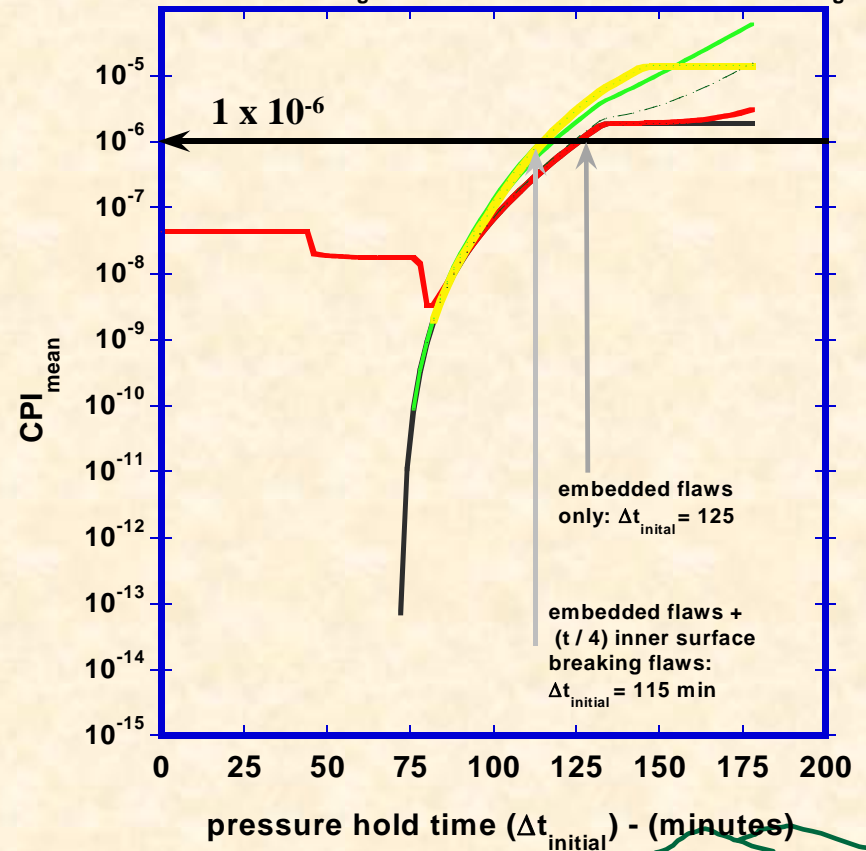
$$(dT / dt)_{\text{initial}} = 100^{\circ}\text{F / hr}$$

$\text{CPI}_{\text{mean}}$  for models containing only embedded flaws  
compared to models containing only surface breaking flaws



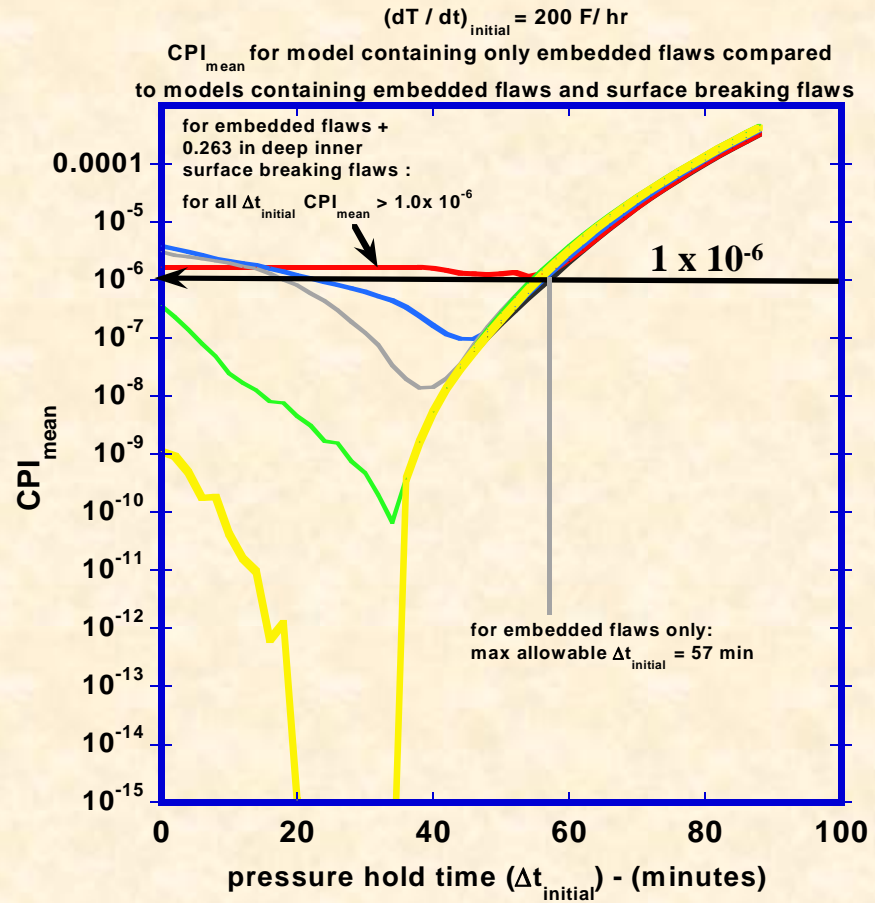
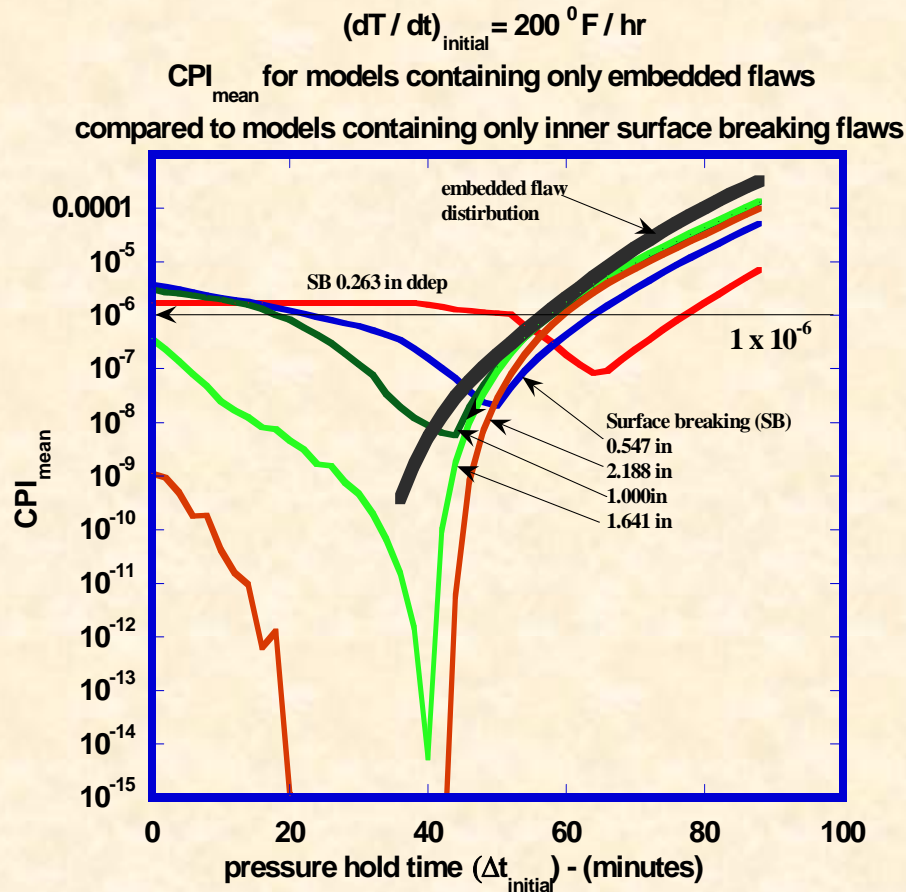
$$(dT / dt)_{\text{initial}} = 100 \text{ F / hr}$$

$\text{CPI}_{\text{mean}}$  for model containing only embedded flaws compared  
to models containing embedded flaws and inner surface breaking flaws



For  $(dT / dt)_{\text{initial}} = 200 \text{ F / hr}$ ; the inclusion of inner surface breaking flaws in the model (in addition to embedded flaws); there is NO value of  $\Delta t_{\text{initial}}$  that satisfies  $1 \times 10^{-6}$  limit;

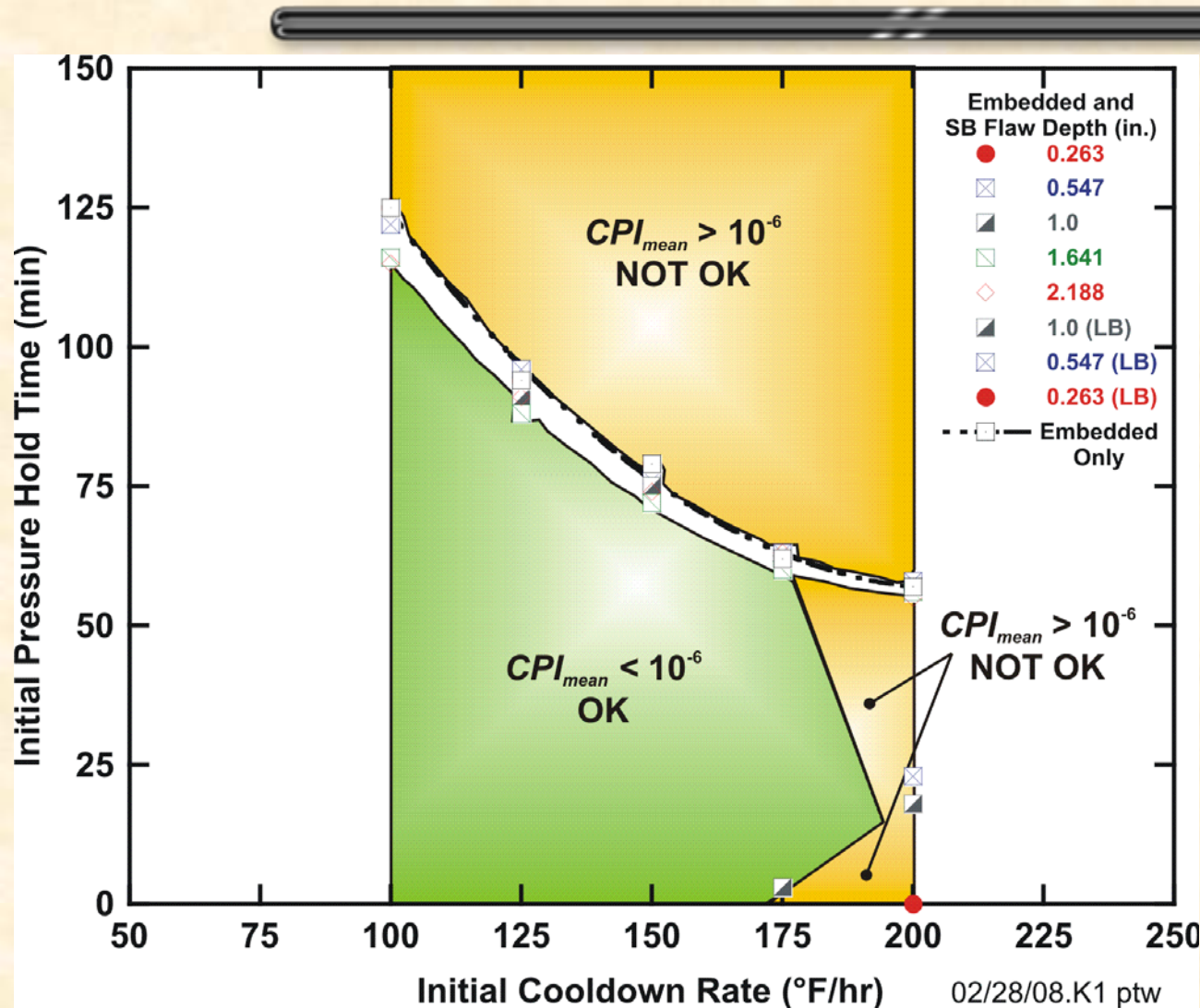
therefore; the operating window is closed for  $(dT / dt)_{\text{initial}} = 200 \text{ F / hr}$



The previously defined window within which planned reactor cool-down operations can be conducted while remaining below the  $10^{-6}$  limit is slightly reduced by the introduction of inner-surface breaking flaws

(1) Maximum cooling rate reduced from 200 F / hr to 175 / hr

(2) maximum allowable  $\Delta t_{\text{initial}}$  slightly reduced



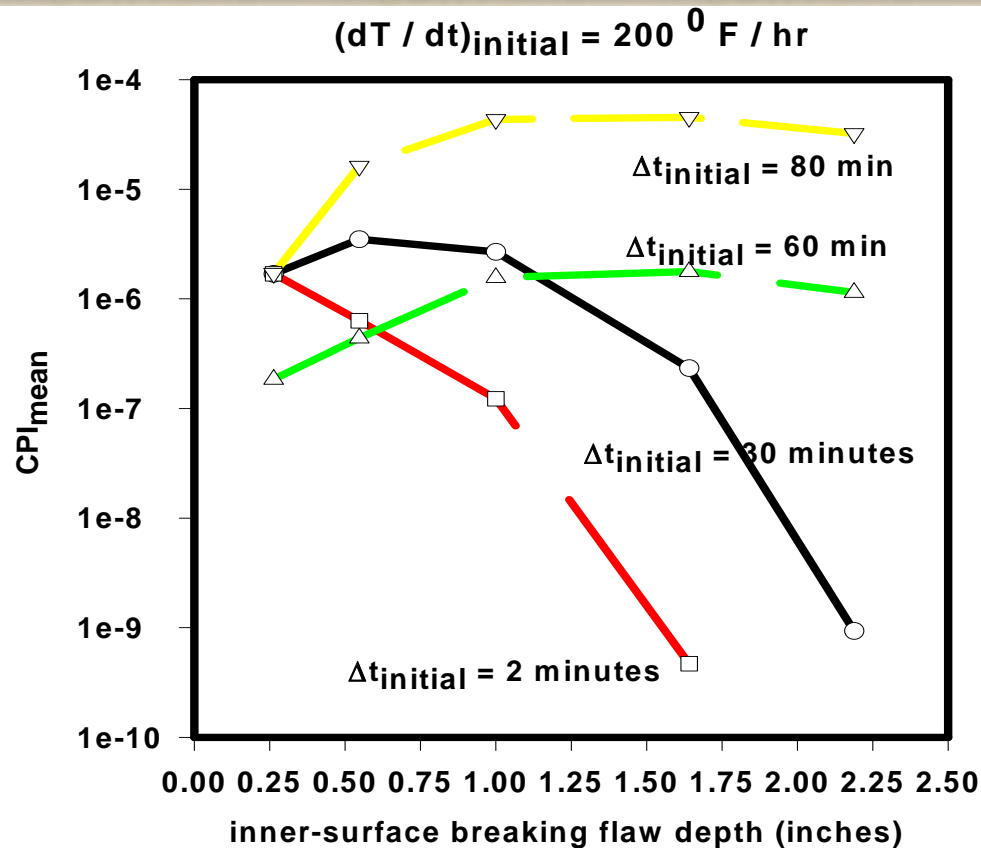
**NOTE:**  $CPI_{\text{mean}}$  of 0 and  $10^{-6}$  used for purpose of illustration only.



A counter-intuitive result from this analysis is that the risk of cleavage fracture does not always increase with increasing inner-surface breaking flaw depth

For some combinations of  $\{(dT / dt)_{\text{initial}}, \Delta t_{\text{initial}}\}$  the applied  $K_I$  for deeper inner-surface breaking flaws reach their peak loading at earlier transient times when the temperature and subsequently the fracture toughness are higher

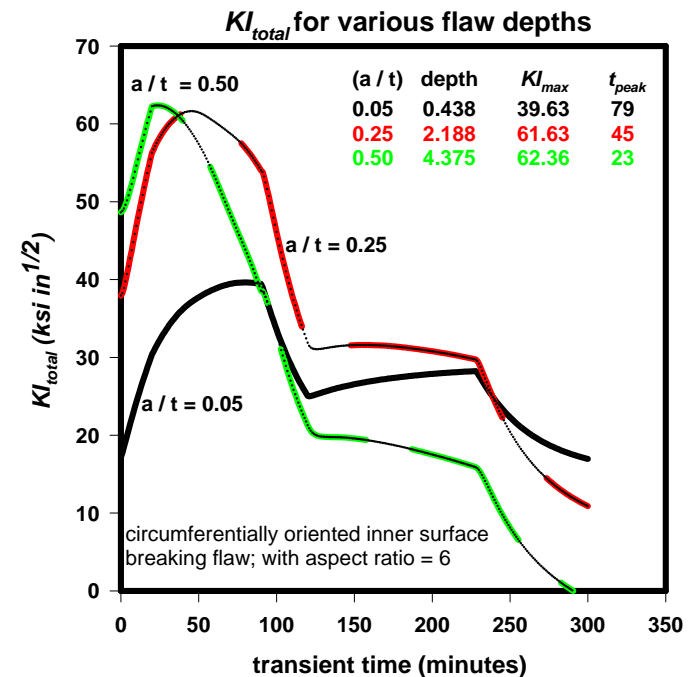
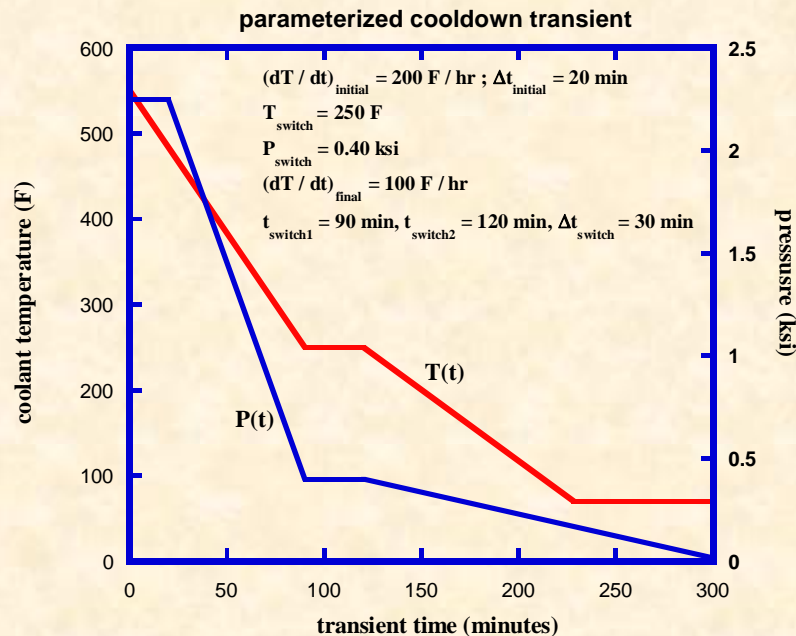
Subject of future publication(s) – mechanistic insights





The time at which the time rate of change of  $KI_{total}$ , designated as  $d(KI_{total}) / dt$ , equals zero corresponds to  $t_{peak}$ , the time at which  $KI_{total}$  reaches its maximum value, which due to WPS, is the last transient time at which a crack can be predicted to initiate in cleavage fracture, also, the last transient time at which a non-zero CPI may be predicted

Subject of future publication(s)

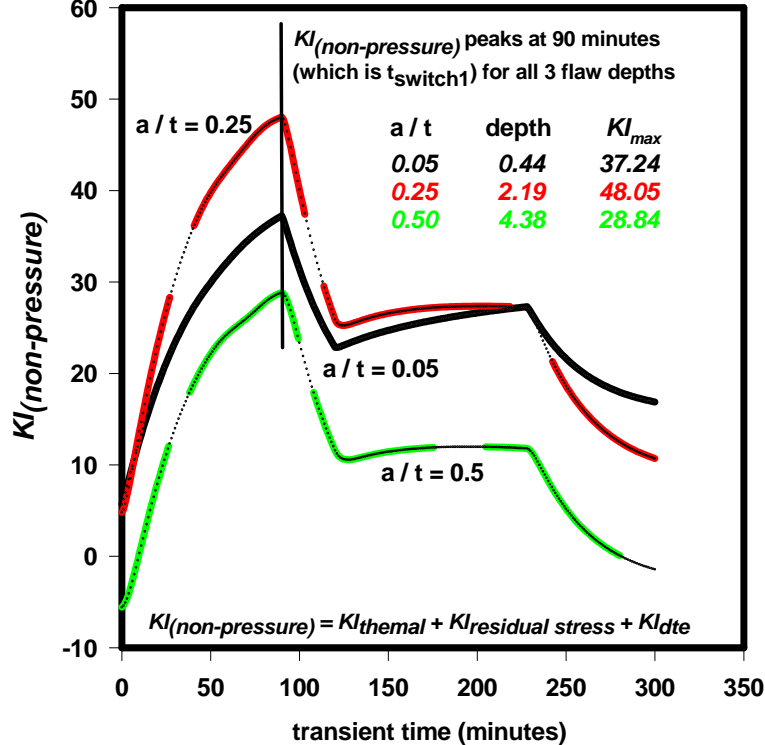


$$KI_{total} = KI_{(non-pressure)} + KI_{(pressure)}$$

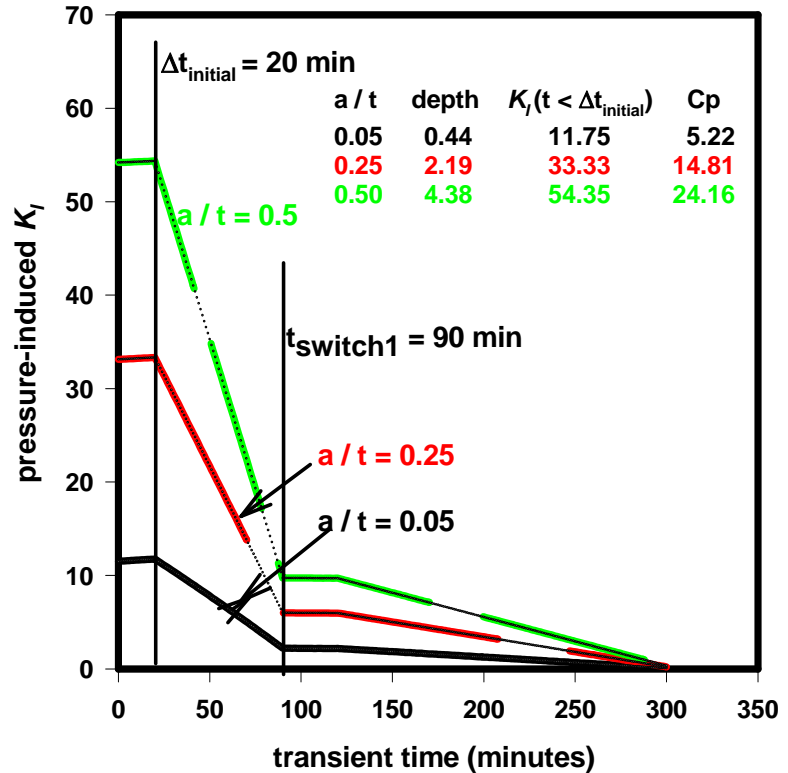
$t_{peak}$  occurs when  $dKI_{total} / dt = (dK / dt)_{pressure} + (dK / dt)_{non-pressure} = 0$

$(dK / dt)_{pressure}$  decreases with flaw depth more rapidly than  $(dK / dt)_{non-pressure}$  increases

Non-pressure induced  $K_I$  peaks at 90 minutes for all three inner-surface breaking flaw depths



Pressure-induced  $K_I$



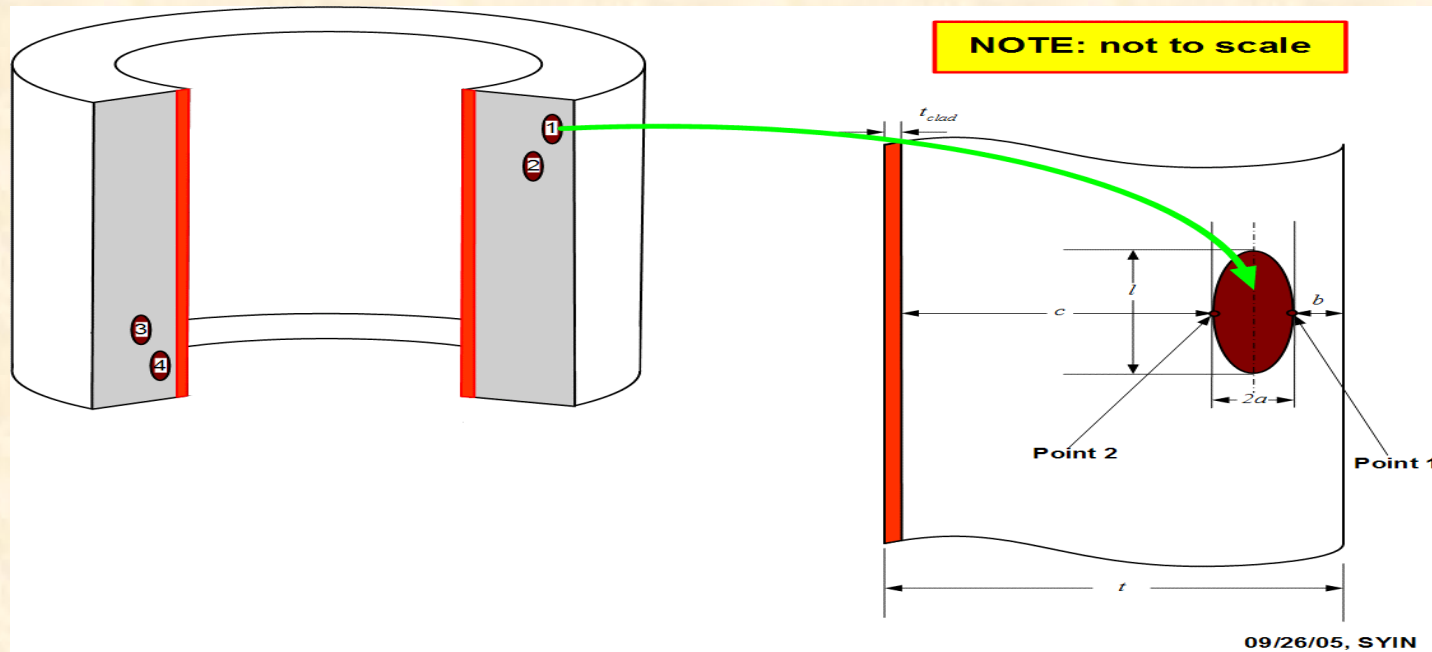
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# FAVOR<sup>HT</sup> was Developed to Perform Fracture Analyses of Heat-Up Transients (such as those associated with reactor start-up)

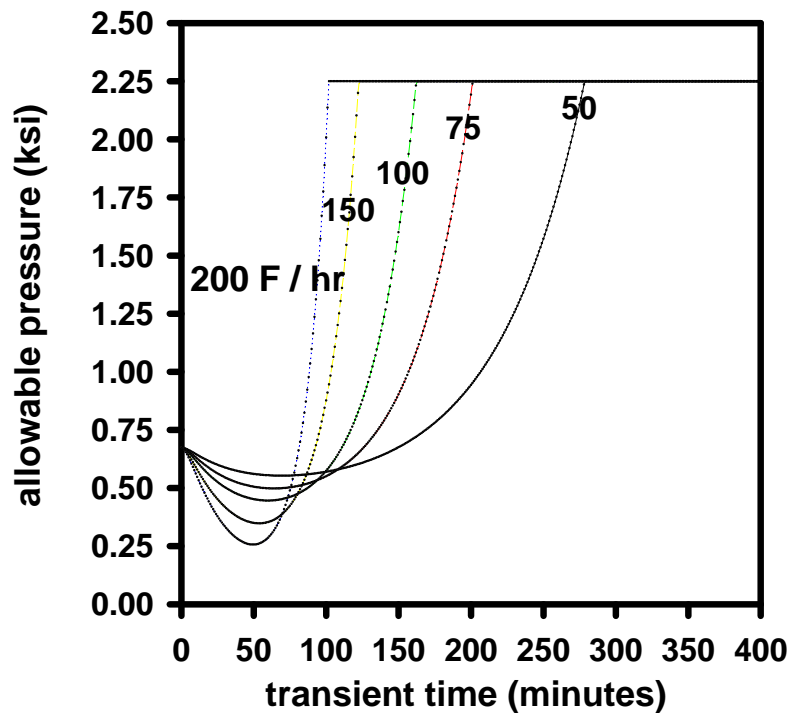
- Previous versions of **FAVOR** designed for analysis of cool-down transients (fracture analyses of **flaws on or near RPV inner surface**)
- **FAVOR<sup>HT</sup>** designed for analysis of heat-up transients (fracture analyses of embedded **flaws near RPV outer surface**)



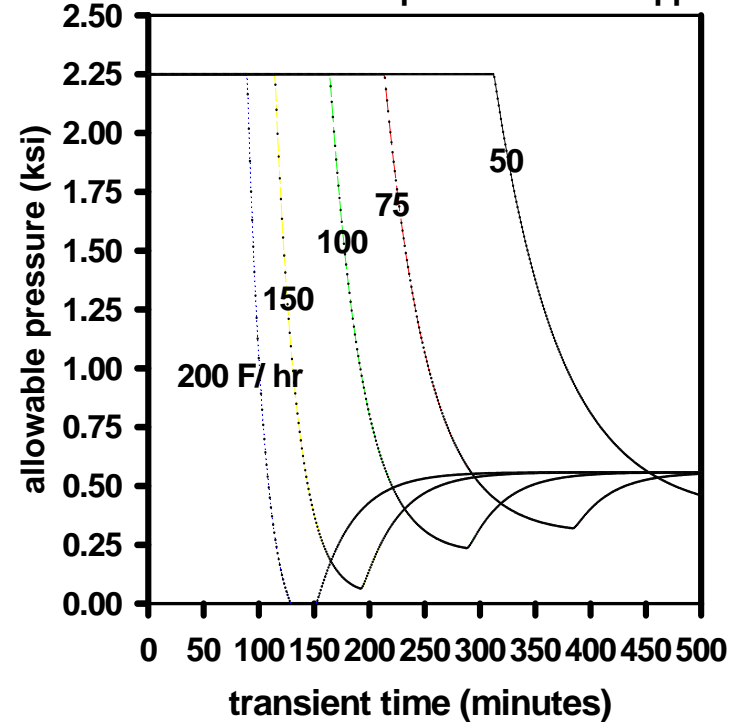
# PFM analyses were performed for Plant X @ 60 EFPY for a range of heat-up and cool-down rates

- FAVOR 06.1 used for cool-down transients
- FAVOR<sup>HT</sup> 06.1 used for heat-up transients

Allowable pressure for heat-up transients for various heat-up rates derived per Section XI - Appendix G



Allowable pressure for cool-down transients for various cool-down rates derived per Section XI - Appendix G



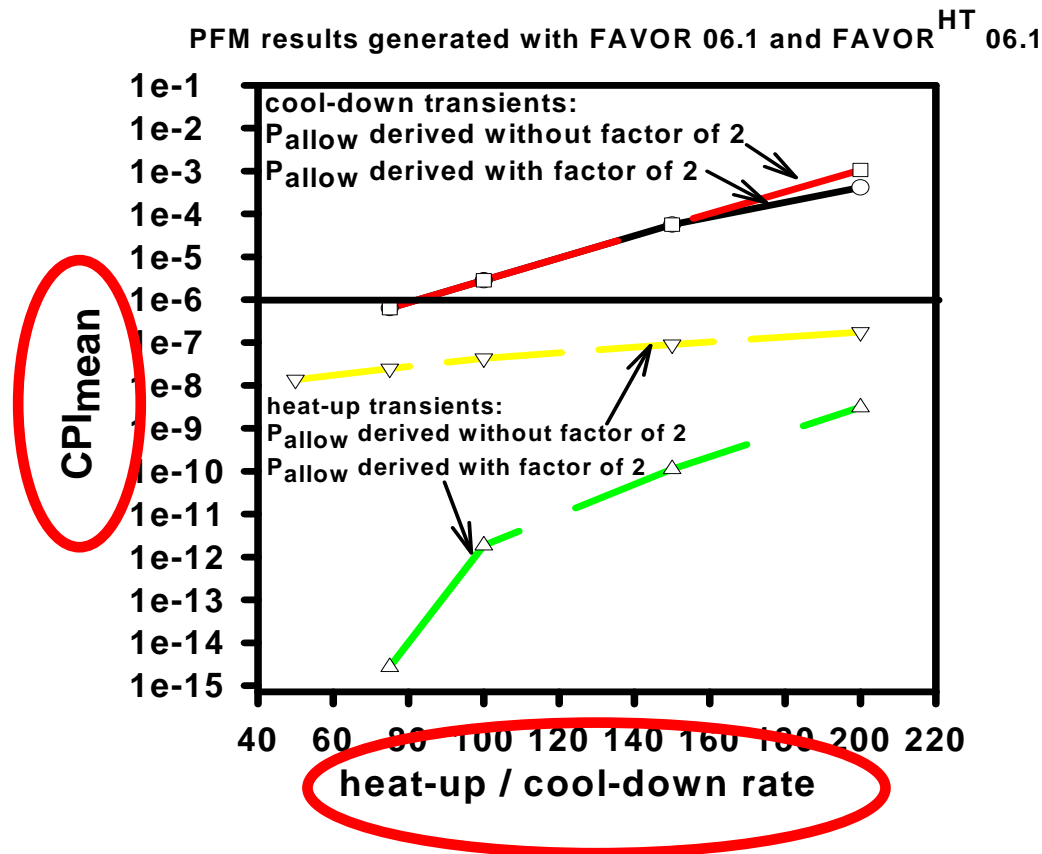


# PFM results for heat-up transients are orders of magnitude lower compared to comparable cool-down transients

For heat-up transients:

(1) the removal of the factor of 2 significantly increases  $CPI_{mean}$

(2) PFM solutions are not sensitive to inclusion of WPS



# Conclusions

(Based on our Analyses of Plant X @ 60 EFPY)

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- (1) PFM analyses for “currently bounding” cool-down transients satisfy the TWCF limit used for PTS of  $10^{-6}$
- (2) For cool-down transients, our calculations indicate that neither of the following changes increase risk
  - (1) Removing the factor of two on pressure
  - (2) Using a smaller reference flaw size
- (3) Initial cooling rates exceeding the current maximum of 100 °F / hr can remain below limits on  $CPI_{\text{mean}}$  of both 0 and  $10^{-6}$  limit if the initial pressure hold time is restricted
- (4) There is potential to develop parametric relationships that satisfy risk-informed criteria for normal cool-down transients.
- (5) Risk associated with reactor heat-up transients are orders of magnitude lower than that for comparable cool-down transients

# Conclusions – continued

(Based on our Analyses of Plant X @ 60 EFPY)

- (6) The introduction of inner-surface breaking flaws into the model does not significantly impact the  $CPI_{\text{mean}}$ -based operating window previously derived using only embedded flaws, i.e., it:
  - (a) Reduces the maximum cool-down rate to  $\sim 175 \text{ F / hr}$
  - (b) slightly reduces the maximum allowable values of  $\Delta t_{\text{initial}}$
- (7) Regardless of flaw type, there is a complex interaction between  $(dT/dt)_{\text{initial}}$  and  $\Delta t_{\text{initial}}$  which determines the time and magnitude of peak loading, which when WPS is included in the model, has a very significant impact on the PFM solution
- (8) PFM results vary by orders of magnitude as a function of pressure hold time  $\Delta t_{\text{initial}}$
- (9) Counter-intuitive result: for some cool-down transients, the risk of brittle fracture does not always vary proportionally with flaw depth; applied  $K_I$  for deeper flaws reach their peak loading value at earlier transient times when the temperature and fracture toughness are higher.

# What's next

- August 21 meeting between NRC / industry
- Need more precise transient characterization
- PFM analyses of other plants at other EFPY

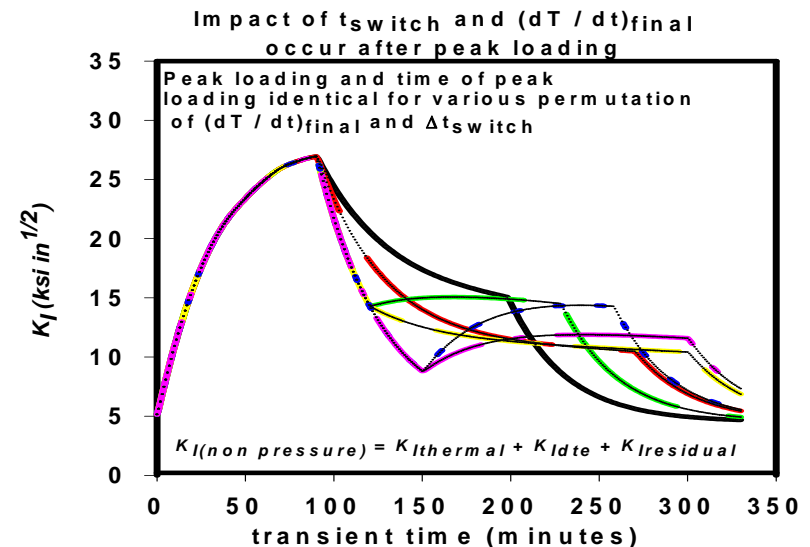
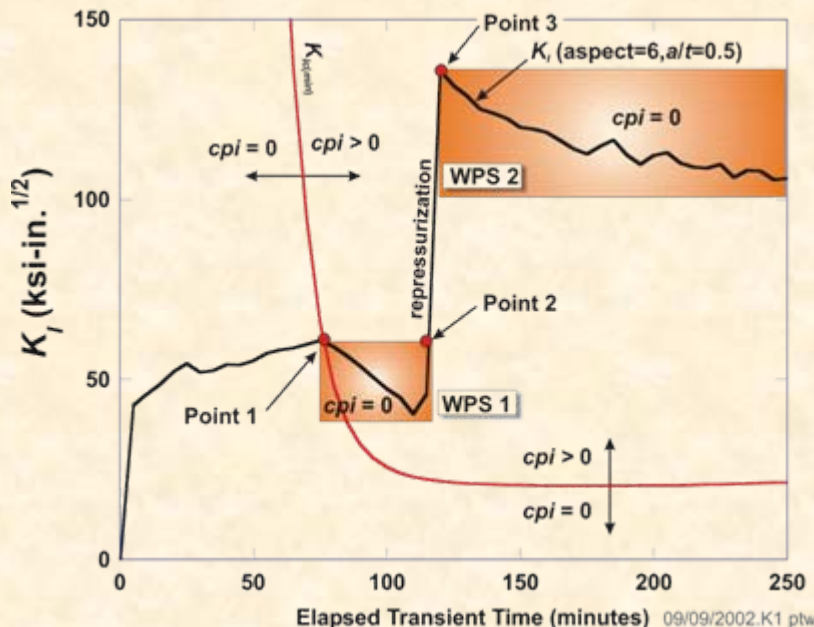
Characteristic of Plant X is that the most embrittled regions are axial welds

Similar analyses for RPVs in which the plate regions are the most highly embrittled could produce different results / conclusions ?

- Generalization of FAVOR to include:
  - (a) ability to also perform analyses of heat-up transients
    - External surface-breaking flaws
    - Embedded flaws in outer 3/8 of RPV wall
  - (b) influence coefficients for internal / external surface breaking flaws for BWR geometries
- Long-term cyclic plasticity effects for normal transients (ORNL paper by Sean Yin, et al; paper 61387)
- Re-visitation of the WPS model – sensitivity analyses with “conservative principle” WPS model
- Incorporation of models that reflect NDE probability and sizing uncertainty
- Definition of vessel “failure,” and tolerable frequency of vessel “failure”

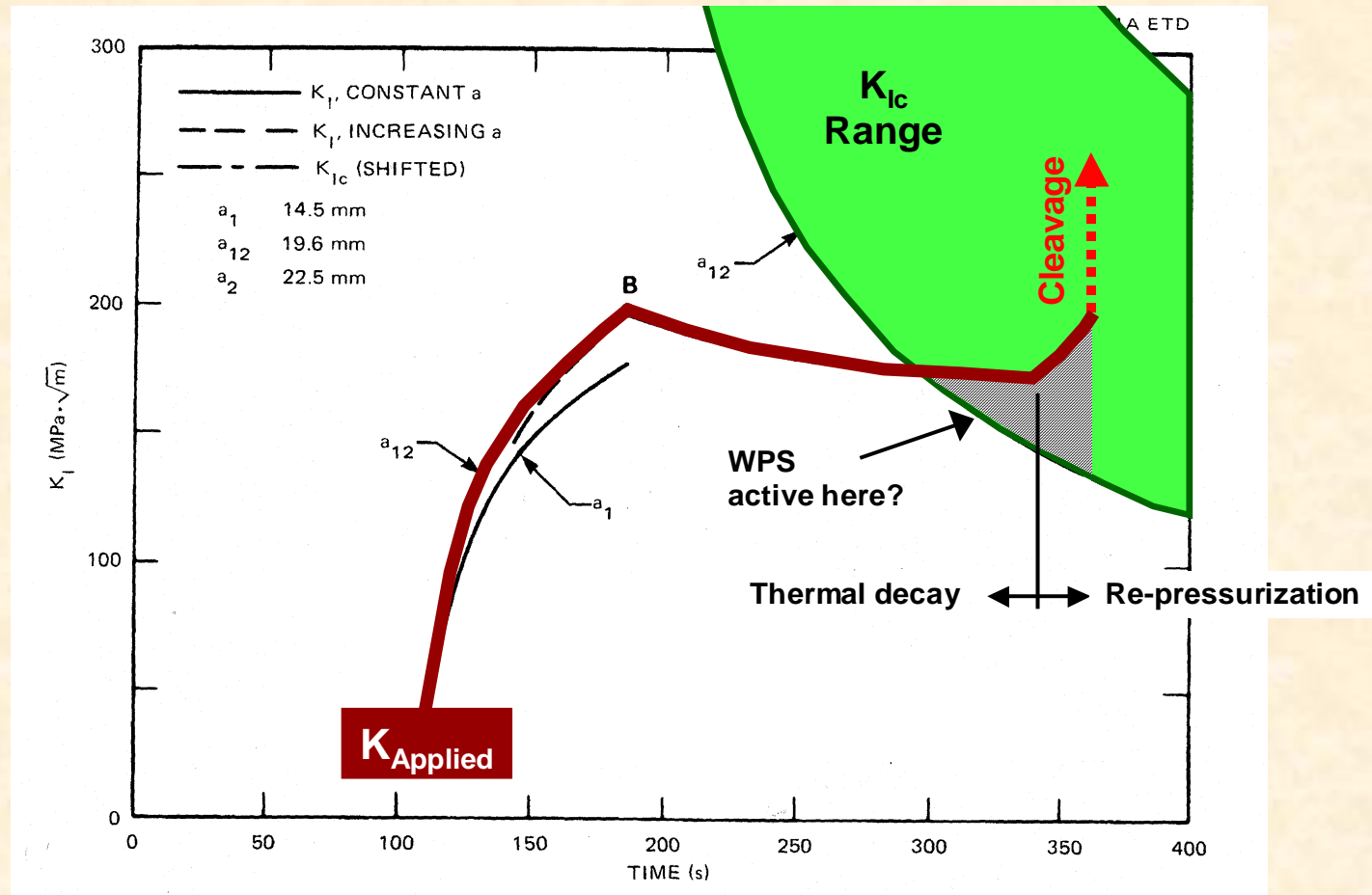
# Currently adding an additional option for “conservative principle” interpretation of warm pre-stress in FAVOR

- Current WPS model in FAVOR: conditions required for  $cpi > 0$ 
  - $K_I > K_{min}$  (Weibull “a” parameter)
  - $K_I(t) >$  previous maximum value  $K_{I(max)}$
  - $cpi = 0$  in reloading phase(s) until  $K_I(t) > K_{I(max)}$
- “Conservative principle” interpretation of WPS: conditions required for  $cpi > 0$ 
  - $K_I > K_{min}$  (Weibull “a” parameter)
  - $K_I(t) > K_I(t-1)$ , i.e., positive slope
  - $cpi > 0$  in reloading phase(s) if  $K_I(t) > K_{min}$

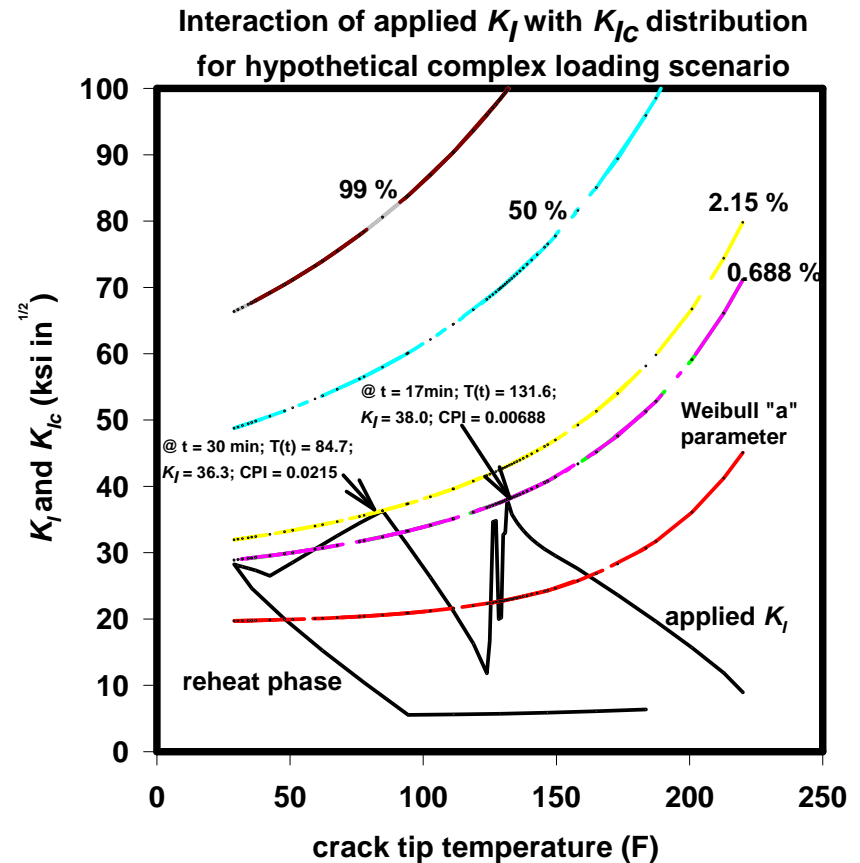
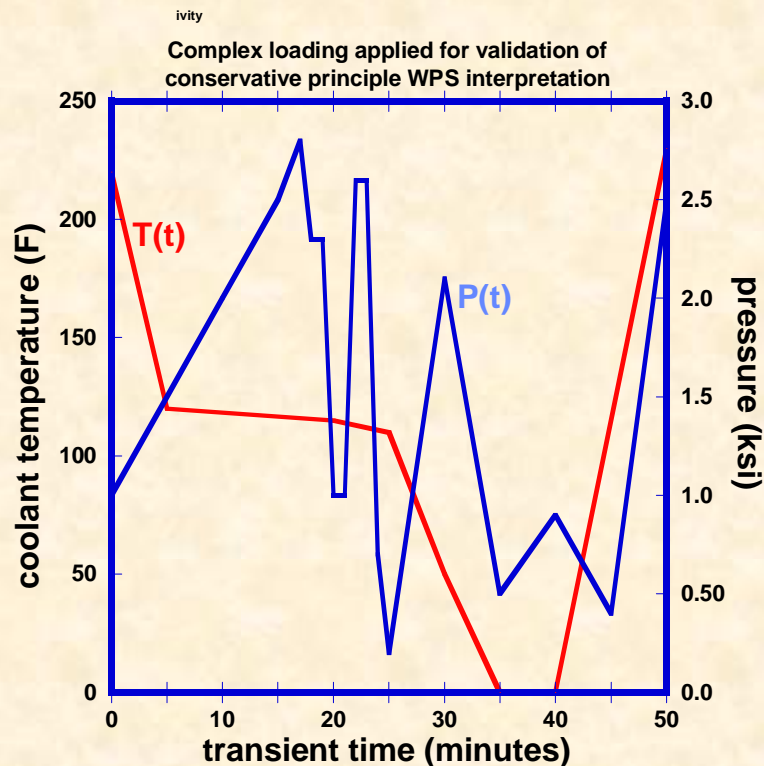




Variation of  $K_{Applied}$  and  $K_{Ic}$  with time in PTSE-2 showing evidence of a potential WPS effect beginning at around 300 seconds.

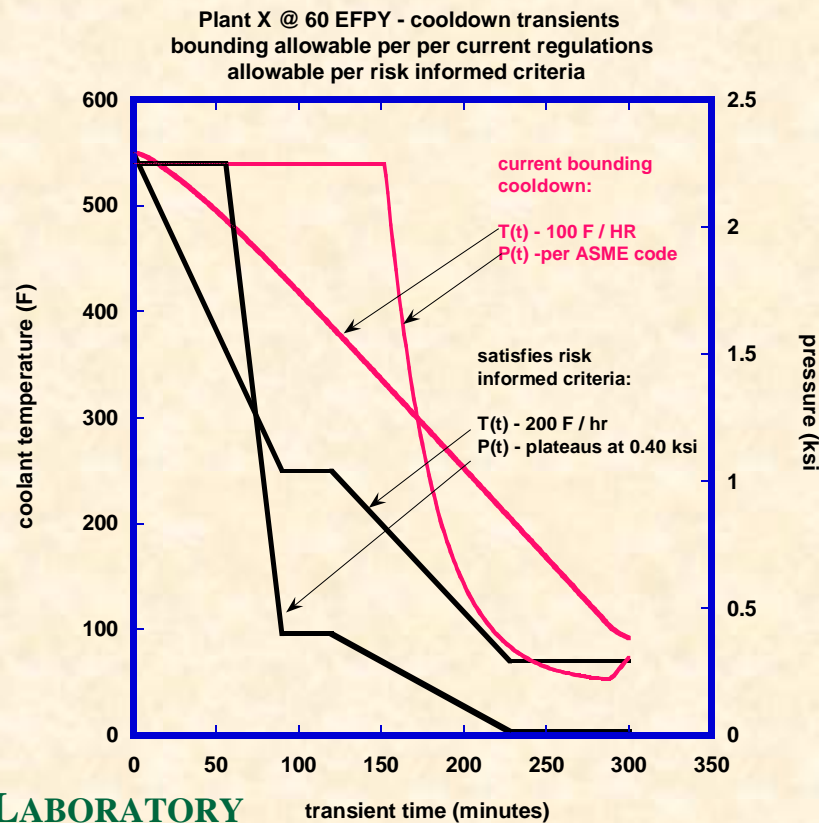


# A hypothetical complex loading scenario is being used to validate the implementation of the “conservative principle” interpretation of WPS into FAVOR



# Backup Slides

# Risk informed approach results in considerable relaxation of maximum cool-down rate relative to bounding transient derived from ASME Section XI – Appendix G

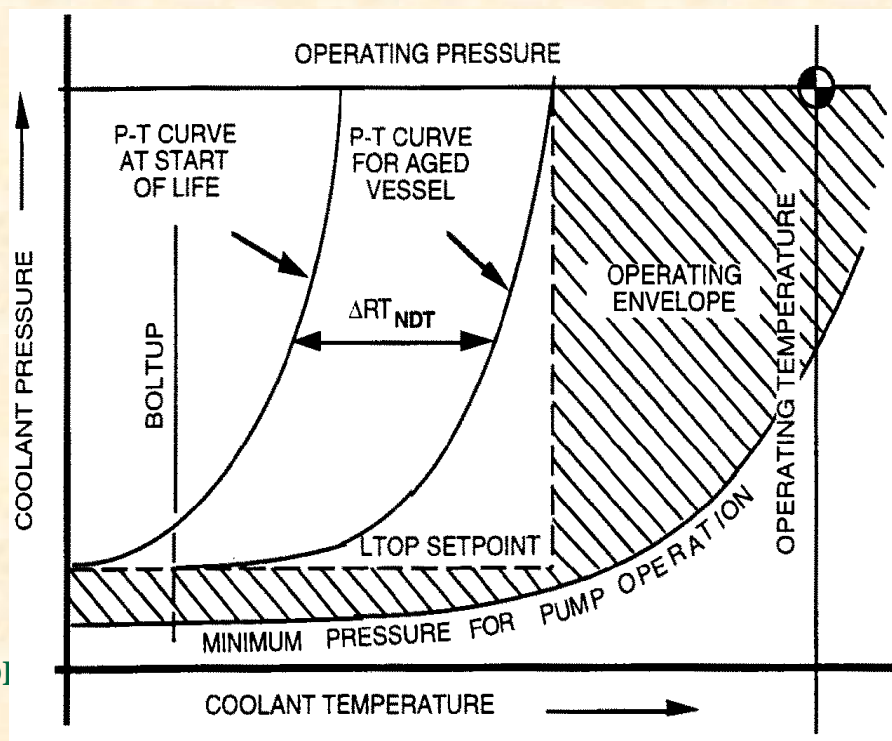


***The P-T operating envelope is progressively restricted to accommodate the effects of irradiation embrittlement of the RPV material***

***The P-T curve controls the upper-bound to the permissible operating envelope for a RPV during normal start-up and cool-down transients***

***The P-T curve is currently derived using a prescriptive deterministic fracture methodology in ASME Section XI – Appendix G***

***An objective of ORNL study is to determine if a technical basis can be established to support a relaxation to the methodology in ASME Section XI – Appendix G***

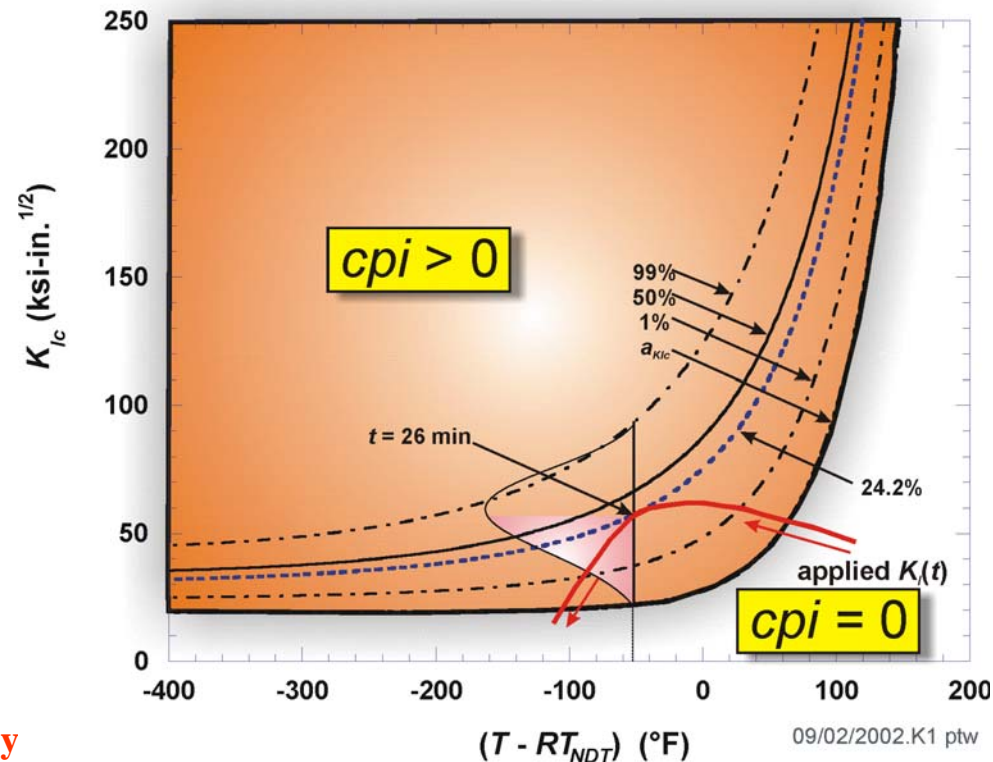




FAVOR Review: cpi is determined from interaction of *applied*  $K_I$  and  $K_{Ic}$

Without WPS: for  $cpi > 0$ , *applied*  $K_I$  must be greater than Weibull “a” parameter which is the lower bound at any transient time

With WPS: for  $cpi > 0$ , *max*  $K_I$  must be greater than Weibull “a” parameter at transient time before maximum load is reached



The statistical distribution in FAVOR is based on an extended  $K_{Ic}$  database relative to that from which the ASME lower bound-curve was derived

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