RAS M-202

NEC-JH_32

VERMONT YANKEE
NUCLEAR POWER CORPORATION

185 OLD FERRY ROAD, PO BOX 7002, BRATTLEBORO, VT 05302-7002
(802) 257-5271

August 20, 2001
BVY 01-66

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Subject: **Vermont Yankee Nuclear Power Station**
License No. DPR-28 (Docket No. 50-271)
Vermont Yankee 2001 Summary Reports for
In-service Inspection and Repairs or Replacements

In accordance with Article IWA-6000 of Section XI of the ASME Boiler and Pressure Vessel Code, Vermont Yankee (VY) hereby submits the Owner's Report for In-service Inspections (Form NIS-1) and the Owner's Report for Repairs and Replacements (Form NIS-2). These reports describe the in-service examinations, repairs and replacements performed during the period from December 4, 1999 to May 20, 2001 (including Refueling Outage 22). VY's third ten-year interval began September 1, 1993.

We trust that the information provided is adequate; however, should you have questions or require additional information, please contact Mr. Jim DeVincentis at (802) 258-4236.

Sincerely,

VERMONT YANKEE NUCLEAR POWER CORPORATION

Gautam Sen
Licensing Manager

Attachments

cc: USNRC Region 1 Administrator
USNRC Resident Inspector – VYNPS
USNRC Project Manager – VYNPS
Vermont Department of Public Service
Inspection Agency - Arkwright

DOCKETED
USNRC

August 12, 2008 (11:00am)

OFFICE OF SECRETARY
RULEMAKINGS AND
ADJUDICATIONS STAFF

AD47

U.S. NUCLEAR REGULATORY COMMISSION
In the Matter of Entergy Nuclear Vermont Yankee LLC
Docket No. 50-271 Official Exhibit No. NEC-JH-32
OFFERED by: Applicant/Licensee (Intervenor) NEC
NRC Staff Other
IDENTIFIED on 7/21/08 Witness/Panel Hopenfeld
Action Taken: (ADMITTED) REJECTED WITHDRAWN
Reporter/Clerk MAC

Template Secy-028

D503

SUMMARY OF VERMONT YANKEE COMMITMENTS

BVY NO.: 01-66

The following table identifies commitments made in this document by Vermont Yankee. Any other actions discussed in the submittal represent intended or planned actions by Vermont Yankee. They are described to the NRC for the NRC's information and are not regulatory commitments. Please notify the Licensing Manager of any questions regarding this document or any associated commitments.

COMMITMENT	COMMITTED DATE OR "OUTAGE"
None	N/A

Vermont Yankee Nuclear Power Corporation

2001 Form NIS-1 Owner's Summary Report for Inservice Inspections

December 4, 1999 through May 20, 2001

Reviewed by: _____ 8/2/01
Plant Inservice Inspection Coordinator

Approved by: DC Simon for J. Dreyfus 8/2/01
System Engineering Manager

FORM NIS-1 OWNER'S REPORT FOR INSERVICE INSPECTIONS
As Required by the Provisions of the ASME Code Rules

1. Owner Vermont Yankee Nuclear Power Corporation, 185 Old Ferry Road, PO Box 7002, Brattleboro VT 05302-7002
(Name and Address of Owner)

2. Plant Vermont Yankee Nuclear Power Station, P.O. Box 157, Governor Hunt Road, Vernon, VT 05354-0157
(Name and Address of Plant)

3. Plant Unit 1 4. Owner Certificate of Authorization (if required) DPR-28

5. Commercial Service Date 11/30/1972 6. National Board Number for Unit NONE

7. Components Inspected - SEE ATTACHED PAGES 2 THROUGH 13.

8. Examination Dates 12/04/1999 to 05/20/2001 9. Inspection Interval from 09/1/1993 to 08/31/2003

10. Applicable Editions of Section XI 1986, no Addenda; 1992 w/1992 Addenda (IWE) and 1995 w/1996 Addenda (ASME Appendix VIII)

11. Abstract of Examinations Including a list of examinations and a statement concerning status of work required for current interval - SEE ATTACHED PAGES 2 THROUGH 21.

12. Abstract of Conditions Noted - SEE ATTACHED PAGES 22 THROUGH 25.

13. Abstract of Corrective Measures Recommended and Taken - SEE ATTACHED PAGES 22 THROUGH 25.

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Certificate of Authorization No. (If applicable) DPR-28 Expiration Date 3/21/2012

Date August 16 20 01 Signed _____ By Vermont Yankee Nuclear Pwr
Owner

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of Vermont and employed by Factory Mutual Insurance Co. of Johnston RI have inspected the components described in this Owner's Report during the period December 4, 1999 to May 20, 2001 and state to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

_____ Commissions VT-345
Inspector's Signature National Board, State, Province, and Endorsements

Date 8-16, 2001

FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS
As Required by the Provisions of the ASME Code rules

Vermont Yankee Nuclear Power Corporation
Vermont Yankee Nuclear Power Station
Owner Certification: DPR-28
Commercial Service Date: 11/30/72

Components Inspected/Abstract of examinations
Sections 7 and 11

ASME Category	*Component ID*	*Exam Type*	*System ID*	*Drawing No.*	*Examination Results*
B-D	N2F	UT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-D	N2F-IR	UT Inner Radius	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-D	N2G	UT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-D	N2G-IR	UT Inner Radius	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-D	N2H	UT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-D	N2H-IR	UT Inner Radius	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-D	N2J	UT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-D	N2J-IR	UT Inner Radius	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-D	N2K	UT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-D	N2K-IR	UT Inner Radius	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-D	N4A-IR	UT Inner Radius	Feedwater	ISI-RPV-103	**Acceptable -** Automated Inner Radius examination in accordance with General Electric Nuclear Energy document GE-NE-523-A71-0594-A, Revision 1 and VY Calculation VYC-1005
B-D	N4B-IR	UT Inner Radius	Feedwater	ISI-RPV-103	**Acceptable -** Automated Inner Radius examination in accordance with General Electric Nuclear Energy document GE-NE-523-A71-0594-A, Revision 1 and VY Calculation VYC-1005

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Components Inspected/Abstract of examinations
Sections 7 and 11

ASME Category	*Component ID*	*Exam Type*	*System ID*	*Drawing No.*	*Examination Results*
B-D	N4C-IR	UT Inner Radius	Feedwater	ISI-RPV-103	**Acceptable -** Automated Inner Radius examination in accordance with General Electric Nuclear Energy document GE-NE-523-A71-0594-A, Revision 1 and VY Calculation VYC-1005
B-D	N4D-IR	UT Inner Radius	Feedwater	ISI-RPV-103	**Acceptable -** Automated Inner Radius examination in accordance with General Electric Nuclear Energy document GE-NE-523-A71-0594-A, Revision 1 and VY Calculation VYC-1005
B-F	N11A WELD	PT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-F	N11B WELD	PT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-F	N12A-SE	PT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-F	N1B-SE	PT	Nuclear Boiler	ISI-RPV-103	**Acceptable -** Examination performed as follow up to indication removal during RFO-21
B-F	N2F-SE	UT, PT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-F	N2G-SE	UT, PT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-F	N2H-SE	UT, PT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-F	N2J-SE	UT, PT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-F	N2K-SE	UT, PT	Nuclear Boiler	ISI-RPV-103	**Acceptable**

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Sections 7 and 11

ASME Category	*Component ID*	*Exam Type*	*System ID*	*Drawing No.*	*Examination Results*
B-F	N6B-SE	UT, PT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-F	N8A-SE	UT, PT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-F	N8B-SE	UT, PT	Nuclear Boiler	ISI-RPV-103	**Acceptable**
B-G-1	01A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13
B-G-1	02A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13
B-G-1	03A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13
B-G-1	04A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13
B-G-1	05A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13
B-G-1	06A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13
B-G-1	07A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13

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Vermont Yankee Nuclear Power Corporation
Vermont Yankee Nuclear Power Station
Owner Certification: DPR-28
Commercial Service Date: 11/30/72

Components Inspected/Abstract of examinations
Sections 7 and 11

ASME Category	*Component ID*	*Exam Type*	*System ID*	*Drawing No.*	*Examination Results*
B-G-1	08A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13
B-G-1	09A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13
B-G-1	10A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13
B-G-1	11A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13
B-G-1	12A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13
B-G-1	13A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13
B-G-1	14A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13
B-G-1	15A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	**Acceptable -** IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13

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Vermont Yankee Nuclear Power Corporation
Vermont Yankee Nuclear Power Station
Owner Certification: DPR-28
Commercial Service Date: 11/30/72

Components Inspected/Abstract of examinations
Sections 7 and 11

ASME Category	*Component ID*	*Exam Type*	*System ID*	*Drawing No.*	*Examination Results*
B-G-1	16A-N/W Recirculation Pump P-18-1A Bolting	VT-1	Nuclear Boiler	ISI-RPV-104	Acceptable - IDR # 01-09 generated for corrosion/plating concern - See Sections 12 and 13
B-J	CS4B-F3ADW	UT, PT	Core Spray	ISI-5920-9206	Acceptable
B-J	CS4B-MF5	UT, PT	Core Spray	ISI-5920-9206	Acceptable
B-J	CS4B-MF5B	UT, PT	Core Spray	ISI-5920-9206	Acceptable
B-J	CS4B-MF6A	UT, PT	Core Spray	ISI-5920-9206	Acceptable
B-J	FW20-F1	UT/FAC	Feedwater	ISI-FDW-PART 5A	Acceptable - Code Case N-560 examination
B-J	FW20-F1B	UT/FAC	Feedwater	ISI-FDW-PART 5A	Acceptable - Code Case N-560 examination
B-J	FW20-F3B	UT/FAC	Feedwater	ISI-FDW-PART 5A	Acceptable - Code Case N-560 examination
B-J	SL11-F28	PT	Standby Liquid Control	ISI-SLC-PART 4	Acceptable
B-J	SL11-F29	PT	Standby Liquid Control	ISI-SLC-PART 4	Acceptable
B-K	270 DEG RPV BRKT	PT	Nuclear Boiler	ISI-RPV-103	Acceptable
B-K	RPV SUPPORT SKIRT	MT	Nuclear Boiler	ISI-RPV-103	Acceptable
B-K	RR-34	PT	Nuclear Boiler	ISI-5920-6802 Sh.2	Acceptable
B-K	RR-35	PT	Nuclear Boiler	ISI-5920-6802 Sh.2	Acceptable
B-O	26-03SH	PT	Control Rod Drive	ISI-RPV-104	Acceptable
B-O	34-39SH	PT	Control Rod Drive	ISI-RPV-104	Acceptable

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Vermont Yankee Nuclear Power Station
Owner Certification: DPR-28
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Components Inspected/Abstract of examinations
Sections 7 and 11

ASME Category	*Component ID*	*Exam Type*	*System ID*	*Drawing No.*	*Examination Results*
C-C	ACSP-H22	MT	Standby Gas Treatment	ISI-5920-9200	Acceptable
C-C	ACSP-H23	MT	Standby Gas Treatment	ISI-5920-9200	Acceptable
C-C	RHR-H192	MT	Residual Heat Removal	ISI-RHR-PART 11 Sh.4	Acceptable
C-C	RHR-H98	MT	Residual Heat Removal	ISI-5920-9208	Acceptable
C-C	RHR-HD25	PT	Residual Heat Removal	ISI-RHR-PART 16 Sh.1	Acceptable - Successive examination
C-F-2	CR4A-S5	UT, MT	Control Rod Drive	ISI-5920-9528	Acceptable
C-F-2	CR6A-S57	UT, MT	Control Rod Drive	ISI-5920-9527	Acceptable
C-F-2	CR6-S10	UT, MT	Control Rod Drive	ISI-5920-9527	Acceptable
C-F-2	CR6-S22	UT, MT	Control Rod Drive	ISI-5920-9527	Acceptable
C-F-2	CR6-S26	UT, MT	Control Rod Drive	ISI-5920-9527	Acceptable
C-F-2	CS1B-S30	UT, MT	Core Spray	ISI-5920-9210	Acceptable
C-F-2	CT27-S30	UT, MT	Core Spray	ISI-5920-9210	Acceptable
C-F-2	FW17-S5	UT, MT	Feedwater	ISI-FDW-PART 5A	Acceptable
C-F-2	HP15A-S101	UT, MT	High Pressure Coolant Injection	ISI-HPCI-PART 5	Acceptable
C-F-2	RH14-T373	UT, MT	Core Spray	ISI-5920-9208	Acceptable
C-F-2	RH1B-S47	UT, MT	Residual Heat Removal	ISI-5920-9285	Acceptable
C-F-2	RH2B-S113	UT, MT	Residual Heat Removal	ISI-5920-9285	Acceptable

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Vermont Yankee Nuclear Power Station
Owner Certification: DPR-28
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Components Inspected/Abstract of examinations
Sections 7 and 11

ASME Category	*Component ID*	*Exam Type*	*System ID*	*Drawing No.*	*Examination Results*
C-F-2	RH2B-S115	UT, MT	Residual Heat Removal	ISI-5920-9285	Acceptable
C-F-2	RH3B-S170	UT, MT	Residual Heat Removal	ISI-5920-9288	Acceptable
C-F-2	RH3D-S200	UT, MT	Residual Heat Removal	ISI-5920-9288	Acceptable
C-F-2	RH3D-S206	UT, MT	Residual Heat Removal	ISI-5920-9288	Acceptable
C-F-2	RH3D-T182	UT, MT	Residual Heat Removal	ISI-5920-9288	Acceptable
C-F-2	RH7-S284	UT, MT	Residual Heat Removal	ISI-5920-9287	Acceptable
C-F-2	RH9-S314	UT, MT	Residual Heat Removal	ISI-RHR-PART 11 Sh.4	Acceptable
C-F-2	RH9-S320	UT, MT	Residual Heat Removal	ISI-RHR-PART 11 Sh.4	Acceptable
C-FAUG	CS2A-S62	UT, MT	Core Spray	ISI-5920-9211	Acceptable
C-FAUG	CS2A-S64	UT, MT	Core Spray	ISI-5920-9211	Acceptable
C-FAUG	CS2A-S65	UT, MT	Core Spray	ISI-5920-9211	Acceptable
C-FAUG	CS2A-S67	UT, MT	Core Spray	ISI-5920-9211	Acceptable
C-FAUG	CT1-S54	UT, PT	Core Spray	ISI-CST-PART 4	Acceptable
C-FAUG	CT1-S56	UT, PT	Core Spray	ISI-CST-PART 4	Acceptable
C-FAUG	RC3-S13	UT, MT	Reactor Core Isolation Cooling	ISI-5920-9255	Acceptable
C-FAUG	RC3-S14	UT, MT	Reactor Core Isolation Cooling	ISI-5920-9255	Acceptable

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Vermont Yankee Nuclear Power Corporation
Vermont Yankee Nuclear Power Station
Owner Certification: DPR-28
Commercial Service Date: 11/30/72

Components Inspected/Abstract of examinations
Sections 7 and 11

ASME Category	*Component ID*	*Exam Type*	*System ID*	*Drawing No.*	*Examination Results*
C-NAUG	MS1D-F9	UT, MT	Main Steam	5920-FS-I1	**Acceptable**
C-NAUG	MS2D-F1	UT, MT	Main Steam	5920-FS-I1	**Acceptable**
D-A	RSW-H171	VT-1	Residual Heat Removal	ISI-SW-PART 9	**Acceptable**
D-A	RSW-H261	VT-1	Residual Heat Removal	ISI-SW-PART 9	**Acceptable -** IDR # 01-01 generated for arc strikes - See Sections 12 and 13
D-A	RSW-HD261B	VT-1	Residual Heat Removal	ISI-SW-PART 9	**Acceptable -** IDR # 01-01 generated for arc strikes - See Sections 12 and 13
E-A	Class MC Containment	General Visual	Class MC Containment	5920-13, 5920-41, 5920-42, 6202-200	**Acceptable -** General Visual Examination as required by ASME Subsection IWE has been 100% completed for the first period of the first IWE Interval. IDR # 01-07 generated for pitting and general corrosion in the Vent Header. IDR # 01-07 generated for pitting and general corrosion in the Vent Header. Also, IDR # 01-08 was generated for general corrosion and material loss in Penetrations X-207A through X-207H. See Sections 12 and 13
E-A	Vent Line Areas (X-5B)	VT-1	Class MC Containment	5920-13	**Acceptable**
E-A	Vent Line Areas (X-5C)	VT-1	Class MC Containment	5920-13	**Acceptable**
E-A	Vent Line Areas (X-5D)	VT-1	Class MC Containment	5920-13	**Acceptable**
E-A	Vent Line Areas (X-5E)	VT-1	Class MC Containment	5920-13	**Acceptable**

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Vermont Yankee Nuclear Power Corporation
Vermont Yankee Nuclear Power Station
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Components Inspected/Abstract of examinations
Sections 7 and 11

ASME Category	*Component ID*	*Exam Type*	*System ID*	*Drawing No.*	*Examination Results*
E-C	Drywell Seal Area	VT-1	Class MC Containment	6202-2	Acceptable
E-C	Drywell Seal Area	VT-3	Class MC Containment	6202-2	Acceptable
E-G	Pen. X-200A	VT-1	Class MC Containment	6202-208	Acceptable
E-G	Pen. X-200B	VT-1	Class MC Containment	6202-208	Acceptable
E-G	V16-19-5A	VT-1	Class MC Containment	5920-675	Acceptable
E-G	V16-19-5B	VT-1	Class MC Containment	5920-675	Acceptable
E-G	V16-19-5C	VT-1	Class MC Containment	5920-675	Acceptable
E-G	V16-19-5D	VT-1	Class MC Containment	5920-675	Acceptable
E-G	V16-19-5E	VT-1	Class MC Containment	5920-675	Acceptable
E-G	V16-19-5F	VT-1	Class MC Containment	5920-675	Acceptable
E-G	V16-19-5H	VT-1	Class MC Containment	5920-675	Acceptable
F-A	ACSP-H22	VT-3	Standby Gas Treatment	ISI-5920-9200	Acceptable
F-A	ACSP-H23	VT-3	Standby Gas Treatment	ISI-5920-9200	Acceptable
F-A	CS-HD60A	VT-3	Core Spray	ISI-5920-9210	Acceptable
F-A	FDW-HD39	VT-3	Feedwater	ISI-FDW-PART 5A	Acceptable - IDR # 01-12 generated for debris/corrosion - See Sections 12 and 13
F-A	H-P-44-1B	VT-3	High Pressure Coolant Injection	ISI-HPCI-PART 13A	Acceptable

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F-A	HPCI-1	VT-3	High Pressure Coolant Injection	ISI-HPCI-PART 2	Acceptable
F-A	HPCI-2	VT-3	High Pressure Coolant Injection	ISI-HPCI-PART 2	Acceptable
F-A	RHR-H129	VT-3	Residual Heat Removal	ISI-5920-9288	Acceptable
F-A	RHR-H191	VT-3	Residual Heat Removal	ISI-RHR-PART 11 Sh.4	Acceptable
F-A	RHR-H192	VT-3	Residual Heat Removal	ISI-RHR-PART 11 Sh.4	Acceptable - IDR # 01-02 generated for gouge - See Sections 12 and 13
F-A	RHR-H83	VT-3	Residual Heat Removal	ISI-RHR-PART 11 Sh.4	Acceptable
F-A	RHR-H98	VT-3	Residual Heat Removal	ISI-5920-9208	Acceptable
F-A	RHR-HD127C	VT-3	Residual Heat Removal	ISI-5920-9285	Acceptable
F-A	RHR-HD127E	VT-3	Residual Heat Removal	ISI-5920-9285	Acceptable - Successive Examination
F-A	RHR-HD127G	VT-3	Residual Heat Removal	ISI-5920-9285	Acceptable
F-A	RHR-HD188A	VT-3	Residual Heat Removal	ISI-5920-9288	Acceptable
F-A	RPV SUPPORT SKIRT	VT-3	Nuclear Boiler	ISI-RPV-103	Acceptable
F-A	RR-15	VT-3	Nuclear Boiler	ISI-5920-6802 Sh.2	Acceptable
F-A	RR-16	VT-3	Nuclear Boiler	ISI-5920-6802 Sh.2	Acceptable
F-A	RR-17	VT-3	Nuclear Boiler	ISI-5920-6802 Sh.2	Acceptable
F-A	RR-2	VT-3	Nuclear Boiler	ISI-5920-6802 Sh.2	Acceptable

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Vermont Yankee Nuclear Power Corporation
Vermont Yankee Nuclear Power Station
Owner Certification: DPR-28
Commercial Service Date: 11/30/72

Components Inspected/Abstract of examinations
Sections 7 and 11

ASME Category	*Component ID*	*Exam Type*	*System ID*	*Drawing No.*	*Examination Results*
F-A	RR-35	VT-3	Nuclear Boiler	ISI-5920-6802 Sh.2	Acceptable
F-A	RR-44	VT-3	Nuclear Boiler	ISI-5920-6802 Sh.2	Acceptable - IDR # 01-04 generated for setting - See Sections 12 and 13
F-A	RR-52	VT-3	Nuclear Boiler	ISI-5920-6802 Sh.2	Acceptable - IDR # 01-05 generated for setting - See Sections 12 and 13
F-A	RR-7,8	VT-3	Nuclear Boiler	ISI-5920-6802 Sh.2	Acceptable - IDR # 01-06 generated for setting - See Sections 12 and 13
F-A	RSW-H167	VT-3	Residual Heat Removal	ISI-SW-PART 1 Sh.2	Acceptable
F-A	RSW-H171	VT-3	Residual Heat Removal	ISI-SW-PART 9	Acceptable
F-A	RSW-H172	VT-3	Residual Heat Removal	ISI-SW-PART 6 Sh.1	Acceptable
F-A	RSW-H241	VT-3	Residual Heat Removal	ISI-SW-PART 1 Sh.2	Acceptable
F-A	RSW-H261	VT-3	Residual Heat Removal	ISI-SW-PART 9	Acceptable
F-A	RSW-HD261B	VT-3	Residual Heat Removal	ISI-SW-PART 9	Acceptable
F-A	SDV-N-R02	VT-3	Control Rod Drive	ISI-5920-9527	Acceptable
F-A	SDV-N-R05	VT-3	Control Rod Drive	ISI-5920-9527	Acceptable
F-AUG	ACSP-H203	VT-3	Standby Gas Treatment	ISI-5920-9201	Acceptable
F-AUG	ACSP-HD203E	VT-3	Standby Gas Treatment	ISI-5920-9201	Acceptable
F-AUG	ACSP-HD203F	VT-3	Standby Gas Treatment	ISI-5920-9201	Acceptable

FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS
As Required by the Provisions of the ASME Code rules

Vermont Yankee Nuclear Power Corporation
Vermont Yankee Nuclear Power Station
Owner Certification: DPR-28
Commercial Service Date: 11/30/72

Components Inspected/Abstract of examinations
Sections 7 and 11

ASME Category	*Component ID*	*Exam Type*	*System ID*	*Drawing No.*	*Examination Results*
F-AUG	RHR-HD25	VT-3, PT	Residual Heat Removal	ISI-RHR-PART 16 Sh.1	**Acceptable**
N/A	ACSP-H201B	VT-3	Standby Gas Treatment	ISI-AC PART 5	**Acceptable**
NNS	HPCI-HD28	VT-3	High Pressure Coolant Injection	ISI-HPCI-PART 4 Sh.1	**Acceptable**
NNS	RHR-HD235	VT-3	Residual Heat Removal	VYI-RHR-PART 7B	**Acceptable**

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Vermont Yankee Nuclear Power Corporation
Vermont Yankee Nuclear Power Station
Owner Certification: DPR-28
Commercial Service Date: 11/30/72

Components Inspected/Abstract of examinations
Sections 7 and 11

Code Category	*Quantity Inspected 2001 Outage*	*Quantity Previously Inspected, Third Interval*	*Quantity Scheduled, Third Interval*	*Percent of Third Interval Complete*
B-A	0	15	16	94%
B-D	10	38	58	83%
B-F	12	19	35	89%
B-G-1	16	152	288	58%
B-G-2	0	77	109	71%
B-J (Code Case N-560 selection)	9	0 (Code Case N-560 was first used for selection during RFO-22)	15	60% (Previously 64% of the standard ASME Category B-J 25% selection had been completed)
B-J (These are ASME Category B-J, Item B9.40 socket welds which are not included in the Code Case N-560 selection. They are selected in accordance with Category B-J 25% criteria.)	2	14	23	70%
B-K	4	3	10	70%
B-L-2	0	0	Per approved Relief Request No. B-1	N/A
B-M-2	0	26	Per approved Relief Request No. B-2	N/A

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Vermont Yankee Nuclear Power Corporation
Vermont Yankee Nuclear Power Station
Owner Certification: DPR-28
Commercial Service Date: 11/30/72

Components Inspected/Abstract of examinations
Sections 7 and 11

Code Category	*Quantity Inspected 2001 Outage*	*Quantity Previously Inspected, Third Interval*	*Quantity Scheduled, Third Interval*	*Percent of Third Interval Complete*
B-N-1	0	2	Each Period	N/A
B-N-2	0	Partial	1	N/A
B-O	2	4	7	86%
C-A	0	3	4	75%
C-B	0	6	8	75%
C-C	4	12	20	80%
C-F-2	20	43	72	88%
D-A	3	6	11	82%
E-A	20%	80%	100%	100%
E-C	100%	100%	100%	100%
E-D				
E-G				
F-A	33	59	119	77%

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Vermont Yankee Nuclear Power Corporation
Vermont Yankee Nuclear Power Station
Owner Certification: DPR-28
Commercial Service Date: 11/30/72

ABSTRACT OF CONDITIONS NOTED/CORRECTIVE MEASURES TAKEN
Sections 12 and 13

Code Category	*Item Identification*	*Conditions Noted and Corrective Measures Taken*
B-G-1	01A-N/W through 16A-N/W	VT-1 examination of Recirculation pump P-18-1A bolting identified possible missing protective thread coating (the bolting was examined in place, under tension). The examination also revealed corrosion on the exposed bolting. Inservice Discrepancy Report # 01-09 was generated to request Engineering evaluation of these indications. Technical Evaluation (TE) 2001-034 was generated and contains: The pump casing cover/body bolting will perform its design function with the as-noted surface conditions. Margin exists in the 2 1/2" diameter cap screws for future corrosion. No additional and/or augmented inspections other than planned inservice inspection is required.
C-C	RHR-H192	VT-3 examination of rigid strut support RHR-H192 revealed a gouge on the pipe clamp. Inservice Discrepancy Report # 01-02 was generated to request Mechanical Design Engineering evaluation of this condition. Technical Evaluation 2001-015 was issued containing: a) The gouge does not extend behind the lugs therefore the lugs have full bearing surface on the clamp. b) The reduction of a maximum of 1/32" of depth on an 8" deep clamp is insignificant (<1%). These indications were determined to be caused during initial installation/fabrication.
D-A	RSW-H172	VT-3 examination of rigid frame support RSW-H172 revealed a crack in the concrete wall adjacent to the base plate. Inservice Discrepancy Report # 01-03 was generated to request Mechanical Design Engineering evaluation of this condition. Technical Evaluation 2001-017 was issued containing: a) The support and the associated anchor bolts will perform their intended design functions in the as-found condition. b) The cracking has been determined to be a surface hairline crack that is the result of normal aging and/or as-expected normal shrinkage cracking of the concrete. This area is monitored in accordance with Vermont Yankee procedure PP 7030 "Structures Monitoring Program" which implements 10 CFR 50.65.

FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS
As Required by the Provisions of the ASME Code rules

Vermont Yankee Nuclear Power Corporation
Vermont Yankee Nuclear Power Station
Owner Certification: DPR-28
Commercial Service Date: 11/30/72

ABSTRACT OF CONDITIONS NOTED/CORRECTIVE MEASURES TAKEN
Sections 12 and 13

Code Category	*Item Identification*	*Conditions Noted and Corrective Measures Taken*
D-A	RSW-H261	VT-3 examination of spring hanger RSW-H261 revealed several arc strikes and a poor weld profile on integrally attached pipe lugs (these lugs are used in common with spring hanger RSW-HD261B - see below) . Inservice Discrepancy Report # 01-01 was generated to request Mechanical Design Engineering evaluation of these conditions. Technical Evaluation 2001-014 was issued containing: a) None of the arc strikes contained cracking b) The maximum recordable depth of any arc strike was .03" c) No overstress conditions exist. d) The weld in question is an "extra weld" not called for in the engineering qualification of the pipe lug (the lug is only required to be welded on 2 sides, this weld is on the third (not required) side. These indications were determined to be caused during initial installation or modification.
D-A	RSW-HD261B	VT-3 examination of spring hanger RSW-HD261B revealed several arc strikes and a poor weld profile on integrally attached pipe lugs (these lugs are used in common with spring hanger RSW-H261 - see above). Inservice Discrepancy Report # 01-01 was generated to request Mechanical Design Engineering evaluation of these conditions. Technical Evaluation 2001-014 was issued containing: a) None of the arc strikes contained cracking b) The maximum recordable depth of any arc strike was .03" c) No overstress conditions exist. d) The weld in question is an "extra weld" not called for in the engineering qualification of the pipe lug (the lug is only required to be welded on 2 sides, this weld is on the third (not required) side. These indications were determined to be caused during initial installation or modification.
E-A	Penetrations X-207A through X-207H	During General Visual examination general corrosion and material loss was found. Inservice Discrepancy Report # 01-08 was generated to request Mechanical Design Engineering evaluation of this condition. Technical Evaluation (TE) 2001-025 was generated and contains: The condition is acceptable as found as there is significant margin remaining to code minimum wall thickness accompanied by a low expected rate of galvanic corrosion in the inerted containment.

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Vermont Yankee Nuclear Power Corporation
Vermont Yankee Nuclear Power Station
Owner Certification: DPR-28
Commercial Service Date: 11/30/72

ABSTRACT OF CONDITIONS NOTED/CORRECTIVE MEASURES TAKEN
Sections 12 and 13

Code Category	*Item Identification*	*Conditions Noted and Corrective Measures Taken*
E-A	Vent Header	During General Visual examination pitting and general corrosion in excess of the allowable values provided by Mechanical Design Engineering were found. The corrosion and pitting are accompanied by loss of coating. There was also significant standing water in Vent Header bowl H. Inservice Discrepancy Report # 01-07 was generated to request Mechanical Design Engineering evaluation of this condition. Technical Evaluation (TE) 2001-025 was generated and contains: a) The observed pitting in the Vent Header is acceptable. b) The standing water was removed and the source was identified and corrected prior to drywell closeout.
F-A	FDW-HD39	VT-3 examination of anchor FDW-HD39 revealed debris in the form of paint chips, insulation and minor corrosion in a required 1/16" gap between a trunion and the base plate. Inservice Discrepancy Report # 01-12 was generated to request Mechanical Design Engineering evaluation of this condition. Technical Evaluation (TE) 2001-038 was generated and contains: The as found condition of the support/anchor is acceptable with the exception of the identified debris. The trunions are not "bound up" restricting thermal growth/movement of the pipe, and the debris does not adversely impact the overall function of the support/anchor. The debris was subsequently cleaned from the anchor.
F-A	RR-44	VT-3 examination of spring hanger RR-44 revealed a spring can setting that was out of tolerance by greater than ± 5% provided by Mechanical Design Engineering. Inservice Discrepancy Report # 01-04 was generated to request Mechanical Design Engineering evaluation of this condition. Technical Evaluation (TE) 2001-021 was generated and contains: The setting was determined to have not affected the supports structural or functional capability. It was noted that the support was "adjusted" as far as possible, i.e., there was no more thread remaining on the rod at the adjustment nut. This condition will be revisited during the next extended refueling outage (RFO-23) to determine if any further action would be warranted.

FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS
As Required by the Provisions of the ASME Code rules

Vermont Yankee Nuclear Power Corporation
Vermont Yankee Nuclear Power Station
Owner Certification: DPR-28
Commercial Service Date: 11/30/72

ABSTRACT OF CONDITIONS NOTED/CORRECTIVE MEASURES TAKEN
Sections 12 and 13

Code Category	*Item Identification*	*Conditions Noted and Corrective Measures Taken*
F-A	RR-52	VT-3 examination of spring hanger RR-52 revealed a spring can setting that was out of tolerance by greater than ± 5% provided by Mechanical Design Engineering. Inservice Discrepancy Report # 01-05 was generated to request Mechanical Design Engineering evaluation of this condition. Technical Evaluation (TE) 2001-022 was generated and contains: The setting was determined to have not affected the supports structural or functional capability. It was noted that the support was "adjusted" as far as possible, i.e., there was no more thread remaining on the rod at the adjustment nut. This condition will be revisited during the next extended refueling outage (RFO-23) to determine if any further action would be warranted.
F-A	RR-7, 8	VT-3 examination of spring hanger RR-7, 8 revealed a spring can setting that was out of tolerance by greater than ± 5% provided by Mechanical Design Engineering. Inservice Discrepancy Report # 01-06 was generated to request Mechanical Design Engineering evaluation of this condition. Technical Evaluation (TE) 2001-023 was generated and contains: a) The setting was determined to have not affected the supports structural or functional capability. b) The spring cans are capable of performing their intended design function in the as-found, as-left condition. This condition will be revisited during the next extended refueling outage (RFO-23) to determine if any further action would be warranted.

Vermont Yankee Nuclear Power Corporation

2001 Form NIS-2 Owner's Summary Report for Repairs or Replacements

December 4, 1999 through May 20, 2001

Reviewed by: _____ 8/2/01
Plant Inservice Inspection Coordinator

Approved by: _____ for S Dreyfus 8/2/01
System Engineering Manager

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As required by the Provisions of the ASME Code Section XI

1. Owner Vermont Yankee Nuclear Power Corporation (Name) Date

185 Old Ferry Road, PO Box 7002, Brattleboro VT 05302-7002 (Address) Sheet 2 of 14

2. Plant Vermont Yankee Nuclear Power Station (Name) Unit 1

P.O. Box 157, Governor Hunt Road, Vernon, VT 05354-0157 (Address) N/A (Repair Organization P.O. No., Job No., etc.)

3. Work Performed by Vermont Yankee Nuclear Power Corporation (Name) Type Code Symbol Stamp N/A

Authorization No. N/A

185 Old Ferry Road, PO Box 7002, Brattleboro VT 05302-7002 (Address) Expiration Date N/A

4. Identification of System See attached table, pages 4 through 14

5. (a) Applicable Construction Code B.31.1 1967 Edition, No Addenda, No Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986 Edition No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components See attached table, pages 4 through 14

7. Description of Work See attached table, pages 4 through 14

8. Tests Conducted See attached table, pages 4 through 14

9. Remarks See attached table, pages 4 through 14

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and these repairs/replacements conform to the rules of ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization Number N/A Expiration Date N/A

Signed D. Leach, Vice President, Engineering Date August 16, 2001

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Vermont and employed by Factory Mutual Insurance Co. of Johnston RI have inspected the components described in this Owner's Report during the period December 4, 1999 to May 20, 2001 and state to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Commissions VT-345
National Board, State, Province, and Endorsements

Date Aug 16 2001

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As required by the provisions of the ASME Code, Section XI, 1986 Edition, No Addenda

Vermont Yankee Nuclear Power Plant Unit 1
P.O. Box 157, Vernon, VT, 05354

Construction Code B31.1, 1967 Edition, No Addenda, No Code Case

Component Equipment Number	*System Identification*	*Name of Manufacturer*	*Manufacturer Serial Number*	*National Board Number*	*Other Identification (Work Order No., Minor Modification, Design Change, etc.)*	*Year Built*	*Repaired, Replaced, or Replacement*	*ASME Code Stamped*	*Description of Work*	*Test Conducted*
P-7-1B	SW	Byron Jackson	691-N-0362	N/A	WO 99-009059-000	1972	Repaired	N/A	Repaired Pump internals	System Leakage
RRU-8	HVAC	H. K. Porter	M-24442	N/A	WO 00-001039-000	1972	Repaired	N/A	Repaired Leak	System inservice
RCW-H88	RBCCW	Plant Fabricated	N/A	N/A	MM 99-05 WO 97-008451-020	1972	Repaired	N/A	Structural Hanger Modifications	N/A - repair made to structural components only
RCW-H89	RBCCW	Plant Fabricated	N/A	N/A	MM 99-05 WO 97-008451-020	1972	Repaired	N/A	Structural Hanger Modifications	N/A - repair made to structural components only
DG-1-1A	DG	Fairbanks Morse	38D870011TDS M12	N/A	MM 2000-001 WO 99-004206-000	1972	Replaced	N/A	Replaced Cooling Water Bellows	System inservice
SFP (Spent Fuel Pool)	SFPC	Plant Fabricated	N/A	N/A	MM 99-064 WO 00-000377-000	1972	Repaired	N/A	Obstacle Removal in Support of Spent Fuel Rack installation	N/A - repair made to structural components only
V13-16	RCIC	Walworth	SMB-00	N/A	WO 00-002526-000 WO 00-001746-002	1972	Repaired	N/A	Replaced Stem and Bonnet, Moved Packing To Repair Leak	System Leakage
SR-10-80B	RHRSW	Consolidated - Dresser	C31419	N/A	WO 00-001935-000	1972	Replaced	N/A	Replaced Relief Valve	System Leakage
SR-10-80A	RHRSW	Consolidated - Dresser	TK43762	N/A	WO 00-001943-000	1972	Replaced	N/A	Replaced Relief Valve	System inservice

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As required by the provisions of the ASME Code, Section XI, 1986 Edition, No Addenda

Vermont Yankee Nuclear Power Plant Unit 1
P.O. Box 157, Vernon, VT, 05354

Construction Code B31.1, 1967 Edition, No Addenda, No Code Case

Component Equipment Number	*System Identification*	*Name of Manufacturer*	*Manufacturer Serial Number*	*National Board Number*	*Other Identification (Work Order No., Minor Modification, Design Change, etc.)*	*Year Built*	*Repaired, Replaced, or Replacement*	*ASME Code Stamped*	*Description of Work*	*Test Conducted*
DG-1-1A	DG	Fairbanks Morse	38D870011TDS M12	N/A	WO 99-004206-002	1972	Repaired	N/A	Performed Weld Repair To Eroded Area	System inservice
VG-9B	CAD	Target Rock	Model # 75E002	N/A	WO 00-002632-000	1972	Repaired	N/A	Rebuilt Valve	System Leakage
3"SW-5E	SW	Plant Fabricated	N/A	N/A	WO 00-003663-000	1972	Replacement	N/A	Replaced Section of 3"SW-5E Piping	System Leakage
V70-1A	SW	Walworth	Model # 5341WE	N/A	WO 96-012700-000	1972	Repaired	N/A	Perform Weld Build Up Valve Body in Hinge Pin Area	System Leakage
TK-3-125-10-19	HCU	Liquidonics	200L-8.2-5	N/A	WO 00-005412-001	1972	Replaced	N/A	Replaced Accumulator Tank	System Functional
TK-3-125-06-35	HCU	General Electric	P/N 921D59G2	N/A	WO 00-005412-000	1972	Replaced	N/A	Replaced Accumulator Tank	System Functional
DG-1-1B	DG	Fairbanks Morse	38D70006TDS M12	N/A	WO 00-005379-000	1972	Repaired	N/A	Machined Eroded Area On Flange Faces.	System Functional
V70-7A	SW	Crane	Cat. 487 1/2	N/A	WO 00-003806-000	1972	Replaced	N/A	Replaced Valve	System Leakage
P-7-1A	SW	Byron Jackson	691-N-0361	N/A	WO 00-004971-001	1972	Repaired	N/A	Rebuilt Pump	System Leakage
V70-101	SW	Walworth	Mod. # 5202WE	N/A	WO 95-005089-000	1972	Replaced	N/A	Replaced Valve	System Leakage
TK-3-125-14-35	HCU	Liquidonics	200L-8.2-5	N/A	WO 00-005489-000	1972	Replaced	N/A	Replaced Accumulator	System Leakage
V70-111A/B	SW	Walworth	Mod. # 5275WE	N/A	MM 2000-01 WO 98-012624-000	1972	Replaced	N/A	Replaced Valves	System Leakage
V11-12A	SLC	Powell	N/A	N/A	WO 00-004934-000	1972	Replaced	N/A	Replaced Valve Bonnet Studs	System Leakage

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
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Vermont Yankee Nuclear Power Plant Unit 1
P.O. Box 157, Vernon, VT, 05354

Construction Code B31.1, 1967 Edition, No Addenda, No Code Case

Component Equipment Number	*System Identification*	*Name of Manufacturer*	*Manufacturer Serial Number*	*National Board Number*	*Other Identification (Work Order No., Minor Modification, Design Change, etc.)*	*Year Built*	*Repaired, Replaced, or Replacement*	*ASME Code Stamped*	*Description of Work*	*Test Conducted*
V13-15	RCIC	Walworth	SMB-00	N/A	WO 96-011053-000	1972	Replaced	N/A	Replaced Valve	System Leakage
LCV-3-33D	CRD	BW/IP	N/A	N/A	WO 99-008912-000	1972	Repair	N/A	Replaced Valve Seats	System Leakage
LCV-3-33C	CRD	BW/IP	N/A	N/A	WO 99-008911-000	1972	Repair	N/A	Replaced Valve Seats	System Leakage
SB-16-19-7B	PCAC	Allis Chalmers	00616-11	N/A	WO 99-011315-000	1972	Replaced	N/A	Replaced Flange Bolts	Tested in accordance with Operations Procedure OP 4202
P-7-1B	SW	Byron Jackson	691-N-0362	N/A	WO 00-004971-001	1972	Repaired	N/A	Rebuilt Pump	System Leakage
V70-1A	SW	Walworth	Mod. # 5341WE	N/A	WO 96-012700-000	1972	Repaired	N/A	Weld Buildup of Valve Body in Hinge Pin Area	System Leakage
DG-1-1A	DG	Fairbanks Morse	38D870011TDS M12	N/A	WO 99-008333-000	1972	Replaced	N/A	Replaced Bolting	System inservice
DG-1-1A	DG	Fairbanks Morse	38D870011TDS M12	N/A	WO 99-009496-000	1972	Replaced	N/A	Replaced Bolting	System inservice
DG-1-1B	DG	Fairbanks Morse	38D70006TDS M12	N/A	WO 99-011236-000	1972	Replaced	N/A	Replaced Support Clamp	N/A - repair made to structural components only

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
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Vermont Yankee Nuclear Power Plant Unit 1
P.O. Box 157, Vernon, VT, 05354

Construction Code B31.1, 1967 Edition, No Addenda, No Code Case

Component Equipment Number	*System Identification*	*Name of Manufacturer*	*Manufacturer Serial Number*	*National Board Number*	*Other Identification (Work Order No., Minor Modification, Design Change, etc.)*	*Year Built*	*Repaired, Replaced, or Replacement*	*ASME Code Stamped*	*Description of Work*	*Test Conducted*
DG-1-1B	DG	Fairbanks Morse	38D70006TDS M12	N/A	WO 99-010257-000	1972	Replaced	N/A	Replaced Support Clamp	N/A - repair made to structural components only
DG-1-1B	DG	Fairbanks Morse	38D70006TDS M12	N/A	WO 99-009229-000	1972	Replaced	N/A	Replaced Bolting	System inservice
DG-1-1B	DG	Fairbanks Morse	38D70006TDS M12	N/A	WO 99-009500-000	1972	Replaced	N/A	Replaced Bolting	System inservice
TK-3-125-22-35	HCU	Liquidonics	200L-8.2-5	N/A	WO 00-006190-000	1972	Replaced	N/A	Replaced Accumulator	System Leakage
TK-3-125-14-31	HCU	Liquidonics	200L-8.2-5	N/A	WO 00-006191-000	1972	Replaced	N/A	Replaced Accumulator	System Leakage
P-7-1A	SW	Byron Jackson	691-N-0361	N/A	WO 00-006381-000	1972	Replaced/ Repaired	N/A	Rebuilt Pump Assembly - Replaced With Spare	System Leakage
S-3-1B	SW	R. P. Adams	106047	N/A	WO 00-006231-000	1972	Repaired	N/A	Opened and Cleaned Strainer	System Leakage
Small Bore Piping at P-18-1A/B Recirc. Pumps	NB	Byron Jackson	671-S-1108	N/A	MM 2000-042 WO 00-001839-000	1972	Repaired	N/A	RBCCW thermal Stress Modifications To Small Bore Piping At P-18-1A/B	System Leakage
SR-16-19-77	N2	Kunkle Valve Co.	L-3072	N/A	WO 00-000596-000	1972	Repaired/ Replaced	N/A	Repaired Relief Valve - Replaced With Spare	System Leakage
V11-41	SLC	Powell	Mod. # 3003 WE	N/A	WO 00-006999-000	1972	Replaced	N/A	Replaced Bonnet Studs and Nuts	System Leakage

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Vermont Yankee Nuclear Power Plant Unit 1
P.O. Box 157, Vernon, VT, 05354

Construction Code B31.1, 1967 Edition, No Addenda, No Code Case

Component Equipment Number	*System Identification*	*Name of Manufacturer*	*Manufacturer Serial Number*	*National Board Number*	*Other Identification (Work Order No., Minor Modification, Design Change, etc.)*	*Year Built*	*Repaired, Replaced, or Replacement*	*ASME Code Stamped*	*Description of Work*	*Test Conducted*
RV-10-210A/B	RHR	Consolidated Dresser	Mod. # 1685	N/A	MM 2000-043 WO 00-006249-000	1972	Repaired	N/A	Removed Valves Rv-10-210a/B	System Leakage
SR-10-80 A&B	RHRSW	Consolidated Dresser	TK43762	N/A	WO 00-001935-004 WO 00-001943-005 WO 00-007021-000	1972	Repaired	N/A	Rebuilt Safety Relief Valve	System Leakage
CRD-06-31	CRD	General Electric	Mod. # 7RDB144BG1	N/A	WO 00-004225-002	1972	Replaced	N/A	Replaced Control Rod Drive	System Leakage
CRD-06-11	CRD	General Electric	Mod. # 7RDB144BG1	N/A	WO 00-004225-003	1972	Replaced	N/A	Replaced Control Rod Drive	System Leakage
CRD-14-31	CRD	General Electric	Mod. # 7RDB144BG1	N/A	WO 00-004225-004	1972	Replaced	N/A	Replaced Control Rod Drive	System Leakage
CRD-42-27	CRD	General Electric	Mod. # 7RDB144BG1	N/A	WO 00-004225-005	1972	Replaced	N/A	Replaced Control Rod Drive	System Leakage
CRD-18-39	CRD	General Electric	Mod. # 7RDB144BG1	N/A	WO 00-004225-006	1972	Replaced	N/A	Replaced Control Rod Drive	System Leakage
CRD-26-15	CRD	General Electric	Mod. # 7RDB144BG1	N/A	WO 00-004225-007	1972	Replaced	N/A	Replaced Control Rod Drive	System Leakage
CRD-34-31	CRD	General Electric	Mod. # 7RDB144BG1	N/A	WO 00-004225-008	1972	Replaced	N/A	Replaced Control Rod Drive	System Leakage
CRD-34-39	CRD	General Electric	Mod. # 7RDB144BG1	N/A	WO 00-004225-009	1972	Replaced	N/A	Replaced Control Rod Drive	System Leakage
P-45-1A	SLC	Union Pump Co.	P-C274713	N/A	WO 99-009881-000	1972	Replaced	N/A	Replaced Stuffing Box Studs and Nuts	System Functional
P-45-1A	SLC	Union Pump Co.	P-C274713	N/A	WO 00-006269-000	1972	Replaced	N/A	Replaced Cylinder Flange Tie Studs and Nuts	System Functional
P-45-1B	SLC	Union Pump Co.	P-C274714	N/A	WO 98-011881-000	1972	Replaced	N/A	Replaced Stuffing Box Studs and Nuts	System Functional

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P-45-1B	SLC	Union Pump Co.	P-C274714	N/A	WO 00-006266-000	1972	Replaced	N/A	Replaced Cylinder Flange Tie Studs and Nuts	System Functional
TK-3-125-18-43	HCU	General Electric	P/N 921D595G2	N/A	WO 00-005707-000	1972	Replaced	N/A	Replaced HCU Piston Accumulator	System Leakage
TK-3-125-02-27	HCU	Liquidonics	Mod. # 200L-8.2-5	N/A	WO 00-005708-000	1972	Replaced	N/A	Replaced HCU Piston Accumulator	System Leakage
SR-10-86A	RHR	Dresser	Mod. # 9352774	N/A	WO 97-002364-000	1972	Replaced	N/A	Replaced Valve	System Leakage
HPCI-HD103FN (Snubber S/N ADH-301-1597 removed for functional testing and returned to stock S/N ADH-301-1598 installed)	HPCI	Anchor Darling	ADH-301-1598	N/A	WO 00-001027-000	1972	Repaired/ Replaced	N/A	Replaced and Rebuilt Snubber	N/A - replacement of snubber only
RHR-H185 (Snubber S/N 32198 removed for functional testing and returned to stock S/N 26351 installed)	RHR	Grinnell	Fig. 200 S/N 32198	N/A	WO 00-000995-000	1972	Repaired/ Replaced	N/A	Replaced and Rebuilt Snubber	N/A - replacement of snubber only
RR-3 (Snubber S/N 32197 removed for functional testing and returned to stock S/N 26347 installed)	NB	Grinnell	Miller Model	N/A	WO 00-000993-000	1972	Repaired/ Replaced	N/A	Replaced and Rebuilt Snubber	N/A - replacement of snubber only
CS-HD54A (Snubber S/N 32195 removed for functional testing and returned to stock S/N 26348 installed)	CS	Miller	Fig. 201	N/A	WO 00-000991-000	1972	Repaired/ Replaced	N/A	Replaced and Rebuilt Snubber	N/A - replacement of snubber only

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RHR-H197A (Snubber S/N 32196 removed for functional testing and returned to stock S/N 26349 installed)	RHR	Lynair	Fig. 200	N/A	WO 00-000989-000	1972	Repaired/ Replaced	N/A	Replaced and Rebuilt Snubber	N/A - replacement of snubber only
RR-35 (Snubber S/N 322003 removed for functional testing and returned to stock S/N 30034 installed)	NB	Miller	Fig. 200	N/A	WO 00-000906-000	1972	Repaired/ Replaced	N/A	Replaced and Rebuilt Snubber	N/A - replacement of snubber only
MSSRV S/N 249	NB	Target Rock	249	N/A	PO VY009397	1972	Repaired/ Replaced	N/A	Replaced/Rebuilt Main Steam Safety Relief Valve	System Leakage
MSSRV S/N 250	NB	Target Rock	250	N/A	PO VY009397	1972	Repaired/ Replaced	N/A	Replaced/Rebuilt Main Steam Safety Relief Valve	System Leakage
MSSRV S/N 67-HH-14	NB	Target Rock	67-HH-14	N/A	PO VY009397	1972	Repaired/ Replaced	N/A	Replaced/Rebuilt Main Steam Safety Relief Valve	System Leakage
MSSRV S/N BL 1134	NB	Target Rock	BL 1134	N/A	PO VY009397	1972	Repaired/ Replaced	N/A	Replaced/Rebuilt Main Steam Safety Relief Valve	System Leakage
MSSRV S/N BL-1137	NB	Target Rock	BL-1137	N/A	PO VY009397	1972	Repaired/ Replaced	N/A	Replaced/Rebuilt Main Steam Safety Relief Valve	System Leakage
Alternate Cooling System to Standby Fuel Pool Cooling System	SFPC	Plant Fabricated	N/A	N/A	VYDC 2000-024 WO 00-005772-000	1972	Repaired	N/A	Installed Alternate Cooling System To Standby Fuel Pool Cooling System Design Change	Hydrostatic and System leakage

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NG-13A	CAD	Target Rock	Mod. # 75E001 S/N	N/A	WO 00-004690-000	1972	Repaired	N/A	Valve internals inspection and Bonnet Tack Weld	
NG-13B	CAD	Target Rock	Mod. # 75E001 S/N 3	N/A	WO 00-004691-000	1972	Repaired	N/A	Valve internals inspection and Bonnet Tack Weld	System Leakage
NG-11B	CAD	Target Rock	Mod. # 75E001 S/N 3	N/A	WO 00-004687-000	1972	Repaired	N/A	Valve internals inspection and Bonnet Tack Weld	System Leakage
NG-12B	CAD	Target Rock	Mod. # 75E001 S/N 4	N/A	WO 00-004689-000	1972	Repaired	N/A	Valve internals inspection and Bonnet Tack Weld	System Leakage
NG-12A	CAD	Target Rock	Mod. # 75E001 S/N 2	N/A	WO 00-004688-000	1972	Repaired	N/A	Valve internals inspection and Bonnet Tack Weld	System Leakage
NG-11A	CAD	Target Rock	Mod. # 75E001	N/A	WO 00-004393-000	1972	Repaired	N/A	Valve internals inspection and Bonnet Tack Weld	System Leakage
P-8-1D	RHRSW	Byron Jackson	Mod. # VTP S/N 691-N-0366	N/A	WO 01-001098-000	1972	Replaced	N/A	Replaced Pump Rotating Assembly	System Leakage
DG-1-1A	DG	Fairbanks Morse	38D870011TDS M12	N/A	WO 01-000804-000	1972	Replaced	N/A	Replaced the Collar Stud Assembly	System inservice
DG-1-1B	DG	Fairbanks Morse	38D70006TDS M12	N/A	WO 01-001101-000	1972	Replaced	N/A	Replaced Broken OCS Scavenging Air Piping Stud	System inservice
SR-72-3A	SA	Kunkle Valve Co.	N/A	N/A	WO 00-000597-000	1972	Replaced	N/A	Replaced Relief Valve	System Leakage
RV-2-71A	NB	Target Rock	Mod. # 67F-000-15 6X10	N/A	WO 00-004226-000	1972	Replaced	N/A	Replaced Relief Valve	System Leakage

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RV-2-71B	NB	Target Rock	Mod. # 67F-000-15 6X10	N/A	WO 00-004720-000	1972	Replaced	N/A	Replaced Relief Valve	System Leakage
RV-2-71C	NB	Target Rock	Mod. # 67F-000-15 6X10	N/A	WO 00-004721-000	1972	Replaced	N/A	Replaced Relief Valve	System Leakage
RV-2-71D	NB	Target Rock	Mod. # 67F-000-15 6X10	N/A	WO 00-004722-000	1972	Replaced	N/A	Replaced Relief Valve	System Leakage
SV-2-70A	NB	Dresser	Mod. # 3707 RA-RT21 S/N BL1137	N/A	WO 00-004230-000	1972	Replaced	N/A	Replaced Safety Relief Valve	System Leakage
SV-2-70B	NB	Dresser	Mod. # 3707 RA-RT21 S/N BL1134	N/A	WO 00-004745-000	1972	Replaced	N/A	Replaced Safety Relief Valve	System Leakage
V70-71C	RBCCW	Honeywell	Mod. # 8105	N/A	WO 00-007152-000	1972	Repaired	N/A	Repaired Plug and Stem	System Leakage
V2-80D	MS	Rockwell	Mod. # 1612JMMY S/N 123	N/A	WO 01-001729-000	1972	Repaired	N/A	Repaired Valve Seat	System Leakage
V14-13A	CS	Rockwell	Mod. # 770 JMMY	N/A	WO 01-001806-000	1972	Repaired	N/A	Repaired Valve Internals	System Leakage
P-18-1B	RBCCW	Byron Jackson	Mod. # DVSS S/N 671-S-1109	N/A	WO 00-001839-000	1972	Repaired/ Replaced	N/A	Repaired/ Replaced Spool on Seal Heat Exchanger Cooling Unit	System Leakage

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V16-19-5G	PCAC	Atwood and Morril	Mod. # 20751H	N/A	WO 00-004108-001	1972	Repaired	N/A	Installed Disc Nut Spacer Shim	Tested in accordance with Operations Procedure OP 4202
V3-114-38-35	HCU	General Electric	N/A	N/A	WO 01-001886-000	1972	Repaired	N/A	Repaired Valve Internals	System Leakage
CV-3-127-38-35	HCU	Hammel-Dahl	Mod. # 2500ASA-999Z1204	N/A	WO 01-001886-001	1972	Repaired	N/A	Replaced Teflon Seat Ring Disc	System Leakage
V13-131	RCIC	Walworth	Mod. # C44099 S/N 5301BSB-WE	N/A	WO 00-006789-000	1972	Repaired	N/A	Repaired Valve Internals	System Leakage
V70-319B	SW	Nibco	Fig. T-134	N/A	WO 01-001934-000	1972	Replaced	N/A	Replaced Valve	System Leakage
V70-319D	SW	Nibco	Fig. T-134	N/A	WO 01-001935-000	1972	Replaced	N/A	Replaced Valve	
V13-6	RCIC	Enertech	Mod. # DRV-2	N/A	WO 01-001732-000	1972	Repaired	N/A	Replaced Valve Spring	System Leakage
V13-7	RCIC	Enertech	Mod. # DRV-2	N/A	WO 01-001733-000	1972	Repaired	N/A	Replaced Valve Spring	System Leakage
V23-3	HPCI	Enertech	Mod. # DRV-B	N/A	WO 01-001740-000	1972	Repaired	N/A	Replaced Valve Spring	System Leakage
V23-4	HPCI	Enertech	Mod. # DRV-B	N/A	WO 01-001748-000	1972	Repaired	N/A	Replaced Valve Spring	System Leakage
RRU-8	HVAC	H. K. Porter	Mod. # 41-523-H S/N M-24442	N/A	WO 00-001039-010	1972	Repaired	N/A	Repaired Leak in Service Water Supply Connection	System Leakage

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Drywell Seal and Coating	Drywell	CBI	Mod. # General Electric Mark I	N/A	MM 2000-010 WO 00-001840-000	1972	Repaired/ Replaced	N/A	Replaced DW Seal and Protective Coating Repairs	N/A