

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401  
400 Chestnut Street Tower II

WB RD-50-390/84-50

3 November 28, 1984  
1:22

U.S. Nuclear Regulatory Commission  
Region II

Attn: Mr. James P. O'Reilly, Regional Administrator  
101 Marietta Street, NW, Suite 2900  
Atlanta, Georgia 30323

Dear Mr. O'Reilly:

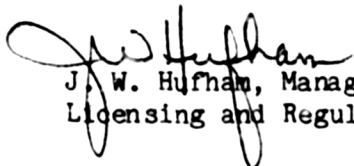
WATTS BAR NUCLEAR PLANT UNIT 1 - RELIEF VALVE THRUST FORCES ASSUMED TO BE  
NEGLIGIBLE - WB RD-50-390/84-50 - FIRST INTERIM REPORT

The subject deficiency was initially reported to NRC-OIE Inspector  
P. E. Fredrickson on October 30, 1984 in accordance with 10 CFR 50.55(e)  
as NCR WBN CEB 8420. Enclosed is our first interim report. We expect to  
submit our next report on or about January 11, 1985.

If you have any questions, please get in touch with R. H. Shell at  
FTS 858-2688.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

  
J. W. Hufham, Manager  
Licensing and Regulations

Enclosure

cc (Enclosure):

Mr. Richard C. DeYoung, Director  
Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Records Center  
Institute of Nuclear Power Operations  
1100 Circle 75 Parkway, Suite 1500  
Atlanta, Georgia 30339

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ENCLOSURE

WATTS BAR NUCLEAR PLANT UNIT 1  
RELIEF VALVE THRUST FORCES ASSUMED TO BE NEGLIGIBLE  
NCR WBN CLB 8420  
WBRD-50-390/84-50  
10 CFR 50.55(e)  
FIRST INTERIM REPORT

Description of Deficiency

While performing the unit 2 analysis of the pipes which connect the relief valves to each accumulator tank in the safety injection system, the analyst discovered that the unit 1 supports for these pipes were inadequate. The analyst was reviewing the unit 1 analysis while performing the analysis for unit 2. The unit 1 alternate analysis was being checked to assure that relief valve forces were being properly considered for unit 2. It has been determined from the unit 1 review that a significant static thrust force of 290 pounds was assumed to be negligible by the unit 1 analyst when alternately analyzing the piping which connects relief valves 1-8855A, 1-8855B, 1-8855C and 1-8855D to the accumulator tanks. This oversight caused an overstress when considering the upset and emergency conditions which violates ASME Code equations 9U and 9E. The alternate analysis problems for unit 1 which were affected by the oversight are N3-63-A01R, -A02R, -A03R, and -A04R.

Interim Progress

TVA is reanalyzing alternate analysis problems N3-63-A01R, -A02R, -A03R, and -A04R which were affected by this unit 1 analysis oversight. Additional information will be provided in our next report.