

FENOC

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10 CFR 50.59(d)(2)

ATTN: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

SUBJECT:
Beaver Valley Power Station, Unit No. 1
Docket No. 50-334, License No. DPR-66
Report of Facility Changes, Tests and Experiments

In accordance with 10 CFR 50.59(d)(2), the Report of Facility Changes, Tests, and Experiments for the Beaver Valley Power Station Unit No. 1 is provided as Attachment 1. The report covers the period of April 20, 2006, through October 24, 2007.

There are no regulatory commitments contained in this letter. If there are any questions or if additional information is required, please contact Mr. Thomas A. Lentz, Manager – FENOC Fleet Licensing, at (330) 761-6071.

Sincerely,



Peter P. Sena III

Attachment: Report of Facility Changes, Tests and Experiments

c: Mr. S. J. Collins, NRC Region I Administrator
Mr. D. L. Werkheiser, NRC Senior Resident Inspector
Ms. N. S. Morgan, NRR Project Manager
Mr. D. J. Allard, Director BRP/DEP
Mr. L. E. Ryan (BRP/DEP)

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Report of Facility Changes, Tests and Experiments

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Change Title

Implementation of Reduced Atmospheric Steam Dump Valve Capacity

Change

The Beaver Valley Power Station Unit No. 1 Steam Generator Tube Rupture (SGTR) analysis was updated to address changes as a result of the Extended Power Uprate (EPU) analyses. These EPU analyses used atmospheric steam dump and residual heat release valve capacities without accounting for the impact of the valve inlet and outlet piping. The following impacts were evaluated:

- Main Steam Atmospheric Dump valve interface criteria
- Steam generator margin to overflow
- Dose (increase due to extended cooldown times to residual heat removal system entry conditions at 350°F)

The evaluation of the above impacts concluded that the BV1 SGTR analysis could be updated for implementation of EPU without submittal of a license amendment request.

Change Title

Containment Sump Strainer

Change

This change replaced the existing containment sump screens with a new containment sump strainer to address new regulatory requirements as a result of NRC Generic Letter 04-02, dated September 13, 2004, "Potential Impact of Debris Blockage on Emergency Recirculation During Design Basis Accidents at Pressurized-Water Reactors". It analyzed the use of a common strainer to provide protection for both trains of Recirculation Spray and Low Head Safety Injection systems and the inability of the wide range level transmitters to monitor the containment water level from the bottom of containment for a short period of time post accident. The evaluation concluded that this design is acceptable without submittal of a license amendment request.