

September 17, 2008

Mr. David A. Christian  
President and Chief Nuclear Officer  
Virginia Electric and Power Company  
Innsbrook Technical Center  
5000 Dominion Boulevard  
Glen Allen, VA 23060-6711

SUBJECT: SURRY POWER STATION, UNIT NOS. 1 AND 2, ISSUANCE OF AMENDMENTS REGARDING THE REVISION TO VARIOUS SETTING LIMITS AND THE OVERTEMPERATURE  $\Delta T$ /OVERPOWER  $\Delta T$  TIME CONSTANTS (TAC NOS. MD6812 AND MD6813)

Dear Mr. Christian:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 261 to Renewed Facility Operating License No. DPR-32 and Amendment No. 261 to Renewed Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2 (Surry 1 and 2), respectively. The amendments change the Technical Specifications (TSs) in response to your application dated September 19, 2007, as supplemented by letter dated April 11, 2008.

These amendments revise various TS setting limits and the overtemperature  $\Delta T$ /overpower  $\Delta T$  time constants in TS 2.3 and TS 3.7 for Surry 1 and 2.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

**/RA/**

Siva P. Lingam, Project Manager  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-280 and 50-281

Enclosures:

1. Amendment No. 261 to DPR-32
2. Amendment No. 261 to DPR-37
3. Safety Evaluation

cc w/encls: See next page

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Package No.: ML082250008  
Tech Spec No.: ML082250026

Amendment No.: ML082250013  
\*\*concurring by telephone      \*transmitted by memo dated

OFFICE	NRR/LPL2-1/PM	NRR/LPL2-1/LA	DE/EICB/BC	DIRS/ITSB/BC	DSS/SRXB/BC	OGC w/ comments	NRR/LPL2-1/BC
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VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 261  
Renewed License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated September 19, 2007, as supplemented by letter dated April 11, 2008, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Renewed Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 261, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

**/RA**

Melanie C. Wong, Chief  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to License No. DPR-32  
and the Technical Specifications

Date of Issuance: September 17, 2008

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 261  
Renewed License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated September 19, 2007, as supplemented by letter dated April 11, 2008, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Renewed Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 261, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

Melanie C. Wong, Chief  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes License No. DPR-37  
and the Technical Specifications

Date of Issuance September 17, 2008

ATTACHMENT

TO LICENSE AMENDMENT NO. 261

RENEWED FACILITY OPERATING LICENSE NO. DPR-32

DOCKET NO. 50-280

AND

TO LICENSE AMENDMENT NO. 261

RENEWED FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NO. 50-281

Replace the following pages of the Licenses and the Appendix A Technical Specifications (TSs) with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages

License

License No. DPR-32, page 3  
License No. DPR-37, page 3

TSs

2.2-2  
2.3-1  
2.3-2  
2.3-3  
2.3-4  
2.3-5  
2.3-6  
2.3-7  
2.3-8  
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3.1-2  
3.1-4a  
3.7-4  
3.7-6  
3.7-7  
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3.7-10  
3.7-11  
3.7-12  
3.7-14

Insert Pages

License

License No. DPR-32, page 3  
License No. DPR-37, page 3

TSs

2.2-2  
2.3-1  
2.3-2  
2.3-3  
2.3-4  
2.3-5  
2.3-6  
2.3-7  
2.3-8  
2.3-9  
3.1-2  
3.1-4a  
3.7-4  
3.7-6  
3.7-7  
3.7-7a  
3.7-10  
3.7-11  
3.7-12  
3.7-14

ATTACHMENT

TO LICENSE AMENDMENT NO. 261

RENEWED FACILITY OPERATING LICENSE NO. DPR-32

DOCKET NO. 50-280

AND

TO LICENSE AMENDMENT NO. 261

RENEWED FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NO. 50-281

Remove Pages (cont'd)

TSs (cont'd)

3.7-18  
3.7-19  
3.7-20  
3.7-21  
3.7-22  
3.7-25  
3.7-26  
4.1-8b

Insert Pages (cont'd)

TSs (cont'd)

3.7-18  
3.7-19  
3.7-20  
3.7-21  
3.7-22  
3.7-25  
3.7-26  
4.1-8b



SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 261 TO

RENEWED FACILITY OPERATING LICENSE NO. DPR-32

AND

AMENDMENT NO. 261 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-280 AND 50-281

1.0 INTRODUCTION

By letter dated September 19, 2007 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML072681096), as supplemented by letter dated April 11, 2008 (ADAMS Accession No. ML081021195), Virginia Electric and Power Company (the licensee) submitted a request for changes to the Surry Power Station, Unit Nos. 1 and 2 (Surry 1 and 2), Technical Specifications (TSs). The requested changes would revise various TS setting limits and the overtemperature  $\Delta T$ /overpower  $\Delta T$  time constants in TS 2.3 and TS 3.7 for Surry 1 and 2. The methodology for determining the revised setting limits and time constants is in agreement with methods 1 and 2 in The Instrumentation, Systems, and Automation Society's (ISA's) Standard ISA-R67.04, Part II, "Methodologies for the Determination of Setpoints for Nuclear Safety-Related Instrumentation." The supplement dated April 11, 2008, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the U.S. Nuclear Regulatory Commission (NRC) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on October 23, 2007 (72 FR 60036). The proposed changes are summarized below:

1.1 Changes to the Setting Limits

Source Range Neutron Flux High Reactor Trip: Changed the allowable value (AV) from  $\leq 10^6$  counts/second to  $\leq 1.51 \times 10^5$  counts/second.

Pressurizer Low-Pressure Reactor Trip: Changed the AV from  $\geq 1860$  pounds per square inch gauge (psig) to  $\geq 1875$  psig.

Pressurizer High-Pressure Reactor Trip: Changed the AV from  $\leq 2385$  psig to  $\leq 2380$  psig.

Reactor Coolant Flow Low Reactor Trip: Changed the AV from  $\geq 90\%$  of normal indicated flow to  $\geq 91\%$  of normal indicated flow.

Pressurizer High-Level Reactor Trip: Changed the AV from  $\leq 92\%$  of span to  $\leq 89.12\%$  of span.

Enclosure

Steam Generator Water Level Low-Low Reactor Trip Safety Injection (SI): Changed the AV from  $\geq 14.5\%$  of narrow-range instrument span to  $\geq 16.0\%$  of narrow-range instrument span.

Steam Generator Water Level Low Coincident Reactor Trip: Changed the AV from  $\geq 15\%$  of narrow-range instrument span to  $\geq 19\%$  of narrow-range instrument span.

Permissive P-7, Unblock High-Power Reactor Trips: Changed the AV from  $\geq 10\%$  of rated power to before or when power increases to 11% of rated power.

Permissive P-10, Unblock Low-Power Reactor Trips: Changed the AV from  $\leq 10\%$  of rated power to before or when power decreases to 7% of rated power.

Permissive P-8, Power Range Neutron Flux: Changed the AV from  $\geq 50\%$  of rated power to before or when the power range nuclear flux increases to 37% of rated power.

Source Range High Flux: Changed the AV from  $\leq 5 \times 10^{-11}$  amperes to before or when the intermediate-range neutron flux decreases to  $5 \times 10^{-11}$  amperes.

Containment Pressure—High: Changed the AV from  $\leq 19$  pounds per square inch absolute (psia) to  $\leq 18.5$  psia.

Containment Pressure—High-High: Changed the AV from  $\leq 25$  psia to  $\leq 24$  psia.

Pressurizer Pressure—Low-Low: Changed the AV from  $\geq 1760$  psig to  $\geq 1770$  psig.

High Differential Pressure between Steam Line and Steam Line Header: Changed the AV from  $\leq 150$  psig to  $\leq 135$  psig.

Steam Line Pressure—Low: Changed the AV from  $\geq 500$  psig to  $\geq 510$  psig.

Low Intake Canal Level: Changed the AV from 23 feet and 6 inches to 23 feet and 5.85 inches.

Steam Generator Water Level—High-High: Changed the AV from  $\leq 80\%$  of narrow range to  $\leq 76\%$  of narrow range.

Refueling Water Storage Tank Level (RWST)—Low (Unit 1) Initiation of Recirculation: Changed the AV from  $\geq 11.25\%$  to  $\geq 12.7\%$ .

RWST Level—Low (Unit 1) Mode Transfer System: Changed the AV from  $\leq 15.75\%$  to  $\leq 14.3\%$ .

Pressurizer Pressure, P-11: Changed the AV from  $\leq 2000$  psig to  $\leq 2010$  psig.

$T_{avg}$ , P-12: Changed the AV from  $\leq 543$  °F to  $\leq 545$  °F.

In TS Section 2.3, the licensee changed the time constants for overtemperature  $\Delta T$  and overpower  $\Delta T$  as follows:

$\tau_1$ : Changed the AV from = 25 seconds to  $\geq 29.7$  seconds.

$\tau_2$ : Changed the AV from = 3 seconds to  $\leq 4.4$  seconds.

$\tau_3$ : Changed the AV from = 10 seconds to  $\geq 9.0$  seconds.

## 1.2 Addition of Explanation to Technical Specification Section 2.3

The licensee has added an explanation to the overtemperature  $\Delta T$  and overpower  $\Delta T$  specification, which states that "The channel's maximum Trip Setpoint shall not exceed its computed Trip Setpoint by more than 2.0% of the  $\Delta T$  span. (Note that 2.0% of the  $\Delta T$  span is equal to 3.0%  $\Delta T$  Power.)"

## 1.3 Addition of Footnote to Technical Specification Tables 3.7.1, 3.7.2, 3.7.3, and 3.7.4

The licensee has added the following footnote to TS table 3.7.1:

There is a Safety Analysis Limit associated with this Reactor Trip function. If during calibration the setpoint is found to be conservative with respect to the Limiting Safety System Setting but outside its predefined calibration tolerance, then the channel shall be brought back to within its predefined calibration tolerance before returning the channel to service. The calibration tolerances are specified in a document controlled under 10 CFR 50.59.

The licensee has added the following footnote to TS tables 3.7.2, 3.7.3, and 3.7.4:

There is a Safety Analysis Limit associated with this [Engineered Safety Feature] ESF function. If during calibration the setpoint is found to be conservative with respect to the Setting Limit but outside its predefined calibration tolerance, then the channel shall be brought back to within its predefined calibration tolerance before returning the channel to service. The calibration tolerances are specified in a document controlled under 10 CFR 50.59.

## 2.0 REGULATORY EVALUATION

The following regulatory bases and guidance documents pertain to the proposed TS change:

- Title 10 of the *Code of Federal Regulations* (10 CFR), Part 50, Section 50.36(d)(1)(ii)(A), states, "Limiting safety system settings for nuclear reactors are settings for automatic protective devices related to those variables having significant safety functions. Where a limiting safety system setting is specified for a variable on which a safety limit has been placed, the setting must be so chosen that automatic protective action will correct the abnormal situation before a safety limit is exceeded. If, during operation, it is determined that the automatic safety function does not function as required, the licensee shall take corrective action, which may include shutting down the reactor."
- Regulatory Guide (RG) 1.105, Revision 3, "Setpoints for Safety-Related Instrumentation," issued December 1999, describes a method acceptable to the NRC staff for complying

with the NRC regulations for ensuring that setpoints for safety-related instrumentation are initially within and remain within the TS limits.

- Regulatory Issue Summary (RIS) 2006-17, "NRC Staff Position on the Requirements of 10 CFR 50.36, 'Technical Specifications,' Regarding Limiting Safety System Settings During Periodic Testing and Calibration of Instrument Channels," dated August 24, 2006. This RIS discusses the requirements of 10 CFR 50.36 and provides one approach that the NRC staff has found acceptable for identifying Limiting Safety System Settings during period testing and calibration of instrumentation guidance.

### 3.0 TECHNICAL EVALUATION

#### 3.1 Changes to the Setting Limits

By letter dated September 19, 2007, the licensee included the revised Technical Report EE-0116, "Allowable value for North Anna Improved Technical Specifications (ITS), Tables 3.3.1-1 and 3.3.2-1 and Setting Limits for Surry Custom Technical Specifications (CTS), Sections 2.3 and 3.7," Rev. 3, to bring Surry 1 and 2 methodology for determining AV into agreement with methods 1 and 2 in ISA Standard ISA-RP67.04.02. This revised technical report was submitted in response to NRC staff concerns identified in RIS 2006-17 about the instrument setpoint methodology, particularly method 3 as defined in ISA Standard ISA-RP67.04.02, "Methodologies for the Determination of Setpoints for Nuclear Safety-Related Instrumentation." The NRC staff found that method 3 did not meet the requirements of 10 CFR 50.36(d)(1)(ii)(A) with respect to instrument operability determination and could exceed the AL if instrument setpoint was not reset within the acceptable as left band after each test. In addition, the licensee, in its September letter, stated that the analytical limit (AL) will be protected if all three conditions are satisfied:

1. The distance between the actual trip setpoint and the AL is equal to or greater than the total loop uncertainty (TLU) for that channel.
2. The distance between the actual AV and the AL is equal to or greater than the non-channel operational test (non-COT) error components of the TLU.
3. The distance between the actual trip setpoint and the actual AV is equal to the COT error components of the TLU without the inclusion of any excessive margin.

The NRC staff in its request for additional information questioned the licensee's justification for the excessive margin statement in the third item. In its response, the licensee revised the sentence to read, "The distance between the Actual Trip Setpoint and the Actual Allowable Value is equal to the COT error components of the Total Loop Uncertainty. The Allowable Value for certain functions may be rounded to a whole number that remains bounded by the available Safety Margin."

The licensee has calculated the instrument setpoints based on this new approach and revised the setting limits for the instrument functions identified in Section 1.0 of this safety evaluation. The licensee's setpoint methodology determines the AV based on the uncertainties included only for COT and does not include any additional margin except rounding off to a whole number. Therefore, AV is the same as the acceptable as-found value defined in RIS 2006-17. The

licensee has defined the acceptable as-left value equal to the calibration tolerance value which is more conservative than that defined in RIS 2006-17. Therefore, the NRC staff finds that the licensee has addressed the NRC staff concerns with respect to operability and protecting the AL. On this basis, the NRC staff concludes that the licensee setpoint methodology meets the guidance provided in RIS 2006-17. The licensee has revised the affected TS Sections 2.3, 3.1, and 4.1-1 and TS Tables 3.7-1, 3.7-2, and 3.7-4 where these settings are listed. The NRC staff has reviewed the instrument setpoint calculation contained in the Technical Report EE-0116 and determined that these settings are calculated based on the acceptable ISA methodology, and therefore, the NRC staff finds them acceptable.

### 3.2 Addition of Explanation to Technical Specification Section 2.3

The licensee has added an explanation to TS Section 2.3, which states that “The channel’s maximum Trip Setpoint shall not exceed its computed Trip Setpoint by more than 2.0% of the  $\Delta T$  span. (Note that 2.0% of the  $\Delta T$  span is equal to 3.0%  $\Delta T$  Power.)” The licensee stated that since the overpower  $\Delta T$  and overtemperature  $\Delta T$  reactor trip setpoint is variable and is constantly calculated based on actual plant conditions, the AV (maximum trip setpoint) cannot be expressed as a constant. The licensee also stated that AV of 2.0 percent of the  $\Delta T$  span is consistent with the original design basis for this function. The NRC staff reviewed this change and found that because this is more conservative with respect to the Channel Statistical Allowance (CSA) calculation assumptions, the proposed addition to the footnote is acceptable.

### 3.3 Addition of Footnote to Technical Specification Tables 3.7-1, 3.7-2, 3.7-3, and 3.7-4

The licensee has added the following footnote to TS Table 3.7-1:

There is a Safety Analysis Limit associated with this Reactor Trip function. If during calibration the setpoint is found to be conservative with respect to the Limiting Safety System Setting but outside its predefined calibration tolerance, then the channel shall be brought back to within its predefined calibration tolerance before returning the channel to service. The calibration tolerances are specified in a document controlled under 10 CFR 50.59.

The licensee has applied this footnote to the following functions in TS Table 3.7-1:

- Functional Unit 2 Neutron Flux Power Range
- Functional Unit 3 Neutron Flux Intermediate Range
- Functional Unit 4 Neutron Flux Source Range
- Functional Unit 5 Overtemperature  $\Delta T$
- Functional Unit 7 Low Pressurizer Pressure
- Functional Unit 8 High Pressurizer Pressure
- Functional Unit 9 Pressurizer-Hi Water Level
- Functional Unit 10 Low Flow [Reactor Coolant]
- Functional Unit 12 Lo-Lo Steam Generator Water Level
- Functional Unit 20.c Power range neutron flux, P-8

The NRC staff finds that the proposed note by the licensee will add operability determination of the subject functions in the TS as required by RIS 2006-17. However, the NRC staff found that the

licensee should add this footnote, or justify why this footnote is not needed for two additional functions, Functional Unit 6-Overpower  $\Delta T$ , and Functional Unit 17-Low steam generator water level with steam/feedwater flow mismatch. The NRC staff required this information to conclude that these functions are not safety limit related functions and therefore, the above footnote is not needed. In response to the NRC staff's concern, the licensee justified the exclusion on the basis that neither of these functions has been credited in the safety analysis and proposed to add another note for consistency in the TS in its supplemental letter dated April 11, 2008. This note reads as follows:

If during calibration the setpoint is found to be conservative with respect to the Limiting Safety System Setting but outside its predefined calibration tolerance, then the channel shall be brought back to within its predefined calibration tolerance before returning the channel to service. The calibration tolerances are specified in a document controlled under 10 CFR 50.59.

The NRC staff finds that this footnote achieves the same requirements as for safety limit related function and identifies that the function is not safety limit related. Therefore, the NRC staff finds the licensee's proposed change acceptable.

The licensee has added a footnote to TS Tables 3.7-2, 3.7-3, and 3.7-4, which states the following:

There is a Safety Analysis Limit associated with this ESF function. If during calibration the setpoint is found to be conservative with respect to the Setting Limit but outside its predefined calibration tolerance, then the channel shall be brought back to within its predefined calibration tolerance before returning the channel to service. The calibration tolerances are specified in a document controlled under 10 CFR 50.59.

The licensee has applied this footnote to the following functions in TS Table 3.7-2:

- Functional Unit 1.b High containment pressure
- Functional Unit 1.c High differential pressure between any steam line and the steam header
- Functional Unit 1.d Pressurizer low-low pressure
- Functional Unit 1.e High steam flow in 2/3 steam lines coincident with low  $T_{avg}$  or low steam line pressure
  - Functional Unit 1.e.1 Steam line flow
  - Functional Unit 1.e.2  $T_{avg}$
  - Functional Unit 1.e.3 Steam line pressure
- Functional Unit 2.b High containment pressure (Hi-Hi)

- Functional Unit 3.a Steam generator water level low-low
- Functional Unit 5.a Low intake canal level
- Functional Unit 7.a RWST Level - Low-Low
- Functional Unit 8.a RWST Level - Low Coincident with High-High Containment Pressure

The licensee has applied this footnote to the following functions in TS Table 3.7-3:

- Functional Unit 1.b.1 High containment pressure
- Functional Unit 1.c.1 High containment pressure (Hi-Hi setpoint)
- Functional Unit 2.a High steam flow in 2/3 lines coincident with 2/3 low  $T_{avg}$  or 2/3 low steam pressures
- Functional Unit 2.b High containment pressure (Hi-Hi setpoint)
- Functional Unit 3.a Steam generator water-level high-high

The licensee has applied this footnote to the following functions in TS Table 3.7-4:

- Functional Unit 1 High Containment Pressure (High Containment Pressure Signal)
- Functional Unit 2 High-High Containment Pressure (High-High Containment Pressure Signals)
- Functional Unit 3 Pressurizer Low-Low Pressure
- Functional Unit 4 High Differential Pressure between Steam Line and the Steam Line Header
- Functional Unit 5 High Steam Flow in 2/3 Steam Lines Coincident with Low  $T_{avg}$  or Low Steam Line Pressure
- Functional Unit 6 Steam Generator Water Level Low-Low
- Functional Unit 8.a Low Intake Canal Level
- Functional Unit 9.a RWST Level-Low-Low
- Functional Unit 10.a Steam Generator Water Level High-High
- Functional Unit 11 RWST Level Low (Coincident with High High Containment Pressure)

The NRC staff finds that the proposed note will add operability requirements for all these functions as required by RIS 2006-17. Therefore, the NRC staff finds the proposed change acceptable.

On the basis of its review of the licensee's submittal, the NRC staff has concluded that the instrument setpoint methodology for the proposed TS changes is acceptable. The NRC staff further concludes that by adding footnotes to the TS tables, the TSs rather than the procedures will control instrument operability. The NRC staff also concludes that the proposed TS changes specified in Section 1.0 of this safety evaluation meet the requirements of 10 CFR 50.36(d)(1)(ii)(A) as the licensee's setpoint methodology meets the NRC staff's guidance provided in RIS 2006-17, and, therefore, are acceptable.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Virginia State official was notified of the proposed issuance of the amendments. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (72 FR 60036). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: H. Garg

Date: September 17, 2008



Surry Power Station, Unit Nos. 1 & 2

cc:

Mr. David A. Christian  
President and Chief Nuclear Officer  
Virginia Electrical and Power Company  
Innsbrook Technical Center  
5000 Dominion Boulevard  
Glen Allen, VA 23060-6711

Ms. Lillian M. Cuoco, Esq.  
Senior Counsel  
Dominion Resources Services, Inc.  
120 Tredegar Street, RS-2  
Richmond, VA 23219

Mr. Donald E. Jernigan  
Site Vice President  
Surry Power Station  
Virginia Electric and Power Company  
5570 Hog Island Road  
Surry, Virginia 23883-0315

Senior Resident Inspector  
Surry Power Station  
U. S. Nuclear Regulatory Commission  
5850 Hog Island Road  
Surry, Virginia 23883

Chairman  
Board of Supervisors of Surry County  
Surry County Courthouse  
Surry, Virginia 23683

Dr. W. T. Lough  
Virginia State Corporation Commission  
Division of Energy Regulation  
Post Office Box 1197  
Richmond, Virginia 23218

Dr. Robert B. Stroube, MD, MPH  
State Health Commissioner  
Office of the Commissioner  
Virginia Department of Health  
Post Office Box 2448  
Richmond, Virginia 23218

Office of the Attorney  
General Commonwealth of Virginia  
900 East Main Street  
Richmond, Virginia 23219

Mr. Chris L. Funderburk, Director  
Nuclear Licensing & Operations Support  
Innsbrook Technical Center  
5000 Dominion Blvd.  
Glen Allen, Virginia 23060-6711

Mr. Matt Adams, Director  
Nuclear Safety and Licensing  
Virginia Electric and Power Company  
Surry Power Station  
5570 Hog Island Road  
Surry, Virginia 23883-0315

Mr. Curtis G. Luffman  
Manager Nuclear Protection Services  
Surry Power Station  
5570 Hog Island Road  
Surry, VA 23883-0315

Mr. Barry Garber  
Supervisor Nuclear Engineering  
Surry Power Station  
5570 Hog Island Road  
Surry, VA 23883-0315

Ms. Leslie N. Hartz, Vice President  
Nuclear Support Services  
Innsbrook Technical Center  
5000 Dominion Boulevard  
Glen Allen, VA 23060-6711

Mr. Nelson K. Martin  
Manager Nuclear Fleet Protection Services  
Innsbrook Technical Center  
5000 Dominion Boulevard  
Glen Allen, VA 23060-6711