UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
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ARLINGTON, TEXAS 76011-4125

July 24, 2008

Rick A. Muench, President and
Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
P.O. Box 411
Burlington, KS 66839

SUBJECT: WOLF CREEK GENERATING STATION - NRC INTEGRATED INSPECTION REPORT 05000482/2008002 ERRATA

Dear Muench:

Please replace page 38 of the Report Details and page 2 of the Supplemental Information in NRC Inspection Report 05000428/2008002, dated May 21, 2008, with the enclosed revised pages. These changes are needed to properly close Licensee Event Report 2008-002-00, associated with the CCP A Room Cooler system.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records component of NRC's document system (ADAMS), accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

Sincerely,

/RA/ Rick W. Deese for

Vincent G. Gaddy, Chief
Project Branch B
Division of Reactor Projects

Docket: 50-482
License: NPF-42

Enclosure: As stated

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SUNSI Review Completed: VGG ADAMS: ☑ Yes □ No Initials: VGG
☑Publicly Available □ Non-Publicly Available □ Sensitive ☑ Non-Sensitive
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SRI:DRP/B	BC:DRP/B		
SDCochrum	VGGaddy		
/RA/	***/RA/ RWDeese for***		
7/17/08	7/24/08		

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Pending completion of the licensee's root cause determination and consequence assessment by a Region IV Senior Reactor Analyst, additional inspection of the finding is needed to determine significance. This issue is considered unresolved pending additional NRC review of Wolf Creek root cause determination. This issue will be tracked as: Unresolved Item (URI) 05000483/2008002-08, Transformer Trip Resulted in an Unplanned Reactor Trip and Forced Outage.

.4 (Closed) LER 05000482/2008-002-00, Technical Specification Allowed Outage Time Exceeded Due to Room Cooler Leak

The inspectors reviewed LER 05000482/2008-002-00 to verify that the cause of the Train A CCP exceeding its allowed outage time was identified and that corrective actions were appropriate. See Section 4OA3.3 for additional information on the event and enforcement actions taken. See also "Notice of Enforcement Discretion for Wolf Creek Nuclear Operating Corporation Regarding Wolf Creek Generating Station [TAC No MD8098, NOED No. 08-4-001]," under ADAMS Accession No. ML080520023 for more information regarding the NOED. This LER is closed.

.5 (Closed) LER 05000482/2005-006-00: Unanalyzed Condition Related to Loss of Reactor Coolant Pump Seal Cooling during a Postulated Appendix R Fire Event

Introduction. The inspectors identified an NCV of Technical Specification 5.4.1.d because Procedure OFN-RP-017, "Control Room Evacuation," Revision 21, failed to ensure that operators took the required actions to reestablish reactor coolant pump seal cooling in a timely manner. Failure to establish seal cooling in a timely manner could have resulted in a small break loss of coolant accident. This finding was determined to be of very low risk significance (Green).

Description. While timing operator actions during a 2005 triennial fire protection inspection (NRC Inspection Report 05000482/2005008, Section 1R05.6.b(2)), the inspectors determined that control room operators could not reestablish seal cooling to the reactor coolant pumps in a timely manner. The failure to reestablish seal cooling within 21 minutes would degrade the seals and could result in a small break loss of coolant accident. The delay in reestablishing seal cooling to the reactor coolant pumps allows the seals to overheat and the subsequent flow of relatively cool water shatters the seals and allows for excessive leakage. Specifically, the inspectors postulated circuit failures that required operators to start the Train B EDG manually, as specified in Procedure OFN-RP-017, Attachment C, Step 10, and manually open Valve BN-LCV-112E, Train B CCP suction from the refueling water storage tank, as specified in Attachment C, Step 24.

The licensee indicated that they had planned to revise Procedure OFN-RP-017 in response to information contained in Information Notice 2005 14, "Fire Protection Findings on Loss of Seal Cooling to Westinghouse Reactor Coolant Pumps," dated June 5, 2005, and Westinghouse WCAP-16396 NP, "Reactor Coolant Pump Seal Performance for Appendix R Assessments," dated January 2005. The licensee reported that the NRC used a more conservative approach to develop the time line for reestablishing seal cooling to the reactor coolant pumps than they had previously used.

The failure to ensure that operators could reestablish seal cooling to the reactor coolant pumps within the prescribed time could cause failure of the pump seals and increase the

05000482/2008002-10	NCV	Failure to analyze motor-operated valve circuits (Section 4OA5.2)

Closed

05000482/2008-002-00	LER	Technical Specification Allowed Outage Time Exceeded Due to Room Cooler Leak (Section 4OA3.4)
05000482/2005-006-00	LER	Unanalyzed Condition Related to Loss of Reactor Coolant Pump Seal Cooling during a Postulated Appendix R Fire Event (Section 4OA3.5)
05000482/2005-007-00	LER	Unanalyzed Condition Related to Loss of EDG Field Flashing during an Appendix R Fire Event (Section 4OA3.6)
05000482/2006-001-00	LER	Potential for Fire Induced Damage to Motor Operated Valves during an Appendix R Fire Event (Section 4OA3.7)
05000482/2006-002-00	LER	Potential for Fire Induced Damage to Class 1E Electrical Equipment Air Conditioning Units during an Appendix R Event (Section 4OA3.8)
05000482/2005008-05	AV	Inadequate Alternative Shutdown Procedure (Section 4OA5.1)
05000482/2005008-06	URI	Failure to Adequately Evaluate Fire Protection Program Deficiencies (Section 4OA5.2)

LIST OF DOCUMENTS REVIEWED

In addition to the documents referred to in the inspection report, the following documents were selected and reviewed by the inspectors to accomplish the objectives and scope of the inspection and to support any findings:

Section 1R01: Adverse Weather

Procedures

STN EF-020B, ESW Train B Warming Line Verification, Revision 6
SYS EF-205, ESW/CIRC Water Cold Weather Operations, Revision 19
AI 14-006, Severe Weather, Revision 7

Section 1R04: Equipment Alignment

Procedures

CKL EF-120, Essential Service Water Valve, Breaker and Switch Lineup, Revision 41
SYS KJ-121,Diesel Generator Lineup for Auto Ops, Revision 39