FPL Energy Point Beach, LLC, 6610 Nuclear Road, Two Rivers, WI 54241

FPL Energy
Point Beach Nuclear Plant

July 18, 2008

NRC 2008-0056
10 CFR 50.90

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Point Beach Nuclear Plant, Unit 1
Docket 50-266
Renewed License No. DPR-24

Supplement to License Amendment Request 257
Technical Specifications 5.5.8 and 5.6.8,
Steam Generator Program & Steam Generator Tube Inspection Report
Interim Alternate Repair Criteria (IARC) for Steam Generator Tube Repair

Reference: (1) FPL Energy Point Beach Letter to NRC Dated May 28, 2008, License Amendment Request 257, Technical Specifications 5.5.8 and 5.6.8, Steam Generator Program & Steam Generator Tube Inspection Report, Interim Alternate Repair Criteria (IARC) for Steam Generator Tube Repair (ML081560178)

In accordance with the provisions of 10 CFR 50.90, "Application for amendment of license, construction permit, or early site permit," FPL Energy Point Beach, LLC is submitting a supplement to the request for an amendment to the Point Beach Nuclear Plant (PBNP) Technical Specifications (TS) for Facility Operating License DPR-24, as described in Reference (1).

This amendment proposes a one cycle revision to the PBNP TS. Specifically, TS 5.5.8, "Steam Generator (SG) Program," and TS 5.6.8, "Steam Generator Tube Inspection Report," will be revised to incorporate an interim alternate repair criterion into the provisions for SG tube repair for use during the PBNP Unit 1, 2008 fall refueling outage (U1R31) and the subsequent operating cycle. The amendment reflects recent industry efforts, including the May 14, 2008, meeting between the Commission and the industry, to resolve technical issues associated with the interim alternate repair criterion.

The purpose of the supplement to the application is to formalize, as a regulatory commitment, FPL Energy Point Beach's intention to use a ratio of 2.5 in the completion of both the condition monitoring and operational integrity assessments upon implementation of the Interim Alternate Repair Criteria (IARC). This supplement also clarifies the additions to TS 5.6.8 to clearly indicate applicability to PBNP Unit 1 only. Enclosure 1 provides the revised marked-up of the proposed TS 5.6.8 additions.

This supplement does not alter the no significant hazards or the environmental considerations previously provided in Reference (1). This license amendment request supplement has been reviewed by the Plant Operations Review Committee.

As stated in Reference (1), FPL Energy Point Beach requests approval of the proposed license amendment by October 1, 2008, to support the fall PBNP Unit 1 refueling outage, which is currently scheduled to start in October 2008. Once approved, the amendment will be implemented prior to entering MODE 4 during startup of PBNP Unit 1 from the refueling outage.

Summary of Regulatory Commitments

The following new Regulatory Commitment is proposed in support of this license amendment request:

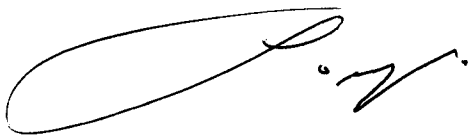
- The ratio of 2.5 will be used in completion of both the condition monitoring (CM) and operational assessment (OA) upon implementation of the IARC. For example, for the CM assessment, the component of leakage from the lower 4 inches for the most limiting steam generator during the prior cycle of operation will be multiplied by a factor of 2.5 and added to the total leakage from any other source and compared to the allowable accident analysis leakage assumption. For the OA, the difference in leakage from the allowable limit during the limiting design basis accident minus the leakage from the other sources will be divided by 2.5 and compared to the observed leakage. An administrative limit will be established to not exceed the calculated value.

In accordance with 10 CFR 50.91, a copy of this application, with enclosures, is being provided to the designated State of Wisconsin Official.

I declare under penalty of perjury that the foregoing is true and correct.
Executed on July 18, 2008.

Very truly yours,

FPL ENERGY POINT BEACH, LLC



Larry Meyer
Site Vice President

Enclosures

cc: Administrator, Region III, USNRC
Project Manager, Point Beach Nuclear Plant, USNRC
Resident Inspector, Point Beach Nuclear Plant, USNRC
PSCW

ENCLOSURE 1

FPL ENERGY POINT BEACH, LLC
POINT BEACH NUCLEAR PLANT, UNIT 1

LICENSE AMENDMENT REQUEST 257

INTERIM ALTERNATE REPAIR CRITERIA (IARC)
FOR STEAM GENERATOR TUBE REPAIR

REVISED TECHNICAL SPECIFICATION 5.6.8 MARKUP

2 pages follow

5.6.7 Tendon Surveillance Report (continued)

Nuclear Regulatory Commission pursuant to the requirements of 10 CFR 50.4 within thirty days of that determination. Other conditions that indicate possible effects on the integrity of two or more tendons shall be reportable in the same manner. Such reports shall include a description of the tendon condition, the condition of the concrete (especially at tendon anchorages), the inspection procedure and the corrective action taken.

5.6.8 Steam Generator Tube Inspection Report

A report shall be submitted within 180 days after the initial entry into MODE 4 following completion of an inspection performed in accordance with the Specification 5.5.8, Steam Generator (SG) Program. The report shall include:

a. The scope of inspections performed on each SG,

b. Active degradation mechanisms found,

c. Nondestructive examination techniques utilized for each degradation mechanism,

d. Location, orientation (if linear), and measured sizes (if available) of service induced indications,

e. Number of tubes plugged during the inspection outage for each active degradation mechanism,

f. Total number and percentage of tubes plugged to date,

g. The results of condition monitoring, including the results of tube pulls and in-situ testing, and

h. The effective plugging percentage for all plugging in each SG.

i. Following completion of an inspection performed in Unit 1 Refueling Outage 31 (and any inspections performed in the subsequent operating cycle), the number of indications and location, size, orientation, whether initiated on primary or secondary side for each service-induced flaw within the thickness of the tubesheet, and the total of the circumferential components and any circumferential overlap below 17 inches from the top of the tubesheet as determined in accordance with TS 5.5.8,

5.6 Reporting Requirements

j. Following completion of an inspection performed in Unit 1 Refueling Outage 31 (and any inspections performed in the subsequent operating cycle), the primary to secondary LEAKAGE rate observed in each steam generator (if it is not practical to assign leakage to an individual SG, the entire primary to secondary LEAKAGE should be conservatively assumed to be from one steam generator) during the cycle preceding the inspection which is the subject of the report, and

k. Following completion of an inspection performed in Unit 1 Refueling Outage 31 (and any inspections performed in the subsequent operating cycle), the calculated accident leakage rate from the portion of the tube below 17 inches from the top of the tubesheet for the most limiting accident in the most limiting steam generator.