July 21, 2008

Mr. William R. Campbell, Jr.
Chief Nuclear Officer and
Executive Vice President
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

SUBJECT: SEQUOYAH NUCLEAR PLANT, UNIT 1 - REQUEST FOR ADDITIONAL INFORMATION REGARDING STEAM GENERATOR TUBE INSERVICE INSPECTION REPORT FOR THE CYCLE 15 REFUELING OUTAGE (TAC NO. MD8755)

Dear Mr. Campbell:

By letter dated April 23, 2008, Tennessee Valley Authority (TVA) submitted information summarizing the results of the fall 2007 steam generator (SG) tube inspections performed at Sequoyah Nuclear Plant Unit 1 during the Cycle 15 refueling outage.

In order for the staff to complete its review of the information provided, we request that TVA provide responses to the enclosed request for additional information (RAI). Based on discussions with your staff, we understand that you plan to respond to the enclosed RAI within 30 days of receipt.

If you have any questions about this material, please contact me at (301) 415-3974.

Sincerely,

/RA/

Brendan T. Moroney, Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-327

Enclosure: Request for Additional Information

cc: See next page

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REQUEST FOR ADDITIONAL INFORMATION

STEAM GENERATOR TUBE INSERVICE INSPECTION REPORT

FOR THE CYCLE 15 REFUELING OUTAGE

SEQUOYAH NUCLEAR PLANT, UNIT 1

By letter dated April 23, 2008, Tennessee Valley Authority submitted information summarizing the results of the fall 2007 steam generator (SG) tube inspections performed at Sequoyah Nuclear Plant Unit 1 during the Cycle 15 refueling outage. In order for the staff to complete its review, the following additional information is requested:

1. For each refueling outage and SG tube inspection since installation of the SGs, please provide the cumulative effective full power years that the SGs have operated.
2. Please discuss the scope and results of any secondary side inspections.
3. It was indicated that the scope of the bobbin coil examinations were expanded in SGs 1, 2, and 3. Please discuss the reason for expanding the scope of inspection in these three SGs.
4. Please discuss what types of indications/locations are included in the diagnostic +Point™ inspections. Please discuss how you concluded that the bobbin indications were a result of U-bend wear (i.e., were rotating probe inspections (or comparable inspections) performed at these locations to permit characterization of the bobbin signal so as to confirm that there was a volumetric indication associated with the tube support "contact" points?).

Enclosure

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SEQUOYAH NUCLEAR PLANT

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